

NOV 26 1976

Docket No. 50-219

Jersey Central Power & Light Company
ATTN: Mr. I. R. Finrock, Jr.
Vice President - Generation
Madison Avenue at Punch Bowl Road
Morristown, New Jersey 07960

Gentlemen:

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The Commission has issued the enclosed Amendment No. 17 to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station. This amendment consists of changes to the Technical Specifications and is in response to your application dated May 20, 1976.

The amendment to the Technical Specifications will revise the present structure for Safety Review and Audit and will implement a revised method by which independent review and audit of facility operations will be accomplished.

Copies of the related Safety Evaluation and the Federal Register Notice also are enclosed.

Sincerely,

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:

- 1. Amendment No. 17 to License DPR-16
- 2. Safety Evaluation
- 3. Federal Register Notice

cc:
See next page

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SURNAME	CParrish	JShea	MW	GLear	Overell
DATE	11/18/76	11/18/76	11/24/76	11/18/76	11/18/76

Jersey Central Power & Light Company - 2 -

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Jersey Central Power & Light Company
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION, UNIT NO. 1

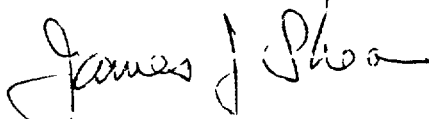
AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 17
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Jersey Central Power and Light Company (the licensee) dated May 20, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.

3. This license amendment is effective 60 days from the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

for

Attachment:
Changes to the
Technical Specifications

Date of Issuance: November 26, 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 17
TO THE TECHNICAL SPECIFICATIONS
PROVISIONAL OPERATING LICENSE NO. DPR-16
DOCKET NO. 50-219

Replace pages 6-3 and 6-5 through 6-9 with the attached revised pages.

Add pages 6-9a, 6-9b and 6-9c.

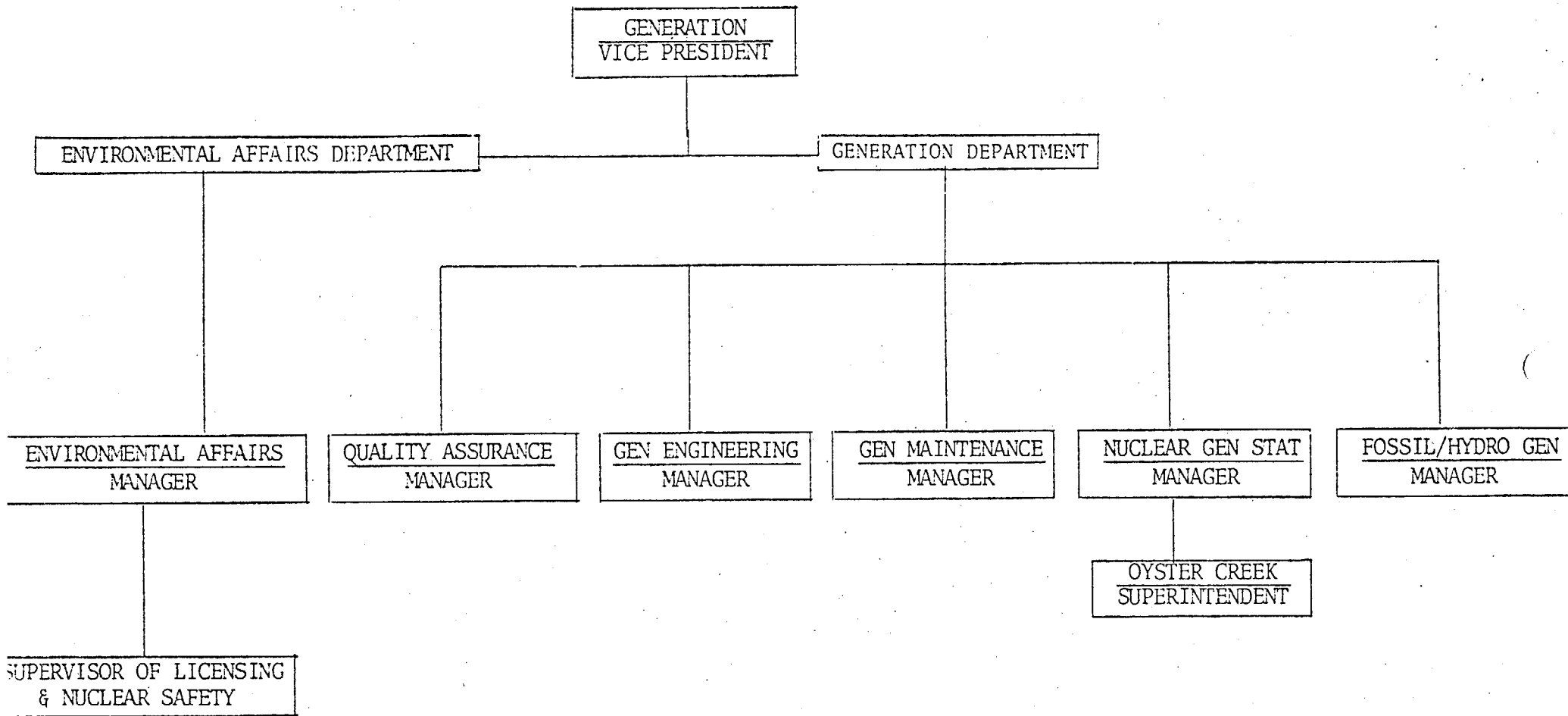


FIGURE 6.2.1
 OYSTER CREEK STATION
 MANAGEMENT ORGANIZATION CHART

6.5 SAFETY REVIEW AND AUDIT

The Station Superintendent and three organizational units, the Plant Operations Review Committee (PORC), the Independent Safety Review Groups (ISRG) and the General Office Review Board (GORB) function to accomplish nuclear safety review and audit of the Oyster Creek Station.

6.5.1 Station Superintendent

FUNCTION

6.5.1.1 The Station Superintendent shall ensure that:

- a. All proposed changes to equipment or systems have been evaluated to determine if they constitute a change to the facility or procedures as described in the Safety Analysis Report.
- b. All proposed changes to equipment or systems that constitute a change of the facility or procedures as described in the Safety Analysis Report have been evaluated to determine whether they involve an unreviewed safety question as defined in paragraph 50.59, Part 50, Title 10, Code of Federal Regulations.
- c. All proposed tests and experiments have been evaluated to determine whether or not they involve unreviewed safety questions as defined in paragraph 50.59, Part 50, Title 10, Code of Federal Regulations.

AUTHORITY

6.5.1.2 The Station Superintendent has the authority to:

- a. Make a determination as to whether proposed changes to equipment, or systems involve a change to the procedures or facility as described in the Safety Analysis Report.
- b. Make a determination as to whether or not proposed tests or experiments and changes to equipment or systems involve an unreviewed safety question.
- c. Direct the Plant Operations Review Committee to review safety evaluations of proposed changes to equipment or systems and safety evaluations of proposed tests and experiments to determine whether or not such changes, tests or experiments involve unreviewed safety questions.

NOTE: Each determination that a proposed test, experiment, or change to a system or equipment that does not involve an unreviewed safety question shall be reviewed by the Independent Safety Review Groups to verify that the determination was correct. This review shall be documented but is not a pre-requisite of the test, experiment, or change to a system or equipment.

RECORDS

6.5.1.3 Any safety evaluations done in accordance with 6.5.1.1 (b) and (c) and any determinations made pursuant to 6.5.1.2(b) must be documented. Copies of these determinations shall be provided to the ISRG Coordinator and the Chairman of the General Office Review Board. Records of all tests and experiments performed and all changes to equipment or systems made under the provisions of 10 CFR Part 50.59 shall also be maintained at the station.

6.5.2 Plant Operations Review Committee (PORC)

FUNCTION

6.5.2.1 The PORC shall function to advise the Station Superintendent.

COMPOSITION

6.5.2.2 The PORC shall consist of the following plant personnel:

Chief Engineer
Operations Engineer
Maintenance Engineer
Technical Engineer
Supervisor Radiation Protection

The Station Superintendent shall designate the Chairman and Vice-Chairman from among the PORC members.

ALTERNATES

6.5.2.3 Alternate members shall be appointed in writing by the PORC Chairman and will have the type of experience and training required of regular members. However, they need not have the extensive longevity in the designated fields as long as in the opinion of the Chairman, their experience and judgement are adequate.

MEETING FREQUENCY

6.5.2.4 The PORC shall meet at least once per calendar month and as convened by the PORC Chairman or the Station Superintendent.

QUORUM

6.5.2.5 A quorum of the PORC shall consist of the Chairman or Vice Chairman and three members/alternates. No more than two alternate members shall be counted in establishing a quorum.

RESPONSIBILITIES

6.5.2.6 The responsibilities of the PORC are included in Table 6.5-1.

AUTHORITY

- 6.5.2.7 a. The PORC shall be advisory to the Station Superintendent. Nothing herein shall relieve the Station Superintendent of his responsibility or authority for overall safe operations including taking immediate emergency action. Determinations on items a and b of Table 6.5-1 shall be documented in writing.
- b. The PORC Chairman shall immediately notify the Manager, Generating Stations - Nuclear and the Chairman of the GORB of any disagreements between the PORC and the Station Superintendent. However, the Station Superintendent shall have the responsibility for resolution of such disagreements pursuant to 6.1.1.

RECORDS

- 6.5.2.8 The PORC shall maintain written minutes of each meeting and copies of minutes and determinations shall be provided to the Station Superintendent; Manager, Generating Stations - Nuclear; ISRG Coordinator and the Chairman of the GORB.

6.5.3 Independent Safety Review Groups (ISRG)

FUNCTION AND COMPOSITION

- 6.5.3.1 The ISRG shall function under the direction of an ISRG Coordinator, who shall be appointed by the Vice-President-Generation, to provide safety reviews. The Coordinator shall have available the competence to review problems in the following areas:
- a. Nuclear Power Plant Operations
 - b. Nuclear Engineering
 - c. Chemistry and Radiochemistry
 - d. Metallurgy
 - e. Instrumentation and Control
 - f. Radiological Safety
 - g. Mechanical and Electrical Engineering
 - h. Quality Assurance Practices

The Coordinator shall establish, as needed, groups of two or more individuals with the expertise required for each topic to be reviewed.

CONSULTANTS

- 6.5.3.2 Consultants shall be utilized as necessary to supplement the expertise available in the Company.

RESPONSIBILITIES

6.5.3.3 The specific responsibility to ensure accomplishment of independent safety review of the plant superintendent's determinations involving safety questions is assigned to the ISRG Coordinator and is accomplished by utilizing, as necessary, the full scope of expertise available in the generation department staff, consultants, contractors and vendors as appropriate. Table 6.5-1 defines the specific independent safety review responsibilities.

AUTHORITY

6.5.3.4 The ISRG advises the Vice President-Generation. It has the authority to conduct reviews and investigations, which will be documented.

AUDITS

6.5.3.5 Audits of facility activities shall be performed under the cognizance of the Manager, Operational Quality Assurance. These audits shall encompass:

- a. The conformance of facility operation to all provisions contained within the Technical Specifications and applicable license conditions at least once per year.
- b. The training and qualifications of the entire facility staff at least once per year.
- c. The results of all actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per six months.
- d. The Facility Emergency Plan and implementing procedures at least once per two years.
- e. The Facility Security Plan and implementing procedures at least once per two years.
- f. Any other area of facility operation considered appropriate by the GORB or the Vice President - Generation.

RECORDS

6.5.3.6 Written documentation of all independent safety reviews and investigations will be forwarded to the Station Superintendent, Vice President - Generation and the Chairman of the General Office Review Board. In addition, any reportable occurrence or item involving an unreviewed safety question which is identified by the ISRG will be documented and reported immediately to the above mentioned persons.

The results of all audits conducted in accordance with Section 6.5.3.5 shall be documented and reported to the above mentioned persons within 30 days after completing the audit. Reports documenting corrective action will receive the same distribution and they will also be forwarded to the ISRG Coordinator.

6.5.4 General Office Review Board (GORB)

FUNCTION

6.5.4.1 The technical and administrative function of the GORB is to provide independent review of major safety issues, to foresee potentially significant nuclear and radiation safety problems, and to advise the Company President on these matters.

COMPOSITION

6.5.4.2 Members of the General Office Review Board shall possess extensive experience in their individual specialities and collectively have the competence in the following areas:

- a. Nuclear Power Plant Operations
- b. Nuclear Engineering
- c. Chemistry and Radiochemistry
- d. Metallurgy
- e. Instrumentation and Control
- f. Radiological Safety
- g. Mechanical and Electric Engineering.

The Chairman and Vice Chairman shall be appointed by the Company President. (Neither shall be an individual with line responsibility for operation of the plant).

The Chairman shall designate a minimum of six additional members. No more than a minority of the Board shall have line responsibility for operation of Oyster Creek Nuclear Generating Station.

ALTERNATES

6.5.4.3 Alternate members shall be appointed in writing by the GORB Chairman and will have the type of experience and training required of regular members, however, they need not have the extensive longevity in the designated fields as long as, in the opinion of the Chairman, their experience and judgement are adequate.

MEETING FREQUENCY

6.5.4.4 The GORB shall meet at least semi-annually and any time at the request of the Chairman or the Company President.

QUORUM

- 6.5.4.5 A quorum shall consist of the Chairman or Vice Chairman and three members/alternates. No more than one alternate member shall be counted when establishing a quorum and no more than a minority of the quorum shall hold line responsibility for operations of the Oyster Creek Station.

RESPONSIBILITIES

- 6.5.4.6 a. The primary responsibility of the GORB is to foresee potentially significant nuclear and radiation safety problems and to recommend to the Company President how they may be avoided or mitigated.
- b. Carry out the specific independent safety review responsibilities listed in Table 6.5-1.

AUTHORITY

- 6.5.4.7 The GORB shall be advisory to the Company President and shall have the authority to conduct reviews, audits, and investigations as requested by the Company President or as deemed necessary by the GORB in the fulfillment of its responsibilities.

AUDITS

- 6.5.4.8 The report of the management review of the QA Plan, initiated by the Vice President, Generation, in accordance with the Operational Quality Assurance Plan, shall be reviewed by the GORB with respect to safety and administrative safety issues.

RECORDS

- 6.5.4.9 Minutes of each GORB meeting shall be recorded and approved by the GORB Chairman. Copies of approved minutes will be forwarded to the Company President, Vice President - Generation, Station Superintendent, PORC Chairman and others designated by the GORB Chairman. GORB recommendations to the Company President will be documented in a letter from the GORB Chairman to the President. Included with each letter will be any dissenting opinions of members of the Board.

SAFETY REVIEW RESPONSIBILITIES

ITEM	INITIAL ACTION	PORC	INDEPENDENT REVIEW	
			ISRG	GORB
a) Proposed change to equipment or systems subject to the Provisions of Section 50.59, Part 50, Title 10 Code of Federal Regulations.	Initiator: Must prepare a complete description of the proposed changes and ensure a safety evaluation of the change is included. Station Superintendent: (1) Must determine if the item is an actual change to equipment or systems as described in the FSAR. (2) Must determine if the item involves an unreviewed safety question. (3) May request the PORC to assist in the above determinations.	Must review item to determine whether or not an unreviewed safety question is involved, if requested by the Superintendent.	Must review all determinations by the Station Superintendent.	May review any determination, but must review those for which the Station Superintendent has requested GORB review.
b) Proposed tests and experiments, subject to provisions of 50.59, Title 10, Code of Federal Regulations.	Initiator: Must prepare a complete description of the proposed test or experiment and ensure a safety evaluation of the test or experiment is included. Station Superintendent: (1) Must determine if the item involves an unreviewed safety question. (2) May request the PORC to assist in the above determinations.	Must review item to determine whether or not an unreviewed safety question is involved, if requested by the Station Superintendent.	As Above.	As Above.
c) Proposed changes in Technical Specifications or in the NRC Operating License.	Initiator: Must prepare a complete description of the proposed change and ensure a safety evaluation of the change is included.	(1) Must review the item for nuclear and radiological safety. (2) Must make recommendations to the Station Superintendent as to whether or not the change is safe.	Must review change and PORC recommendation prior to submittal to the NRC.	May review any item but must review those for which the Station Superintendent or his superiors have requested GORB review.
d) Reportable occurrences.	Station Superintendent: Must have investigations performed for all Reportable Occurrences and a report prepared including the safety significance of the incident.	Must review Reportable Occurrences Report for safety significance and make recommendations to the Station Superintendent on how to avoid recurrence.	Must review Reportable Occurrence Reports for safety significance and review PORC recommendations.	As Above

SAFETY REVIEW RESPONSIBILITIES

ITEM	INITIAL ACTION	PORC	INDEPENDENT REVIEW	
			ISRG	GORB
e) Facility Operations including Security Plan, Emergency Plan and implementing procedures; review is to detect potential safety hazards.		Continuing responsibility	See item i below.	As Above.
f) Significant operation abnormalities or deviations from normal and expected performance.	Station Superintendent: Report such matters to the PORC, ISRG Coordinator, and the Chairman. GORB.	Review matter and report evaluation of safety significance to the ISRG and GORB.	Perform independent review of PORC evaluation.	As Above.
g) Any indication of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems or components.	As Above.	As Above.	As Above.	As Above.
h) PORC Minutes and reports.			Review to determine if any matters discussed involve unreviewed safety questions.	As Above.
i) Audit Reports and NRC Inspection Reports.			Review to determine if any matters reported involve Violations of Technical Specifications, license requirements or regulations or have any nuclear or radiation safety implications.	The report of the management review of the QA Plan, initiated by the Vice President, Generation, in accordance with the Operational Quality Assurance Plan, shall be reviewed by the GORB with respect to technical and administrative safety issues.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 17 TO FACILITY LICENSE NO. DPR-16

JERSEY CENTRAL POWER AND LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

Introduction

By letter dated May 20, 1976, Jersey Central Power and Light Company proposed changes to the Technical Specifications for the Oyster Creek Nuclear Generating Station, Unit No. 1. The proposed changes would revise the present structure for Safety Review and Audit at the Oyster Creek Station and implement a revised method by which independent review and audit of facility operations would be accomplished.

Evaluation

The proposed changes are administrative in nature and are the result of Jersey Central's evaluation of their six years experience of review and audit of facility operations. Areas covered by the proposed specification include organizational structure, responsibilities, functions, authority and documentation of safety reviews. The modified structure is consistent with the intent of the Regulatory Staff's Standard Technical Specifications Section 6.5, "Review and Audit." Specifically, the proposed change differs from the model specifications in two respects: (1) three organizational units are proposed for nuclear reviews and audits in lieu of two organizations as suggested in the model specifications, and (2) the Station Superintendent is not a member of the advisory review groups.

The proposed specification separates line and advisory responsibility for both onsite and offsite safety review. The major responsibility for safe operation of the station rests with the Station Superintendent. He initiates the majority of safety review activities by an advisory group, the "Plant Operations Review Committee" (PORC). The Station Superintendent is not a member of the PORC, thereby permitting the PORC to function as an autonomous review group. Offsite, line responsibility rests with the Vice President, Generation and the Company President. They are advised by two advisory groups: Independent Safety Review Groups (ISRG) and the General Office Review Board (GORB). The ISRG performs routine review and audit functions. The GORB activities concentrate on overall safety and major or potentially significant safety problems.

We have concluded that the proposed changes improve the licensee's program for evaluating plant performance and the reporting of information needed by the Commission to assess safety related activities and is acceptable. The administrative controls are consistent with requirements being incorporated in Technical Specifications for new licensed facilities.

Environmental Consideration

We have determined that this amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: November 26, 1976

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-219

JERSEY CENTRAL POWER & LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENT

TO PROVISIONAL OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 17 to Provisional Operating License No. DPR-16 issued to Jersey Central Power & Light Company which revised Technical Specifications for operation of the Oyster Creek Nuclear Generating Station, located in Ocean County, New Jersey. The amendment is effective 60 days from its date of issuance.

The amendment to the Technical Specifications will revise the present structure for Safety Review and Audit and will implement a revised method by which independent review and audit of facility operations will be accomplished.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and

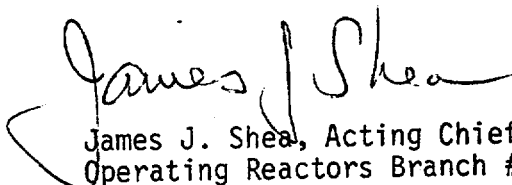
environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated May 20, 1976, (2) Amendment No. 17 to License No. DPR-16, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Ocean County Library, 15 Hopper Avenue, Toms River, New Jersey 08753.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 26 day of November 1976.

FOR THE NUCLEAR REGULATORY COMMISSION



James J. Shea, Acting Chief
Operating Reactors Branch #3
Division of Operating Reactors