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Docket No. 50-219

Mr. I. R. Finfrock, Jr.
Vice President - Generation
Jersey Central Power & Light Company
Madison Avenue at Punch Bowl Road
Morristown, New Jersey 07960

Dear Mr. Finfrock:

The Commission has issued the enclosed Amendment No. 38 to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station. The amendment consists of changes to the Appendix A Technical Specifications in response to your request dated May 23, 1979 (Addendum 1 to Change Request No. 72 dated May 19, 1979), as supported by your letter dated May 19, 1979, and the enclosed analysis titled "Bounding Loss of Coolant Inventory Transient for the Oyster Creek Plant."

The amendment: 1) adds a limiting safety system setting for the automatic initiation of the Isolation Condenser on a reactor low-low water level signal (Section 2.3), 2) revises Table 3.1.1 to reflect the change in 1) above, 3) adds the Assembly Average Power Void Relationship for all fuel types in the reactor (Section 3.10.c), 4) removes Figure 3.10-2 and the reference to Figure 3.10-2 from Section 3.10.A, and 5) removes the word interim from Figure 3.10-9, which was inadvertently included in the title of this Figure during the issuance of License Amendment No. 35, dated May 30, 1979.

Copies of our related Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original Signed by:
Dennis L. Ziemann

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Enclosures:
1. Amendment No. 38 to DPR-16
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures: See next page

DOR:ORB#2
HSmith:sah
7/12/79

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OFFICE			DOR:ORB#2	OELD	DOR:ORB#2	DOR:ADSEP
SURNAME			SNowicki	Woodhead	DLZiemann	RHVollmer
DATE			7/12/79	7/16/79	7/20/79	7/20/79

file — additional limiting safety system settings, Section 2.3 (Add. dum/ to 5-19-79 appl.)

William O. Miller, Chief
License Fee Management Branch, ADM

Date: 6/5/79
Amended Form Date: ________

:ILITY AMENDMENT CLASSIFICATION - DOCKET NO(S). 50-219

Licensee: Jersey Central Power + Light Co.

Plant Name and Unit(s): / Oyster Creek

License No(s): DPR-16 Mail Control No: 7905290178

Request Dated: 5-23-79 Fee Remitted: Yes ~~x~~ No ✓

Assigned TAC No: 11654

Licensee's Fee Classification: Class I ___, II ___, III ___, IV ___, V ___, VI ___,
None ✓.

Amendment No. [redacted] 39 Date of Issuance 7/23/79

[X] 1. This request has been reviewed by DOR/DPM in accordance with Section 170.22 of Part 170 and is properly categorized. (The ~~basis~~ evaluations for this change request are contained in the Restart Eval. and the Evaluation for Amend No. 36,* dated 5/30/79)

[ ] 2. This request is incorrectly classified and should be properly categorized as Class ____. Justification for classification or reclassification: ______

[X] 3. Additional information is required to properly categorize the request.
Our original fee position remains in effect!

Snowicki 7/23/79 DLZ Ziemann 7/23/79 DATE

[ ] 4. This request is a Class ____ type of action and is exempt from fees because it:
(a) ____ was filed by a nonprofit educational institution,
(b) ____ was filed by a Government agency and is not for a power reactor,
(c) ____ is for a Class ____ (can only be a I, II, or III) amendment which results from a written Commission request dated ________ for the application and the amendment is to simplify or clarify license or technical specifications, has only minor safety significance, and is being issued for the convenience of the Commission, or
(d) ____ other (state reason therefor): ____________

HS 6/4/79
Snowicki

Dennis L. Ziemann — also see the attached note
Division of Operating Reactors/Project Management

[ ] The above request has been reviewed and is exempt from fees.

incoming attached

William O. Miller, Chief
License Fee Management Branch

Date

* in response to 5-19-79 appl.

LFMB 6/78

Jersey Central Power & Light Company is a Member of the General Public Utilities System

cc
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Shaw, Pittman, Potts and Trowbridge
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Licensing Manager
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Director, Technical Assessment
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U. S. Environmental Protection
Agency
Region II Office
ATTN: EIS COORDINATOR
26 Federal Plaza
New York, New York 10007

**(w/copy of incoming dated 5/23/79 and 5/19/79)

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 39
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Jersey Central Power & Light Company (the licensee) dated May 23, 1979, as supported by letter dated May 19, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Provisional Operating License No. DPR-16 is hereby amended to read as follows:

   B. Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 39 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 23, 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 39

PROVISIONAL OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain vertical lines indicating the areas of change.

REMOVE	INSERT
2.3-3	2.3-3
3.1-8	3.1-8
3.10-1	3.10-1
3.10-2	3.10-2
3.10-9	3.10-9
	(Figure 3.10-1)
3.10-10	--
(Figure 3.10-2)	

FUNCTION	LIMITING SAFETY SYSTEM SETTINGS
7) Low Pressure Main Steam Line, MSIV Closure	≥ 825 psig
8) Main Steam Line Isolation Valve Closure, Scram	≤ 10% Valve Closure from full open
9) Reactor Low Water Level, Scram	≥ 11',5" above the top of the active fuel as indicated under normal operating conditior
10) Reactor Low-Low Water Level, Main Steam Line Isolation Valve Closure.	≥ 7',2" above the top of the active fuel as indicated under normal operating conditions.
11) Reactor Low-Low Water Level, Core Spray Initiation	≥ 7'2" above the top of the active fuel.
12) Reactor Low-Low Water Level, Isolation Condenser Initiation	≥ 7'2" above the top of the active fuel with time delay ≤ 3 seconds.
13) Turbine Trip Scram	10 percent turbine stop valve(s closure from full open.
14) Generator Load Rejection Scram	Initiate upon loss of oil pressure from turbine acceleration relay.

BASES: Safety limits have been established in Specifications 2.1 and 2.2 to protect the integrity of the fuel cladding and reactor coolant system barriers. Automatic protective devices have been provided in the plant design to take corrective action to prevent the safety limits from being exceeded in normal operation or operational transients caused by reasonable expected single operator error or equipment malfunction. This Specification establishes the trip settings for these automatic protection devices.

The Average Power Range Monitor, APRM(1), trip setting has been established to assure never reaching the fuel cladding integrity safety limit. The APRM system responds to changes in neutron flux. However, near rated thermal power the APRM is calibrated, using a plant heat balance, so that the neutron flux that is sensed is read out as percent of rated thermal power. For slow maneuvers, those where core thermal power, surface heat flux, and the power transferred to the water follow the neutron flux, the APRM will read reactor thermal power. For fast transients, the neutron flux will lead the power transferred from the cladding to the water due to the effect of the fuel time constant. Therefore, when the neutron flux increases to the scram setting, the percent increase in heat flux and power transferred to the water will be less than the percent increase in neutron flux.

The APRM trip setting will be varied automatically with recirculation flow with the trip setting at rated flow 61.0 x 10^5 lb/hr or greater being 115.7% of rated neutron flux. Based on a complete

TABLE 3.1.1 PROTECTIVE INSTRUMENTATION REQUIREMENTS (CONTD)

Function	Trip Setting	Reactor Modes in Which Function Must Be Operable				Min. No. of Operable or Operating (Tripped) Trip Systems	Min. No. of Operable Instrument Channels Per Operable Trip Systems	Action Required*
		Shutdown	Refuel	Startup	Run			
Reactor Isolation								Close main steam isolation valves and close isolation condenser vent valves, or place in cold shutdown condition
1. Low-Low Reactor Water Level	**	X	X	X	X	2	2	
2. High Flow in Main Steam-line A	≤ 120% rated	X	X	X	X	2	2	
3. High Flow in Main Steam-line B	≤ 120% rated	X	X	X	X	2	2	
4. High Temperature in Main Steamline Tunnel	≤ Ambient at Power + 50°F	X	X	X	X	2	2	
5. Low Pressure in Main Steam-line	**				X	2	2	
6. High Radiation in Main Steam Tunnel	≤ 10X Normal Background	X	X	X	X	2	2	
C. **Isolation Condenser**								
1. High Reactor Pressure	**	X	X	X	X	2	2	Place plant in cold shutdown condition
2. Low-Low Reactor Water	**	X	X	X	X	2	2	

3.10 CORE LIMITS

Applicability: Applies to core conditions required to meet the Final Acceptance Criteria for Emergency Core Cooling Performance.

Objective: To assure conformance to the peak clad temperature limitations during a postulated loss-of-coolant accident as specified in 10 CFR 50.46 (January 4, 1974) and to assure conformance to the 17.2 KW/ft. (for 7 x 7 fuel) and 14.5 KW/ft. (for 8 x 8 fuel) operating limits for local linear heat generation rate.

Specification: A. Average Planar LHGR

During power operation, the average linear heat generation rate (LHGR) of all the rods in any fuel assembly, as a function of average planar exposure, at any axial location shall not exceed the maximum average planar LHGR (MAPLHGR) limit shown in Figure 3.10-1. If at any time during power operation it is determined by normal surveillance that the limiting value for APLHGR is being exceeded, action shall be initiated to restore operation to within the prescribed limits. If the APLHGR is not returned to within the prescribed limits within two (2) hours, action shall be initiated to bring the reactor to the cold shutdown condition within 36 hours. During this period surveillance and corresponding action shall continue until reactor operation is within the prescribed limits at which time power operation may be continued.

B. Local LHGR

During power operation, the linear heat generation rate (LHGR) of any rod in any fuel assembly, at any axial location shall not exceed the maximum allowable LHGR as calculated by the following equation:

$$LHGR \leq LHGR_d \left[1 - \frac{(\Delta P)}{P} \max \left(\frac{L}{LT} \right) \right]$$

Where: $LHGR_d$ = Limiting LHGR

$\frac{\Delta P}{P}$ = Maximum Power Spiking Penalty

LT = Total Core Length - 144 inches

L = Axial position above bottom of core

and

Fuel Type	LHGR$_d$	P/P
II	17.2	.032
IIIE	17.2	.046
IIIF	17.2	.033
V	14.5	.033
VB	14.5	.039

If at any time during operation it is determined by normal surveillance that the limiting value for LHGR is being exceeded, action shall be initiated to restore operation to within the prescribed limits. If the LHGR is not returned to within the prescribed limits within two (2) hours, action shall be initiated to bring the reactor to the cold shutdown condition within 36 hours. During this period, surveillance and corresponding action shall continue until reactor operation is within the prescribed limits at which time power operation may be continued.

C. Assembly Averaged Power Void Relationship

During power operation, the assembly average void fraction and assembly power shall be such that the following relationship is satisified:

$$\left(\frac{1-VF}{PR \times FCP}\right) \geq B$$

Where: VF = Bundle average boid fraction
PR = Assembly radial power factor
FCP = Fractional core power (relative to 1930 MWt)
B = Power-Void limit

The limiting values of "B" for each fuel type are shown in the table below:

Fuel Type(s)	B
I, II, III	.365
IIIE, IIIF	.377
V, VB	.332

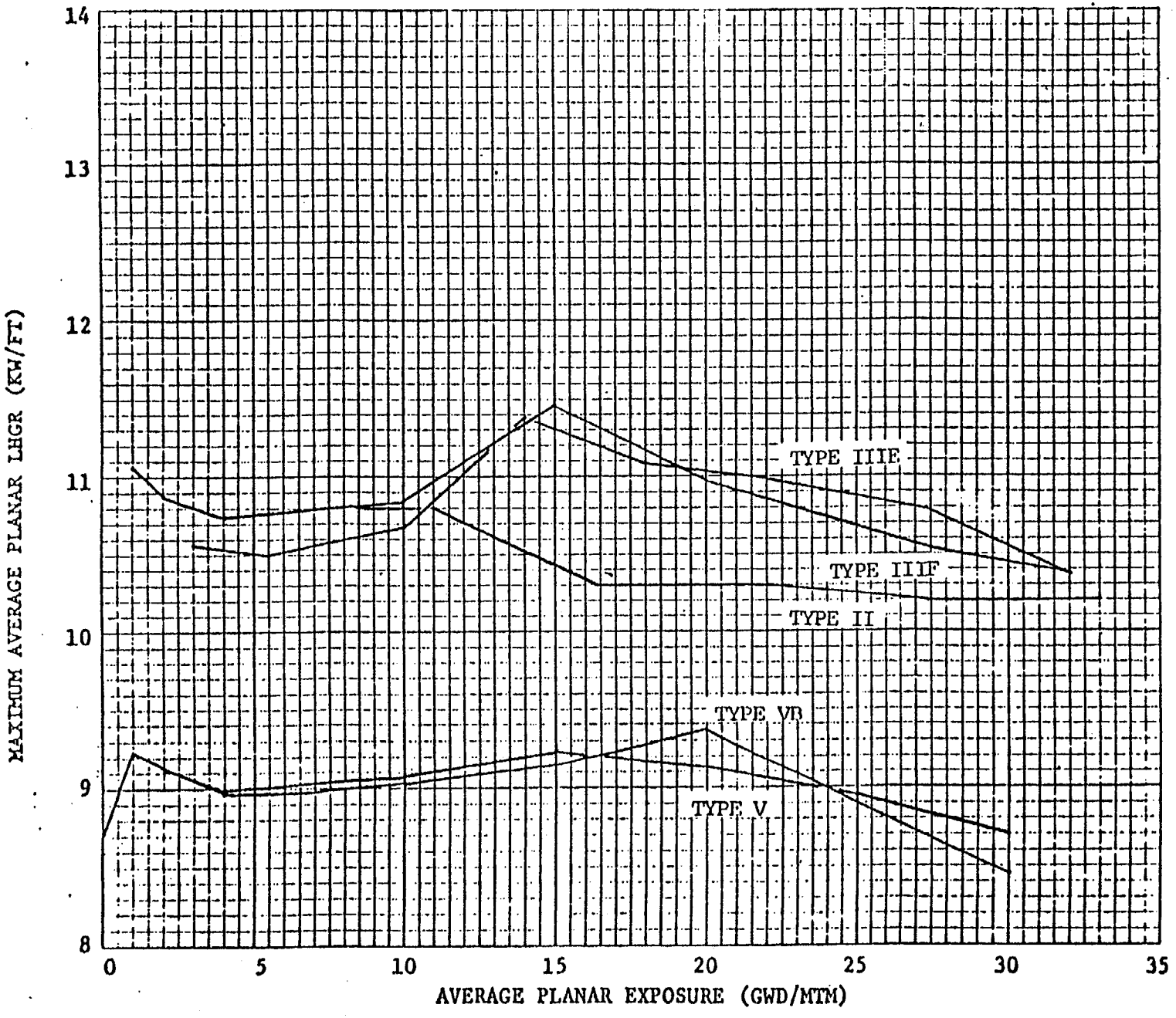
D. Minimum Critical Power Ratio (MCPR)

During steady state power operation, MCPR shall be greater than or equal to the following:

ARPM Status	MCPR Limit
1. If any two (2) LPRM assemblies which are input to the APRM system and are separated in distance by less than	1.64

FIGURE 3.10-1

MAXIMUM ALLOWABLE AVERAGE PLANAR
LINEAR HEAT GENERATION RATE



MAPLHGR LIMITS
FOR 4 RECIRCULATION
PUMP OPERATION IN OYSTER CREEK

Amendment No. 16, 24, 30, 33, 35, 39

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 39 TO LICENSE NO. DPR-16

JERSEY CENTRAL POWER & LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

Introduction

By letter dated May 23, 1979 (Addendum 1 to Change Request No. 72 dated May 19, 1979), Jersey Central Power & Light Company (the licensee) requested an amendment to the Technical Specifications of License No. DPR-16 for the Oyster Creek Nuclear Generating Station. The proposed changes would revise the Appendix A Technical Specifications to add a limiting safety system setting for the automatic initiation of the Isolation Condenser on a reactor low-low water level to Section 2.3. Table 3.1.1 would also be revised to reflect this change.

Discussion

On May 2, 1979, a loss of feedwater transient occurred at the Oyster Creek Nuclear Generating Station which resulted in a low-low-low (triple low) water level alarm. The NRC evaluated the event and corrective actions proposed to prevent recurrence (Reference 1). Three changes to the Technical Specifications were proposed as a result of the event: 1) the triple low water level was defined as a Safety Limit for all modes of reactor operation, 2) a new Safety Limit was defined which requires two recirculation loops to have the pump suction and discharge valves open at all times, and 3) a limiting safety system setting for automatic initiation of the Isolation Condenser on a reactor low-low water level signal was proposed.

License Amendment No. 36, dated May 30, 1979 (Reference 2), approved changes 1) and 2) above. This proposed amendment would incorporate the remaining proposed change, change 3) above, into the Technical Specifications.

Prior to the May 2, 1979 loss of feedwater transient, the NRC was reviewing the licensee's operation of Oyster Creek, using previously approved restrictions of Maximum Average Planar Linear Heat Generation Rate (MAPLHGR) limits (originally authorized by Amendment No. 30, dated March 14, 1978). Since Oyster Creek was operating with only 4 of the five recirculation loops and with MAPLHGR limits

approved by Amendment No. 33, dated November 11, 1978, which did not consider 4-loop operation, we concluded that a Technical Specification change to Section 3.10 was necessary. The licensee agreed. As a result license Amendment No. 35 was issued. However, Amendment No. 35 referenced incorrect MAPLHGR associated curves even though the proper limits were incorporated into the Technical Specifications. This license amendment would correct this administrative error, but the conclusions and results of Amendment No. 35 and its related Safety Evaluation Report (Reference 3) are not affected.

Evaluation

Limiting Safety System Settings

To assure that the triple-low water level safety limit will not be violated during any transient, the licensee has analyzed the transient which results in the largest loss of coolant system inventory. The licensee states (Reference 4) that a loss of feedwater (LOFW) starting from full power, results in the most severe reduction in reactor vessel water level.

We have reviewed the licensee's analysis including: 1) codes and methods, 2) coolant inventory loss assumptions, 3) coolant inventory distribution assumptions, and 4) the results.

In our Safety Evaluation Report dated May 30, 1979 (Reference 1), we concluded that: 1) the licensee's assumptions provide an adequately conservative basis upon which to calculate the minimum core water level attained during the limiting loss of coolant inventory transient, 2) with only one recirculation loop assured unisolated, recirculation flow is sufficient to prevent boiloff from reducing core water level below 6'7" above the top of the active fuel, and 3) the triple-low water level fuel cladding integrity safety limit would not be violated; therefore, the results of the limiting loss of coolant inventory transient are acceptable. However, because of the importance of automatic actuation of the Isolation Condenser at low-low level, both the low-low level and the maximum time delay before Isolation Condenser valve opening should be included as limiting safety system settings.

To assure the proper initiation and operation of the Isolation Condenser on low-low water level in the annulus in accordance with the bounding analysis assumptions, the licensee has proposed to add a limiting safety system requirement to Section 2.3 of the Oyster Creek plant Technical Specifications. The Specification states that the limiting safety system setting is the low-low water level setpoint which was assumed in the bounding analysis, i.e., 7'2" above the top of the active fuel. The limiting safety system setting incorporates a maximum three second time delay (Reference 4) to assure that the system will not fail to initiate because the core low-low water level momentarily clears as a result of the water level swell in the annulus caused by a simultaneous recirculation pump trip. Additionally, based on our review of actual plant operating data of isolation condenser initiations and possible isolation, a time delay of three seconds or less will not cause the isolation condensers to reisolate on high flow conditions caused by recirculation pump coastdown effects. This time delay is also adequate for recirculation flow coastdown effects

applicable to four loop operation. The limiting conditions of operation and surveillance requirements for the isolation condenser will not be changed. Based on the above, we find the proposed Technical Specification change acceptable.

APLHGR Limits

We have reviewed operation of Oyster Creek with 4 recirculation loops using the MAPLHGR limits approved with License Amendment No. 16 and based on combined General Electric and Exxon analyses. Although these limits were previously approved for 4 loop operation, we have reevaluated the limits using current criteria. Based on this reassessment we have concluded that: 1) the ECCS evaluation for operation with one loop out of service approved in License Amendment No. 16 is fully applicable to the present core configuration, 2) with the present restriction of Technical Specification 3.3.F.2, idle loop startup and the cold water reactivity addition accident is not a concern, 3) the manner of accounting for backflow through the inactive loop is acceptable, 4) the locked rotor event while operating with four recirculation pumps is bounded by the previously reviewed transient analysis and remains acceptable, and 5) the newly proposed MAPLHGR limits are based on calculational methods previously accepted, they do not change MAPLHGR limits previously found acceptable, but, only extend the previous calculations to higher fuel exposures and are therefore acceptable.

Based on the above conclusions, the staff issued License Amendment No. 35 (Reference 3). However, the Technical Specification changes incorporated with License Amendment No. 35 were incomplete. Although the proper limits had been incorporated some of the text material still referred to the limits established by License Amendment No. 33, dated November 11, 1978. To correct these inconsistencies, the following changes should be made. The licensee agrees with these changes: 1) reinstating the Assembly Averaged Power Void Relationship for all fuel types on page 3.10-2 which was removed by Amendment No. 33, 2) delete the word Interim from the title of Figure 3.10-1. which was on the copy submitted by the licensee and inadvertently not removed, 3) remove Figure 3.10-2 and delete the reference to Figure 3.10-2 from page 3.10-1 since the axial multiplier was derived for the less restrictive MAPLHGR limits approved by License Amendment No. 33.

These three changes correct the inconsistencies between the use of the limits approved by License Amendment No. 35 and License Amendment No. 33. Since the changes are administrative and establish the more restrictive limits approved by License Amendment No. 16 and 35 they are acceptable.

Summary

The changes to the Technical Specifications to include the automatic initiation of the Isolation Condenser as a limiting safety system setting was evaluated and found acceptable in our Safety Evaluation Report dated May 30, 1979 (Reference 1), the change in the Technical Specifications to use the more restrictive MAPLHGR limits for 4-loop operation were evaluated and found

acceptable in the Safety Evaluation Report issued with License Amendment No. 35 dated May 30, 1979. This amendment would incorporate the Limiting Safety System Settings for automatic initiation of the Isolation Condenser and correct administrative errors in License Amendment No. 35. For the reasons stated in this evaluation we find the proposed Technical Specification changes acceptable.

Environmental Considerations

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because this amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: July 23, 1979

REFERENCES

1. Safety Evaluation Report dated May 30, 1979, Review of the May 2, 1979 Transient at the Oyster Creek Nuclear Generating Station.

2. License Amendment No. 36 dated May 30, 1979, Modification of Section 2.1.D to extend the applicability of the minimum water level Safety Limit to all Modes of Operation and Addition of a new Safety Limit in Section 2.1.F to Require that Two Recirculation Loops Remain Open During All Modes of Operation.

3. License Amendment No. 35 dated May 30, 1979, for the Oyster Creek Nuclear Generating Station.

4. Letter dated May 19, 1979 from I. R. Finfrock (JCP&L) to NRC enclosing an analysis titled "Bounding Loss of Coolant Inventory Transient for the Oyster Creek Plant."

7590-01

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-219

JERSEY CENTRAL POWER & LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO PROVISIONAL OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 39 to Provisional Operating License No. DPR-16, issued to Jersey Central Power & Light Company (the licensee), which revised the Technical Specifications for operation of the Oyster Creek Nuclear Generating Station (the facility) located in Ocean County, New Jersey. The amendment is effective as of its date of issuance.

The amendment: 1) adds a limiting safety system setting for the automatic initiation of the Isolation Condenser on a reactor low-low water level signal (Section 2.3), 2) revises Table 3.1.1 to reflect the change in 1) above, 3) adds the Assembly Average Power Void Relationship for all fuel types in the reactor (Section 3.10.c), 4) removes Figure 3.10-2 and the reference to Figure 3.10-2 from Section 3.10.A, and 5) removes the word interim from Figure 3.10-9, which was inadvertantly included in the title of this Figure during the issuance of License Amendment No. 35, dated May 30, 1979.

The application for amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated May 23, 1979, and the licensee's submittal dated May 19, 1979, (2) Amendment No. 39 to License No. DPR-16, (3) the Commission's related Safety Evaluation, (4) Amendment Nos. 35 and 36 dated May 30, 1979 and the related Safety Evaluations, and (5) the Commission's letter and Safety Evaluation Report dated May 30, 1979. All of these are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Ocean County Library, Brick Township Branch, 401 Chambers Bridge Road, Brick Town, New Jersey 08723. A copy of items (2), (3), (4), and (5) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 23rd day of July, 1979.

FOR THE NUCLEAR REGULATORY COMMISSION

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors