JAN 2 1 1976

DISTRIBUTION:
Docket
NRC PDR
Local PDR
ORB Rdg
KRGoller
TJCarter
CParrish
WPaulson
OELD
OI&E (3)
BJones (4)
BScharf (15)
JMcGough
JSaltzman
GWilliams
SVarga
CHebron
AESteen
ACRS (16)
CMiles
TBAbernathy
JRBuchanan
Gray File
Xtra Copies

Docket No. 50-219

Jersey Central Power & Light Company
ATTN: Mr. I. R. Finfrock, Jr.
Vice President - Generation
Madison Avenue at Punch Bowl Road
Morristown, New Jersey 07960

Gentlemen:

The Commission has issued the enclosed Amendment No. 12 to Facility Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station. The amendment consists of changes to the Technical Specifications in response to your application dated December 3, 1975.

These changes to the Technical Specifications permit reduced core spray system capability for the duration of the plant outage during which improvements to the operation of the emergency core cooling system are being made when the reactor is in the cold shutdown condition, in the refuel mode with the reactor system at a temperature less than 212°F, or in the startup mode for the purpose of low power physics testing.

We have reviewed your proposed modifications to the emergency core cooling systems described in your letter of June 24, 1975 and revised by letter dated July 15, 1975 and further supplemented by letters dated November 7, 1975 and January 16, 1976. We conclude that the modifications are acceptable and we hereby authorize you to make the modifications. The bases for our acceptance of the modifications are discussed in the enclosed Safety Evaluation.

Your reevaluation of the Oyster Creek emergency core cooling system performance was submitted by letter dated December 23, 1975 and included consideration of (1) a revised single failure analysis and (2) proposed ECCS modifications. We are reviewing this reevaluation. Therefore, you are not authorized to operate the Oyster Creek reactor without our prior approval of this reevaluation.

SEE PREVIOUS YELLOW FOR CONCURRENCE

OFFICE	ORB#3	ORB#3	OELD	ORB#3	AD:OR/DOR
SURNAME	CParrish	WPaulson:acr		GLear	KRGoller
DATE	1/ /76	1/ /76	1/ /76	1/ /76	1/ /76

Form AEC-318 (Rev. 9-53) AECM 0240 ☆ U. S. GOVERNMENT PRINTING OFFICE: 1974-526-166

DISTRIBUTION:

Docket	SVarga
NRC PDR	CHebron
Local PDR	AESteen
ORB Rdg	ACRS (16)
KRGoller	CMiles
TJCarter	TBAbernathy
CParrish	JRBuchanan
WPaulson	Gray File
OELD	Xtra Copies
OI&E(3)	
BJones (4)	
BScharf (15)	
JMcGough	
JSaltzman	
GWilliams	

Docket No. 50-219

Jersey Central Power & Light Company
ATTN: Mr. I. R. Finfrock, Jr.
Vice President - Generation
Madison Avenue at Punch Bowl Road

Gentlemen:

The Commission has issued the enclosed Amendment No. 12 to Facility Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station. The amendment consists of changes to the Technical Specifications in response to your application dated December 3, 1975.

This amendment will revise the Technical Specifications to allow the facility to remain in the shutdown refueling, or startup mode with reduced core spray system capability for the duration of the outage during which improvements to the ECCS system are made.

We have reviewed your proposed modifications to the emergency core cooling systems described in your letter of June 24, 1975 and revised by letter dated July 15, 1975 and further supplemented by letters dated November 7, 1975 and January 16, 1976. Based on our review, we find that the proposed modifications are acceptable. The bases for our acceptance are discussed in the enclosed Safety Evaluation. Our review of your reevaluation of the Oyster Creek emergency core cooling system (ECCS) performance submitted by letter dated December 23, 1975 and based on a revised single failure analysis and proposed ECCS modifications, will be the subject of a separate Safety Evaluation.

Shouldn't Stello's people be on Concurrence?

OFFICE ➤	ORB#3	ORB#3	OELD	ORB#3	AD:OR/DOR	
SURNAME ➤	CParrish	WPaulson:acr		GLear	KRGoller	
DATE ➤	1/ 20 /76	1/ 20 /76	1/ 20 /76	1/ /76	1/ /76	

Form AEC-318 (Rev. 9-53) AECM 0240 ☆ U. S. GOVERNMENT PRINTING OFFICE: 1974-526-166

Copies of the Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosure:
1. Amendment No. 12
2. Safety Evaluation
3. Federal Register Notice

cc: See Next Page

DISTRIBUTION:
Docket
NRC PDR
Local PDR
ORB#3 Rdg
KRGoller
TJCarter
CParrish
WPaulson
OELD
OI&E (3)
BJones (4)
BScharf (15)
JMcGough
JSaltzman
GWilliams
SVarga
CHebron
AESteen
ACRS (16)
CMiles
Gray File
Xtra Copies
TBAbernathy
JRBuchanan

OFFICE	ORB#3	ORB#3	OELD	ORB#3	RL:AD/ORS	
SURNAME	CParrish	WPaulson:acr		GLear	KRGoller	
DATE	1/ /76	1/ /76		1/ /76	1/ /76	

cc:

G. F. Trowbridge, Esquire
Shaw, Pittman, Potts and Trowbridge
Barr Building
910 17th Street, N. W.
Washington, D. C. 20006

Jersey Central Power & Light Company
ATTN: Mr. Thomas M. Crimmins, Jr.
Safety and Licensing Manager
GPU Service Corporation
260 Cherry Hill Road
Parsippany, New Jersey 07054

Anthony Z. Roisman, Esquire
Roisman, Kessler and Cashdan
1712 N Street, N. W.
Washington, D. C. 20036

Paul Rosenberg, Esquire
Daniel Rappoport, Esquire
2323 S. Broad Street
Trenton, New Jersey 08610

Honorable Joseph W. Ferraro, Jr.
Deputy Attorney General
State of New Jersey
101 Commerce Street - Room 208
Newark, New Jersey 07102

George F. Kugler, Jr.
Attorney General
State of New Jersey
State House Annex
Trenton, New Jersey 08625

Ocean County Library
15 Hooper Avenue
Toms River, New Jersey 08753

The Honorable W. M. Mason
Mayor, Lacey Township
P. O. Box 475
Forked River, New Jersey 08731

Honorable Wm. F. Hyland
Attorney General
State of New Jersey
State House Annex
Trenton, New Jersey 08601

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 12
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Jersey Central Power & Light Company (the licensee) dated December 3, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

   E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.

2. Accordingly, the licensee is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Karl R. Goller

Karl R. Goller, Assistant Director
for Operating Reactors
Division of Operating Reactors

Attachment:
Changes to the
Technical Specifications

Date of Issuance: JAN 21 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 12

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Replace page 3.4-1, page 3.4-4 and 3.4-5 with the attached revised pages. Add page 3.4-1a.

3.4 EMERGENCY COOLING

Applicability: Applies to the operating status of the emergency cooling systems.

Objective : To assure operability of the emergency cooling systems.

Specification: A. Core Spray System

1. The core spray system shall be operable at all times with irradiated fuel in the reactor vessel, except as specified in Specifications 3.4.A.3 and 3.4.A.4.

2. The absorption chamber water volume shall be at least 82,000 ft^3 in order for the core spray system to be considered operable.

3. If one core spray system loop or its core spray header ΔP instrumentation becomes inoperable during the run mode, the reactor may remain in operation for a period not to exceed 7 days provided the remaining loop has no inoperable components and is demonstrated daily to be operable.

4. If one of the redundant active loop components in the core spray system becomes inoperable during the run mode, the reactor may remain in operation for a period not to exceed 15 days provided the other similar component in the loop is demonstrated daily to be operable. If two of the redundant active loop components become inoperable, the limits of Specification 3.4.A. shall apply.

5. During the period when one diesel is inoperable, the core spray loop connected to the operable diesel shall have no inoperable components.

6. If Specifications 3.4.A.3, 3.4.A.4, and 3.4.A.5 are not met, the reactor shall be placed in the cold shutdown condition. If the core spray system becomes inoperable, the reactor shall be placed in the cold shutdown condition and no work shall be performed on the reactor or its connected systems which could result in lowering the reactor water level to less than 4'8" above the top of the active fuel.

7. If necessary to accomplish maintenance or modifications to the core spray systems, their power supplies or water supplies, reduced system availability is permitted when the reactor is: (a) maintained in the cold shutdown condition, or (b) in the refuel mode with the reactor coolant system maintained at less than 212°F and vented, and (c) no work is performed on the reactor vessel and connected systems that could result in lowering the reactor water level to less than 4' 8" above the top of the active fuel. Reduced

Core Spray System availability is minimally defined as follows:

a. At least one core spray pump, and system components necessary to deliver rated core spray to the reactor vessel, must remain operable to the extent that the pump and any necessary valves can be started or operated from the control room or from local control stations,

b. The fire protection system is operable, and

c. These systems are demonstrated to be operable on a weekly basis.

8. If necessary to accomplish maintenance or modifications to the core spray systems, their power supplies or water supplies, reduced system availability is permitted when the reactor is in the refuel mode with the reactor coolant system maintained at less than 212°F or in the startup mode for purposes of low power physics testing. Reduced core spray system availability is defined as follows:

   a. At least one core spray pump in each loop, and system components necessary to deliver rated core spray to the reactor vessel, must remain operable to the extent that the pump and any necessary valves in each loop can be started or operated from the control room or from local control stations,

   b. The fire protection system is operable, and

   c. Each core spray pump and all components in 3.4.A.8a are demonstrated to be operable every 72 hours.

9. If Specifications 3.4.A.7 and 3.4.A.8 cannot be met, the requirements of Specification 3.4.A.6 will be met and work will be initiated to meet minimum operability requirements of 3.4.A.7 or 3.4.A.8.

B. Automatic Depressurization System

1. Five electromatic relief valves of the automatic depressurization system shall be operable when the reactor is pressurized above 110 psig, except as specified in 3.4.B.2.

within that time. The 7 days is based upon the consideration discussed in the bases of Specification 3.2 and the pump operability tests of Specification 4.4. If repairs cannot be made, the reactor is depressurized and vented to prevent pressure buildup and no work is allowed to be performed on the reactor which could result in lowering the water level below the safety limit of 4'8".

Each core spray loop contains redundant active components. Therefore, with the loss of one of these components the system is still capable of supplying rated flow and the system as a whole (both loops) can tolerate an additional single failure of one of its active components and still perform the intended function and prevent clad melt. Therefore, if a redundant active component fails, a longer repair period is justified based on the consideration given in the bases of Specification 3.2. The consideration indicates that for a one out of 4 requirement the time out of service would be

$$\frac{\tau}{1.71} = \frac{30 \text{ days}}{1.71} = 17.5 \text{ days.}$$

Specification 3.4.A.5 ensures that if one diesel is out of service for repair, the core spray system loop on the other diesel must be operable with no components out of service. This ensures that the loop can perform its intended function, even assuming one of its active components fails. If this condition is not met, the reactor is placed in a condition where core spray is no longer required.

When the reactor is in the shutdown or refueling mode and the reactor coolant system is less than 212°F and vented and no work is being performed that could result in lowering the water level to less than 4' 8" above the core, the likelihood of a leak or rupture leading to uncovering of the core is very low. The only source of energy that must be removed is decay heat and one day after shutdown this heat generation rate is conservatively calculated to be not more than 0.6% of rated power. Sufficient core spray flow to cool the core can be supplied by one core spray pump or one of the two fire protection system pumps under these conditions. When it is necessary to perform repairs on the core spray system components, power supplies or water sources, Specification 3.4.A.7 permits reduced cooling system capability to that which could provide sufficient core spray flow from two independent sources. Manual initiation of these systems is adequate since it can be easily accomplished within 15 minutes during which time the temperature rise in the reactor will not reach 2200°F.

In order to allow for certain primary system maintenance, which will include control rod drive repair, LPRM removal/installation, reactor leak test, etc., (all performed according to approved procedures), Specification 3.4.A.8 requires the availability of an additional core spray pump in an independent loop, while this maintenance is being performed the likelihood of the core being uncovered is still considered to be very low, however, the requirement of a second core

spray pump capable of full rated flow and the 72 hour operability demonstration of both core spray pumps is specified.

The relief valves of the automatic depressurization system enable the core spray system to provide protection against the small break in the event the feedwater system is not active. Three of the five relief valves are sufficient for accident protection and since the relief valves are not readily testable, the short time period of only 8 hours has been specified for repairs.

The containment spray system is provided to remove heat energy from the containment in the event of a loss-of-coolant accident. The flow from one pump in either loop is more than ample to provide the required heat removal capability[2]. The emergency service water system provides cooling to the containment spray heat exchangers and, therefore, is required to provide the ultimate heat sink for the energy release in the event of a loss-of-coolant accident. The emergency service water pumping requirements are those which correspond to containment cooling heat exchanger performance implicit in the containment cooling description. Since the loss-of-coolant accident while in the cold shutdown condition would not require containment spray, the system may be deactivated to permit integrated leak rate testing of the primary containment while the reactor is in the cold shutdown condition.

The control rod drive hydraulic system can provide high pressure coolant injection capability. For break sizes up to 0.002 ft^2, a single control rod drive pump with flow of 110 gpm is adequate for maintaining the water level nearly five feet above the core, thus alleviating the necessity for auto-relief actuation [3].

The core spray main pump compartments and containment spray pump compartments were provided with water-tight doors. [4] Specification 3.4.E ensures that the doors are in place to perform their intended function.

"Similarly, since a loss-of-coolant accident when primary containme integrity is not being maintained would not result in pressure buil up in the drywell or torus, the system may be made inoperable under these conditions. This prevents possible personnel injury associat with contact with chromated torus water."

References

(1) Licensing Application, Amendment
(2) Licensing Application, Amendment 32, Question 3
(3) Licensing Application, Amendment 18, Question 1
(4) Licensing Application, Amendment 18, Question 4

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 12 TO

PROVISIONAL OPERATING LICENSE NO. DPR-16

JERSEY CENTRAL POWER AND LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

Introduction

By letter dated June 24, 1975 and revised by letter dated July 15, 1975, and further supplemented by letters dated November 7, 1975 and January 16, 1976, Jersey Central Power and Light Company (JCP&L) (1) presented the results of a reassessment of the effects of single passive electrical failures on the performance of the emergency core cooling system (ECCS) of the Oyster Creek Nuclear Generating Station, and (2) proposed modifications to the Oyster Creek plant that would enable the facility to automatically accommodate a single passive electrical failure, including a single passive failure of an emergency diesel generator bus, without adversely affecting the ability of the ECCS to perform its function. The proposed modifications are based on analyses performed in accordance with the conditions added to Provisional Operating License No. DPR-16 by Amendment No. 8 dated May 24, 1975.

In order to accomplish the proposed modifications, portions of the core spray system will have to be temporarily removed from service. By letter dated December 3, 1975, JCP&L has submitted a request for license amendment that would permit reduced core spray system capability for the duration of the plant outage during which improvements to the operation of the ECCS are being made when the reactor is in the cold shutdown condition, in the refuel mode with the reactor system at a temperature less than 212°F, or in the startup mode for the purpose of low power physics testing.

Our review of JCP&L's submittal dated December 23, 1975 regarding the reevaluation of ECCS performance in accordance with Section 50.46 of 10 CFR Part 50 will be the subject of a separate safety evaluation.

Evaluation

1. Proposed ECCS Modifications

   The proposed modification to the core spray system will permit either emergency diesel generator to power one core spray pump and one

booster pump combination in each of the two core spray loops. The proposed redistribution of power sources for the core spray pumps and associated valves will permit the minimum required number of pumps and valves to function in the event of any break, including a core spray line break, and the simultaneous loss of one division of electrical equipment. We find that this modification enhances the operability of the core spray system and, therefore, it is acceptable.

The modified diesel generator sequencing times presented in JCP&L's letter of November 7, 1975, will assure that source voltages will be provided from the diesel generators that will be equal to or greater than the voltages which are presently available to the engineered safety feature (ESF) motors. The modified sequence times provide for reduced thermal stress in the ESF equipment and they are consistent with the equipment availability times assumed in the Oyster Creek accident analyses; therefore, we find that the modified sequence times are acceptable.

The licensee will test the modification after it has been installed. The tests will include automatic start of each core spray system with the simulated loss of a diesel generator. All emergency loads in the other (non-faulted) division will be loaded in their normal sequence from the operating diesel generator. This test will verify the assumptions and calculations used in the modification design. Thus, the assumptions and calculations used in the engineering design of the proposed modification will be demonstrated to have no adverse affect on the performance of the emergency core cooling system.

JCP&L proposes to install new wiring in accordance with separation criteria which more closely follow present standards. Although the original design criteria provide an acceptable degree of safety, we find that the new criteria further enhance public health and safety and, therefore, are acceptable.

During the course of our review, we requested the licensee to provide surveillance, inspection, and testing requirements for the time delay relays which control the loading of the control rod drive pumps onto the diesel generator in order to assure that failure of these relays would not be undetected and cause over loading and possible failure of the diesel generators to deliver the required voltage and power. By letter dated January 16, 1976, the licensee agreed to propose Technical Specifications for surveillance, inspection, and testing of these relays. We find this acceptable.

JCP&L also proposed to modify the Automatic Depressurization System (ADS) logic to eliminate the possibility that a single failure of vital DC panel D or E could prevent the ADS from initiating reactor depressurization. Based on our review of the revised logic, we find that the proposed modification is acceptable.

In addition, the licensee has proposed a redistribution of the power source for the emergency condenser isolation valves to eliminate the possibility that loss of power to motor control center DC-1 can prevent both isolation valves from operating. Based on our review of the proposed change, we find that the modification is acceptable.

We conclude that the proposed ECCS modifications are acceptable. By letter dated December 23, 1975 JCP&L submitted a reevaluation of the Oyster Creek ECCS performance that included consideration of (1) a revised single failure analysis, and (2) proposed ECCS modifications. Our review of this reevaluation of ECCS performance will be the subject of a separate safety evaluation.

2. Proposed Technical Specification Changes

By letter dated December 3, 1975, JCP&L has requested an amendment to Provisional Operating License No. DPR-16 that would temporarily reduce the core spray system availability during the period of time necessary to make modifications or perform maintenance on the core spray system. The current Technical Specifications require that the core spray system be operable at all times with irradiated fuel in the reactor with certain limited exceptions when in the Run mode.

JCP&L has proposed that reduced core spray system availability be permitted if necessary to accomplish maintenance or modifications to the core spray systems, their power supplies, or water supplies when the reactor is (1) in the cold shutdown condition, or (2) in the refuel mode with the reactor system maintained at a temperature less than 212°F and vented, and (3) no work is being performed on the reactor vessel and connected systems that could result in lowering the reactor water level to less than 4 feet - 8 inches above the top of the active fuel. Reduced core spray system availability for these conditions is defined as:

a. At least one core spray pump, and system components necessary to deliver rated core spray to the reactor vessel, must remain operable to the extent that the pump and any necessary valves can be started or operated from the control room or from local control stations.

b. The fire protection system is operable, and

c. These systems are demonstrated to be operable on a weekly basis.

Normal shutdown cooling is provided by the reactor shutdown cooling system. During a reactor outage when the reactor is in the shutdown or refueling mode and the reactor coolant system is vented and at a temperature less than 212°F, and no work is being performed that could cause the water level to decrease to less than 4 feet 8 inches, it is unlikely that a leak or system failure could occur that would result in uncovering of the reactor core during the period of time that maintenance or modification to the core spray system is being performed. Nevertheless, JCP&L proposes to provide the capability to initiate core spray from (1) one core spray pump and associated components pumping water from the torus or condensate storage tank, and (2) from the fire protection system pumping water from the fresh water fire pond. JCP&L has provided an analysis that demonstrates that core spray flow from either of these independent sources is adequate to provide shutdown cooling twenty-four hours after shutdown and at later times. JCP&L has stated that manual initiation of (1) the core spray pump and associated components, and (2) the fire protection system can be accomplished within 15 minutes. The analysis provided by JCP&L shows that the temperature of the fuel, assuming adiabatic heatup for fifteen minutes occurring at twenty-four hours after shutdown, is well below 2200°F. Based on our review, we find that the proposed reduced core spray system availability is acceptable for the period of time during which modifications or maintenance described above are being performed when the reactor is (1) in the cold shutdown condition, or (2) in the refuel mode with the reactor system maintenance at a temperature less the 212°F and vented, and (3) no work is being performed that could cause the reactor water level to decrease to less than 4 feet 8 inches above the top of the fuel.

JCP&L has also proposed that reduced core spray availability be permitted if necessary to accomplish maintenance or modification to the core spray system, their power supplies, or water supplies, when the reactor is (1) in the refuel mode and the reactor coolant system is maintained at a temperature less than 212°F, or (2) in the startup mode for purposes of low power physics tests. When the reactor is in the refuel mode for the purpose of performing leak tests or in the startup mode for the purpose of performing low power physics tests, one core spray pump in each of the two core spray loops will be operable in addition to the fire protection system being operable.

During leak testing, the reactor will be hydraulically pressurized to 800 psig and the reactor coolant system temperature will be less than 212°F. In the event of a system leak or break during the hydraulic leak test, the depressurization rate in the coolant system will be extremely rapid due to the extremely low compressibility of the water; hence, system pressure will reduce to a level that will permit full flow by the core spray pumps or by the fire protection system within the fifteen minutes required to manually initiate the system.

When the reactor is in the startup mode for the purpose of low power physics tests, the system will be at a temperature less than 212°F and it will be depressurized; hence, full flow could be initiated by the core spray pumps or the fire protection system at any time within the fifteen minutes required to manually initiate either system. Based on our review, we find that the proposed reduced core spray availability is acceptable for the period of time during which modifications or maintenance are performed to the core spray systems, their power supplies, or water supplies.

Environmental Considerations

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental statement, negative declaration, or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: JAN 2 1 1976

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-219

JERSEY CENTRAL POWER & LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENT

TO PROVISIONAL OPERATING LICENSE

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 12 to Facility Operating License No. DPR-16 issued to Jersey Central Power & Light Company which revised Technical Specifications for operation of the Oyster Creek Nuclear Generating Station, located in Ocean County, New Jersey. The amendment is effective as of the date of issuance.

The amendment will revise the Technical Specifications to (1) allow the facility to remain in the shutdown refueling, or startup mode with reduced core spray system capability for the duration of the outage during which improvements to the ECCS system are made. The proposed action is in accordance with your license amendment application dated June 24, 1975, and supplements dated July 15, 1975, November 7, 1975, and December 3, 1975.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Notice of Proposed Issuance of Amendment to Facility Operating

OFFICE	ORB#3	ORB#3	ORB#3			
SURNAME	CParrish CP	WPaulson:acr WP	GLear GL			
DATE	1/ 20 /76	1/ 20 /76	1/ 21 /76			

License in connection with this action was published in the FEDERAL REGISTER on December 17, 1975 (40 F.R. 58517). No request for a hearing or petition for leave to intervene was filed following notice of the proposed action.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental statement, negative declaration or environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated December 3, 1975, (2) Amendment No. 12 to License No. DPR-16, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Ocean County Library, 15 Hooper Avenue, Toms River, New Jersey 08753.

A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: director, division of Operating Reactors.

Dated at Bethesda, Maryland, this day of

FOR THE NUCLEAR REGULATORY COMMISSION

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

OFFICE ➔	ORB#3	ORB#3	ORB#3			
SURNAME ➔	CParrish	WPaulson:acr	GLear			
DATE ➔	1/ /76	1/ /76	1/ /76			

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-219

JERSEY CENTRAL POWER & LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENT

TO PROVISIONAL OPERATING LICENSE

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 12 to Facility Operating License No. DPR-16 issued to Jersey Central Power & Light Company which revised Technical Specifications for operation of the Oyster Creek Nuclear Generating Station, located in Ocean County, New Jersey. The amendment is effective as of the date of issuance.

The amendment will revise the Technical Specifications to (1) allow the facility to remain in the shutdown refueling, or startup mode with reduced core spray system capability for the duration of the outage during which improvements to the ECCS system are made, and (2) provide operability requirements for the condensate system until the ECCS modifications are completed. The proposed action is in accordance with your license amendment application dated June 24, 1975, and supplements dated July 15, 1975, November 7, 1975, and December 3, 1975.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Notice of Proposed Issuance of Amendment to Facility Operating

OFFICE ➔						
SURNAME ➔						
DATE ➔						

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-219

JERSEY CENTRAL POWER & LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENT

TO PROVISIONAL OPERATING LICENSE

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 12 to Facility Operating License No. DPR-16 issued to Jersey Central Power & Light Company which revised Technical Specifications for operation of the Oyster Creek Nuclear Generating Station, located in Ocean County, New Jersey. The amendment is effective as of the date of issuance.

The amendment will revise the Technical Specifications to allow the facility to remain in the shutdown refueling, or startup mode with reduced core spray system capability for the duration of the outage during which improvements to the ECCS system are made. The proposed action is in accordance with the license amendment application dated June 24, 1975, and supplements dated July 15, 1975, November 7, 1975, and December 3, 1975.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Notice of Proposed Issuance of Amendment to Facility Operating

License in connection with this action was published in the FEDERAL REGISTER on December 17, 1975 (40 F.R. 58517). No request for a hearing or petition for leave to intervene was filed following notice of the proposed action.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental statement, negative declaration or environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated December 3, 1975, (2) Amendment No. 12 to License No. DPR-16, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Ocean County Library, 15 Hooper Avenue, Toms River, New Jersey 08753.

A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: director, division of Operating Reactors.

Dated at Bethesda, Maryland, this 21 day of January 1976

FOR THE NUCLEAR REGULATORY COMMISSION

George Lear

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors