

1/16/76

Docket No. 50-219

Jersey Central Power & Light Company
ATTN: Mr. I. R. Finfrock, Jr.
Vice President - Generation
Madison Avenue at Punch Bowl Road
Morristown, New Jersey 07960

Gentlemen:

The Commission has issued the enclosed Amendment No. 10 to Provisional Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station. This amendment consists of a change to the Technical Specifications, Appendix A, and is in response to your request dated January 10, 1975 and modified by your letter dated May 5, 1975.

The amendment incorporates into the Oyster Creek Nuclear Generating Station Technical Specifications changes to the Administrative Controls. Changes to your proposal were necessary to meet our requirements. These have been discussed with your staff. The Technical Specifications are based on the regulatory positions described in Guides 1.8, "Personnel Selection and Training", 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4, and 1.33, "Quality Assurance Program Requirements".

We request that you use the formats presented in the Appendices to Regulatory Guide 1.16, Revision 4, for reporting operating information and that you report events of the type described under the section "Events of Potential Public Interest". Instructions for using these reporting formats are contained in Regulatory Guide 1.16 (a copy is enclosed for your use), and AEC report OOE-SS-001 titled "Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File" of which you were previously provided a copy. This report is modified by updated instructions dated December 8, 1975 which are enclosed. Copy requirements are summarized in Regulatory Guide 10.1, "Compilation of Reporting Requirements for Persons Subject to NRC Regulations", a copy of which is also enclosed. This guide will

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assist you in identifying reports that are required by the Commission's regulations set forth in Title 10 Code of Federal Regulations but are not contained in your technical specifications. Reports that are required by the regulations have not been repeated in your Technical Specifications.

Please note that we have discontinued the use of separate identifying numbers for changes to technical specifications. Sequential amendment numbers will be continued as in the past.

Copies of the related Safety Evaluation and the Federal Register Notice are also enclosed.

We note that in the enclosure to your letter dated January 10, 1975, it is stated that a comparative analysis between Section 6.5, Review and Audit, and the standard specification has not been completed; however, when it is completed, a Technical Specification change request will be submitted. We request that you inform us of the date that this change request will be submitted. In addition, it is stated in the above cited enclosure, that the Oyster Creek Station procedures are being updated in accordance with Sections 5.1 and 5.3 of ANSI N18.7-1972 and Appendix A of Regulatory Guide 1.33, but since the update has not been completed, references to the Regulatory Guide and ANSI standard have not been included in Section 6.8, Procedures. We request that you inform us of the date that you expect to complete this update and modify Section 6.8 to include the above references.

Sincerely,

George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

Enclosures:

- 1. Amendment No. 10
- 2. Regulatory Guide 1.16
- 3. Updated Instructions
- 4. Regulatory Guide 10.1
- 5. Safety Evaluation
- 6. Federal Register Notice

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See next page

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Jersey Central Power & Light Co.

- 2 -

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

JERSEY CENTRAL POWER AND LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION, UNIT 1

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 10
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Jersey Central Power and Light Company (the licensee) dated January 10, 1975, and modified by your letter dated May 5, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.
 - E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B. of Provisional Operating License No. DPR-16 is hereby amended to read as follows:

"B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensees shall operate the facility in accordance with the Technical Specifications, as revised".

3. This license amendment is effective 60 days from the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

Attachment:
Changes to the
Technical Specifications

Date of Issuance: 10/10/70

ATTACHMENT TO LICENSE AMENDMENT NO. 10
TO THE TECHNICAL SPECIFICATIONS
PROVISIONAL OPERATING LICENSE NO. DPR-16
DOCKET NO. 50-219

Replace page i, page ii, page 1.0-3 and pages 6-1 through 6-33
with the attached revised pages.

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- A. At least one door at each access opening is closed.
- B. The standby gas treatment system is operable.
- C. All reactor building ventilation system automatic isolation valves are operable or are secured in the closed position.

1.15 (DELETED)

6.1 RESPONSIBILITY

6.1.1 The Station Superintendent shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2.1.

FACILITY STAFF

6.2.2 The facility organization shall be as shown on Figure 6.2.2 and:

- a. Each on duty shift shall include at least the shift staffing indicated on Figure 6.2.2.
- b. At least one licensed operator shall be in the control room when fuel is in the reactor.
- c. Two licensed operators shall be in the control room during all reactor startups, shutdowns, and other periods involving planned control rod manipulations.
- d. ALL CORE ALTERATIONS shall be directly supervised by a licensed Senior Reactor Operator who has no other concurrent responsibilities during this operation, or a licensed Reactor Operator will be assigned to manipulate the fuel grapple.
- e. An individual qualified in radiation protection measures shall be on site when fuel is in the reactor.

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 The members of the facility staff shall meet or exceed the following qualifications:

Station Superintendent

Requirements: Ten years total power plant experience of which three years must be nuclear power plant experience. Four years of academic training may fulfill four of the remaining seven years of required experience. The Station Superintendent must be capable of obtaining or possess a Senior Reactor Operator's License.

May 1975

Amendment No. 10

Dated:

Chief Engineer

Requirements: Eight years total power plant experience of which three years must be nuclear power plant experience. Two years of academic or related technical training may fulfill two years of the remaining five years of required experience. The Chief Engineer must be capable of obtaining or possess a Senior Reactor Operator's License.

Operations Engineer

Requirements: Eight years total power plant experience of which three years must be nuclear power plant experience. Two years of academic or related technical training may fulfill two of the remaining five years of required experience. The Operations Engineer must possess a Senior Reactor Operator's License.

Maintenance Engineer

Requirements: Seven years of total power plant experience of which one year must be nuclear power plant experience. Two years of academic or related technical training may fulfill two of the remaining six years of required experience.

Technical Engineer

Requirements: Eight years of responsible positions of which one year must be nuclear power plant experience. Four years of academic training may fulfill four of the remaining seven years of required experience.

Supervisor-Radiation Protection

Requirements: Five years of related experience. Four years of these five years of experience may be fulfilled by two years of academic education and two years of related technical training.

6.4 TRAINING

- 6.4.1 A retraining program for operators shall be maintained under the direction of the senior staff assistant and shall meet the requirements and recommendation of Appendix A of 10 CFR Part 55. Replacement training programs, the content of which shall meet the requirements of 10 CFR Part 55, shall be conducted under the direction of the senior staff assistant, for licensed operators and Senior Reactor Operators.

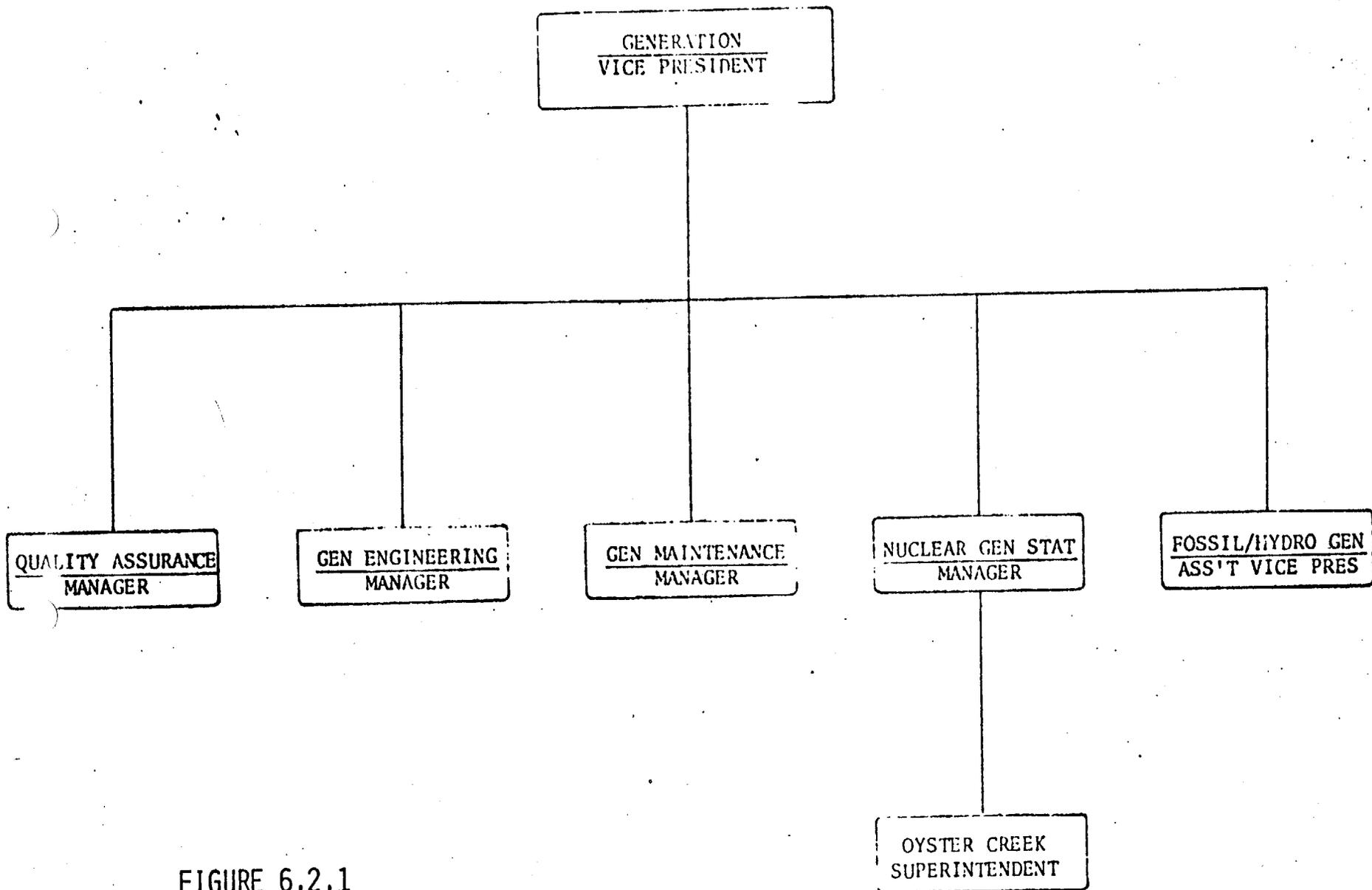


FIGURE 6.2.1
OYSTER CREEK STATION
MANAGEMENT ORGANIZATION CHART

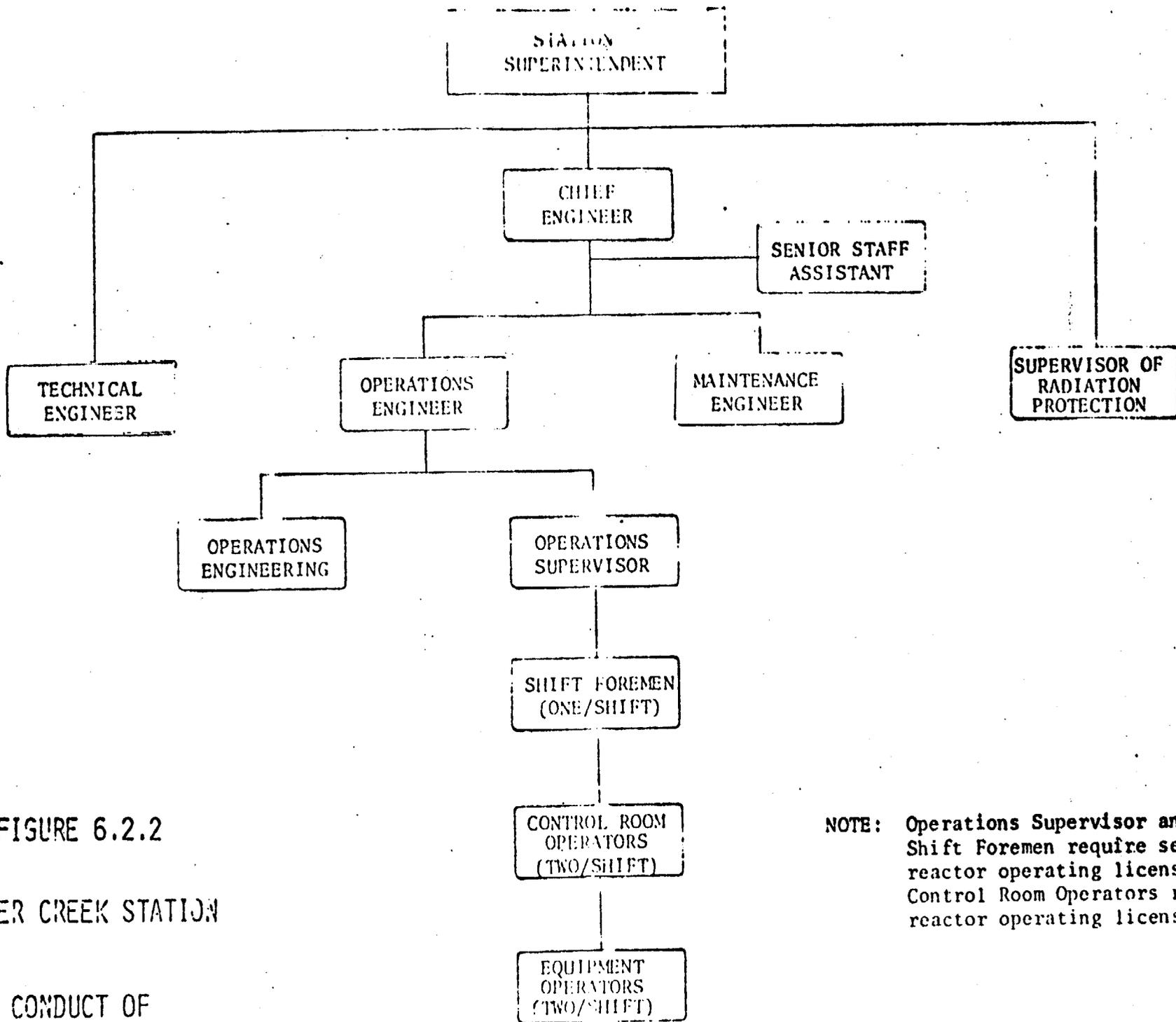


FIGURE 6.2.2

OYSTER CREEK STATION

CONDUCT OF OPERATIONS CHART

NOTE: Operations Supervisor and Shift Foremen require senior reactor operating licenses. Control Room Operators require reactor operating licenses.

6.5 REVIEW AND AUDIT

Organizational units for the review and audit of plant operations shall be constituted and have the responsibilities and authorities outlined below.

6.5.1 Plant Operations Review Committee

a. Membership:

- (1) Chairman: Station Superintendent
- (2) Chief Engineer
- (3) Operations Engineer
- (4) Technical Engineer
- (5) Maintenance Engineer
- (6) Supervisor-Radiation Protection
- (7) Two members of GORB as appointed by GORB chairman
- (8) The qualifications of the plant members of the Plant Operations Review Committee with regard to the combined experience and technical specialties of the individual members shall be maintained at a level at least equal to or higher than as described in 6.3.1 above.

b. Meeting Frequency: Monthly, and as required on call of the chairman.

c. Quorum: Chairman, plus three members.

d. Responsibilities:

- (1) Review all proposed normal, abnormal, maintenance and emergency operating procedures involving:
 1. Normal startup, operation, and shutdown of the complete plant and of all systems and components involving nuclear safety of the plant.
 2. Specific and foreseen potential malfunctions of systems or components, including alarms and abnormal reactivity changes.
 3. Emergency conditions involving possible or actual release of radioactive materials.

4. Preventive or corrective maintenance operations which could have an effect on the safety of the reactor.
5. Refueling operations.
6. Radiation control procedures. These procedures will show permissible radiation exposure levels and methods for control of radiation exposure.

Any other proposed procedures or changes thereto determined by the Station Superintendent to affect nuclear safety shall also be reviewed by the Plant Operations Review Committee.

- (2) Review all proposed tests and experiments that affect nuclear safety.
- (3) Review proposed changes to Technical Specifications.
- (4) Review proposed changes or modifications to plant systems or equipment, which changes would require a change in or would be covered by procedures in (1) above.
- (5) Review plant operation to detect potential safety hazards.
- (6) Investigate all reported instances of violations of Technical Specifications, including reporting evaluation and recommendations to prevent recurrence, to the Manager, Generating Stations-Nuclear, and to the Chairman of the General Office Review Board.
- (7) Perform special reviews and investigations and render reports thereon as requested by the Chairman of the General Office Review Board.

e. Authority:

- (1) The Plant Operations Review Committee shall be advisory.
- (2) The Plant Operations Review Committee shall recommend to the Station Superintendent approval or disapproval of proposals under items d (1) through (5) above.

In the event of disagreement between the recommendations of the Plant Operations Review Committee and the actions contemplated by the Station Superintendent on safety matters not clearly or implicitly excluded from consideration by the General Office Review Board, the course determined by the Station Superintendent to be the more conservative will be followed with immediate notification of the Manager, Generating Stations-Nuclear, and to the Chairman of the General Office Review Board.

- (3) The Plant Operations Review Committee shall report to the Chairman of the General Office Review Board on all reviews and investigations conducted under items d (6) and (7).
- (4) The Plant Operations Review Committee shall make tentative determinations regarding whether or not proposals considered by the committee involve unreviewed safety questions. This determination shall be subject to review and approval by the General Office Review Board.

f. Records:

Minutes shall be kept for all meetings of the Plant Operations Review Committee. Copies of the minutes shall be forwarded to the Chairman of the General Office Review Board.

g. Procedures:

Written administrative procedures for committee operation shall be prepared and maintained describing the method for submission and content of presentations to the committee provisions for use of subcommittees, review and approval by members of written committee evaluations and recommendations, dissemination of minutes, and such other matters as may be appropriate.

6.5.2 General Office Review Board

a. Membership:

- (1) Chairman and vice chairman appointed by name by company president.
- (2) Six members designated by the chairman.
- (3) Technical consultants, as required. Duly appointed consultant members shall have equal vote with permanent members of the Board.
- (4) Members of the company staff holding line responsibility for day-to-day operation of the Oyster Creek Nuclear Generating Station shall not serve as voting members of the General Office Review Board.
- (5) The qualifications of the General Office Review Board with regard to the combined experience and technical specialties of the individual members shall be maintained at a level equal to or higher than the following:

Reactor Engineering

Minimum qualifications: Engineering graduate or equivalent with over ten years' experience in nuclear power field and over five years' responsible engineering management.

Utility Operations

Minimum qualifications: Engineering graduate or equivalent with ten years' experience in utility operations and with over five years' responsible engineering management.

Reactor Physics

Minimum qualifications: Physics graduate or equivalent with over five years' experience in reactor physics work.

Heat and Fluid Flow

Minimum qualifications: Engineering or physics graduate or equivalent with over five years' experience in heat and fluid flow analysis.

Environmental Analysis

Minimum qualifications: Engineering graduate or equivalent with over five years' experience in environmental hazard analysis.

Reactor Control & Instrumentation

Minimum qualifications: Engineering graduate or equivalent with over five years' experience in nuclear engineering.

Power Plant Operations

Minimum qualifications: Engineering graduate or equivalent with over five years' experience in power plant operations.

Safety Analysis

Minimum qualifications: Engineering graduate or equivalent with over five years' experience in nuclear engineering.

- b. Meeting Frequency: Semiannually, and as required on call of the chairman.
- c. Quorum: Chairman or vice chairman, plus three members.
- d. Responsibilities:
 - (1) Review proposed changes to the operating license including Technical Specifications.

- (2) Review matters including proposed changes or modifications to plant systems or equipment referred to it by the Plant Operations Review Committee or by the Station Superintendent.
- (3) Review minutes of meetings of the Plant Operations Review Committee to determine if matters considered by that committee involve unreviewed safety questions.
- (4) Conduct periodic audits of plant operations at least quarterly.
- (5) Investigate all reported instances of violations of Technical Specifications, reporting findings and recommendations to prevent recurrence to the company president.
- (6) Perform special reviews and investigations and render reports thereon as requested by the company president.

e. Authority:

- (1) Approve proposed changes to the operating license including Technical Specifications and revised bases for submission to the NRC.
- (2) Approve for implementation changes or modifications to plant systems or equipment, provided such changes or modifications do not involve unreviewed safety questions.
- (3) Evaluate actions taken under paragraph 6.4.1.e(2)(a).
- (4) Recommend to the company president appropriate action to prevent recurrence of any violations of Technical Specifications.

f. Records:

Minutes shall be recorded for all meetings of the General Office Review Board. Copies of the minutes shall be forwarded to the company president, Manager, Generating Stations-Nuclear, and the Station Superintendent, and such others as the chairman may designate.

g. Procedures:

Written administrative procedures for committee operation shall be prepared and maintained describing the method for submission and content of presentations to the committee, provisions for use of subcommittees, review and approval by members of written committee evaluations and recommendations, dissemination of minutes, and such other matters as may be appropriate.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken in the event of an Reportable OCCURRENCE:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each Reportable Occurrence Report submitted to the Commission shall be reviewed by the Plant Operations Review Committee and submitted to the General Office Review Board and Manager, Generating Stations-Nuclear.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. If any Safety Limit is exceeded, the reactor shall be shut down immediately until the Commission authorizes resumption of operation.
- b. The Safety Limit violation shall be reported to the Commission, Manager Generating Stations-Nuclear, and GORB Chairman.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the Plant Operations Review Committee and submitted to the Manager, Generating Stations-Nuclear. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission within 10 days of the violation. It shall also be submitted to the General Office Review Board Chairman.

6.8 PROCEDURES

6.8.1 Detailed written procedures with appropriate check-off lists and instructions shall be provided for the following conditions:

1. Normal startup, operation, and shutdown of the complete plant and of all systems and components involving nuclear safety of the plant.
2. Specific and foreseen potential malfunctions of systems or components, including alarms and abnormal reactivity changes.
3. Emergency conditions involving possible or actual release of radioactive materials.

4. Preventive or corrective maintenance operations which could have an effect on the safety of the reactor.
 5. Refueling operations.
- 6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the Plant Operations Review Committee and approved by the Station Superintendent prior to implementation and periodically as specified in the Administrative Procedures.
- 6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:
- a. The intent of the original procedure is not altered.
 - b. The change is approved by two members of the supervisory staff, at least one of whom shall be a shift foreman.
 - c. The change is documented, subsequently reviewed by the Plant Operations Review Committee and approved by the Station Superintendent as specified in the Administrative Procedures.

6.9 REPORTING REQUIREMENTS

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following identified reports shall be submitted to the Director of the appropriate Regional Office of Inspection and Enforcement unless otherwise noted.

6.9.1. Routine Reports

- a. Startup Report. A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant. The report shall address each of the tests identified in the FSAR and shall in general include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

- b. Annual Operating Report.^{1/} Routing operating reports covering the operation of the unit during the previous calendar year should be submitted prior to March 1 of each year. The initial report shall be submitted prior to March 1 of the year following initial criticality.

The annual operating reports made by licensees shall provide a comprehensive summary of the operating experience gained during the year, even though some repetition of previously reported information may be involved. References in the annual operating report to previously submitted reports shall be clear.

^{1/}A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station.

Each annual operating report shall include:

- (1) A narrative summary of operating experience during the report period relating to safe operation of the facility, including safety-related maintenance not covered in item 1.b.(2)(e) below.
- (2) For each outage or forced reduction in power^{2/} of over twenty percent of design power level where the reduction extends for greater than four hours:
 - (a) the proximate cause and the system and major component involved (if the outage or forced reduction in power involved equipment malfunction);
 - (b) a brief discussion of (or reference to reports of) any reportable occurrences pertaining to the outage or power reduction;
 - (c) corrective action taken to reduce the probability of recurrence, if appropriate;

^{2/} The term "forced reduction in power" is normally defined in the electric power industry as the occurrence of a component failure or other condition which requires that the load on the unit be reduced for corrective action immediately or up to and including the very next weekend. Note that routine preventive maintenance, surveillance and calibration activities requiring power reductions are not covered by this section.

- (d) operating time lost as a result of the outage or power reduction (for scheduled or forced outages,^{3/} use the generator off-line hours; for forced reductions in power, use the approximate duration of operation at reduced power);
 - (e) a description of major safety-related corrective maintenance performed during the outage or power reduction, including the system and component involved and identification of the critical path activity dictating the length of the outage or power reduction; and
 - (f) a report of any single release of radioactivity or radiation exposure specifically associated with the outage which accounts for more than 10% of the allowable annual values.
- (3) A tabulation on an annual basis of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according

^{3/} The term "forced outage" is normally defined in the electric power industry as the occurrence of a component failure or other condition which requires that the unit be removed from service for corrective action immediately or up to and including the very next weekend.

to work and job functions,^{4/} e.g.; reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

- (4) Indications of failed fuel resulting from irradiated fuel examinations, including eddy current tests, ultrasonic tests, or visual examinations completed during the report period.

- c. Monthly Operating Report. Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Office of Inspection and Enforcement U.S. Nuclear Regulatory Commission, Washington, D.C. 20555,

^{4/} This tabulation supplements the requirements of §20.407 of 10 CFR Part 20.

with a copy to the appropriate Regional Office, to arrive no later than the tenth of each month following the calendar month covered by the report.

6.9.2. Reportable Occurrences

Reportable occurrences, including corrective actions and measures to prevent reoccurrences, shall be reported to the NRC. Supplemental reports may be required to fully describe final resolution of occurrence. In case of corrected or supplemental reports, a licensee event report shall be completed and reference shall be made to the original report date.

- a. Prompt Notification With Written Followup. The types of events listed below shall be reported as expeditiously as possible, but within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the appropriate Regional Office, or his designate no later than the first working day following the event, with a written followup report within two weeks. The written followup report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

- (1) Failure of the reactor protection system or other systems subject to limiting safety system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety system setting in the technical specifications or failure to complete the required protective function.

Note: Instrument drift discovered as a result of testing need not be reported under this item but may be reportable under items 2.a(5), 2.a(6), or 2.b(1) below.

- (2) Operation of the unit or affected systems when any parameter or operation subject to a limiting condition is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.

Note: If specified action is taken when a system is found to be operating between the most conservative and the least conservative aspects of a limiting condition for operation listed in the technical specifications,

the limiting condition for operation is not considered to have been violated and need not be reported under this item, but it may be reportable under item 2.b(2) below.

- (3) Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.

Note: Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

- (4) Reactivity anomalies, involving disagreement with the predicted value of reactivity balance under steady state conditions during power operation, greater than or equal to 1% $\Delta k/k$; a calculated reactivity balance indicating a shutdown margin less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds or, if sub-critical, an unplanned reactivity insertion of more than 0.5% $\Delta k/k$ or occurrence of any unplanned criticality.

(5) Failure or malfunction of one or more components which prevents or could prevent, by itself, the fulfillment of the functional requirements of system(s) used to cope with accidents analyzed in the SAR.

(6) Personnel error or procedural inadequacy which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the SAR.

Note: For items 2.a(5) and 2.a(6) reduced redundancy that does not result in a loss of system function need not be reported under this section but may be reportable under items 2.b(2) and 2.b(3) below.

(7) Conditions arising from natural or man-made events that, as a direct result of the event require plant shutdown, operation of safety systems, or other protective measures required by technical specifications.

(8) Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the

technical specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.

- (9) Performance of structures, systems, or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during plant life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

Note: This item is intended to provide for reporting of potentially generic problems.

- b. Thirty Day Written Reports. The reportable occurrences discussed below shall be the subject of written reports to the Director of the appropriate Regional Office within thirty days of occurrence of the event. The written report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee

event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

- (1) Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.
- (2) Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

Note: Routine surveillance testing, instrument calibration, or preventative maintenance which require system configurations as described in items 2.b(1) and 2.b(2) need not be reported except where test results themselves reveal a degraded mode as described above.

- (3) Observed inadequacies in the implementation of administrative or procedural controls which threaten to

cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.

- (4) Abnormal degradation of systems other than those specified in item 2.a(3) above designed to contain radioactive material resulting from the fission process.

Note: Sealed sources or calibration sources are not included under this item. Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

6.9.3. Unique Reporting Requirements

Special reports shall be submitted to the Director of Regulatory Operations Regional Office within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification.

- a. Materials Radiation Surveillance Specimen Reports (4.3A)
- b. Integrated Primary Containment Leakage Rate Tests (4.5)
- c. Semi-annual reports specifying effluent release shall be submitted to the NRC. These reports shall include the following:

(1). Radioactive Effluent Releases

A statement of the quantities of radioactive effluents released from the plant with data summarized on a monthly basis following the format of USAEC Regulatory Guide 1.21.

(a). Gaseous Effluents

1. Gross Radioactivity Releases

- a. Total gross radioactivity (in curies), primarily noble and activation gases.
- b. Maximum gross radioactivity release rate during any one-hour period.
- c. Total gross radioactivity (in curies) by nuclide released based on representative isotopic analyses performed.
- d. Percent of technical specification limit.

2. Iodine Releases

- a. Total iodine radioactivity (in curies) by nuclide released based on representative isotopic analyses performed.
- b. Percent of technical specification limit for I-131 released.

3. Particulate Releases

- a. Total gross radioactivity (β, γ) released (in curies) excluding background radioactivity.
- b. Gross alpha radioactivity released (in curies) excluding background radioactivity.
- c. Total gross radioactivity (in curies) of nuclides with half-lives greater than eight days.
- d. Percent of technical specification limit for particulate radioactivity with half-lives greater than eight days.

4. Liquid Effluents

- a. Total gross radioactivity (β, γ) released (in curies) excluding tritium and average concentration released to the unrestricted area.
- b. The maximum concentration of gross radioactivity (β, γ) released to the unrestricted area (averaged over the period of release).
- c. Total tritium and total alpha radioactivity (in curies) released and average concentration released to the unrestricted area.
- d. Total dissolved gas radioactivity (in curies) and average concentration released to the unrestricted area.
- e. Total volume (in liters) of liquid waste released.
- f. Total volume (in liters) of dilution water used prior to release from the restricted area.
- g. Total gross radioactivity (in curies) by nuclide released based on representative isotopic analyses performed.
- h. Percent of technical specification limit for total radioactivity.

(2). Solid Waste

- (a). The total amount of solid waste shipped (in cubic feet).
- (b). The total estimated radioactivity (in curies) involved.
- (c). Disposition including date and destination.

(3). Environmental Monitoring

- (a). For each medium sampled during the reporting period, e.g., air, baybottom, surface water, soil, fish, include:
 1. Number of sampling locations.
 2. Total number of samples.
 3. Number of locations at which levels are found to be significantly above local backgrounds, and
 4. Highest, lowest, and the average concentrations or level of radiation for the sampling point with the highest average and description of the location of that point with respect to the site.

- (b). If levels of radioactive materials in environmental media as determined by an environmental monitoring program indicate the likelihood of public intakes in excess of 1% of those that could result from continuous exposure to the concentration values listed in Appendix B, Table II, Part 20 estimates of the likely resultant exposure to individuals and to population groups, and assumptions upon which estimates are based shall be provided.
- (c). If statistically significant variations of offsite environmental concentrations with time are observed, correlation of these results with effluent release shall be provided.

6.10 RECORD RETENTION

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. REPORTABLE OCCURRENCE Reports
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of reactor tests and experiments.
- f. Records of changes made to Operating Procedures.
- g. Records of radioactive shipments.
- h. Records of sealed source leak tests and results.
- i. Records of annual physical inventory of all source material of record.

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

- a. Record and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of facility radiation and contamination surveys.
- d. Records of radiation exposure for all individuals entering radiation control areas.
- e. Records of gaseous and liquid radioactive material released to the environs.
- f. Records of transient or operational cycles for those facility components designed for a limited number of transients or cycles.
- g. Records of training and qualification for current members of the plant staff.

- h. Records of inservice inspections performed pursuant to these Technical Specifications.
- i. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- j. Records of meetings of the Plant Operations Review Committee and the General Office Review Board.

6.10.3 Quality Assurance Records shall be retained as specified by the Quality Assurance Plan.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 RESPIRATORY PROTECTION PROGRAM

ALLOWANCE

- 6.12.1 Pursuant to 10 CFR 20.103(c)(1) and (3), allowance may be made for the use of respiratory protective equipment in conjunction with activities authorized by the operating license for this facility in determining whether individuals in restricted areas are exposed to concentrations in excess of the limits specified in Appendix B, Table I, Column 1, of 10 CFR 20, subject to the following conditions and limitations:
- a. The limits provided in Section 20.103(a) and (b) shall not be exceeded.
 - b. If the radioactive material is of such form that intake through the skin or other additional route is likely, individual exposures to radioactive material shall be controlled so that the radioactive content of any critical organ from all routes of intake averaged over 7 consecutive days does not exceed that which would result from inhaling such radioactive material for 40 hours at the pertinent concentration values provided in Appendix B, Table I, Column 1, of 10 CFR 20.
 - c. For radioactive materials designated "Sub" in the "Isotope" column of Appendix B, Table I, Column 1 of 10 CFR 20, the concentration value specified shall be based upon exposure to the material as an external radiation source. Individual exposures to these materials shall be accounted for as part of the limitation on individual dose in §20.101. These materials shall be subject to applicable process and other engineering controls.

PROTECTION PROGRAM

6.12.2 In all operations in which adequate limitation of the inhalation of radioactive material by the use of process or other engineering controls is impracticable, the licensee may permit an individual in a restricted area to use respiratory protective equipment to limit the inhalation of airborne radioactive material, provided:

- a. The limits specified in 6.12.1 above are not exceeded.
- b. Respiratory protective equipment is selected and used so that the peak concentrations of airborne radioactive material inhaled by an individual wearing the equipment do not exceed the pertinent concentration values specified in Appendix B, Table I, Column 1, of 10 CFR 20. For the purposes of this subparagraph, the concentration of radioactive material that is inhaled when respirators are worn may be determined by dividing the ambient airborne concentration by the protection factor specified in Table 6.12.1 for the respirator protective equipment worn.

If the intake of radioactivity is later determined by other measurements to have been different than that initially estimated, the later quantity shall be used in evaluating the exposures.

- c. The licensee advises each respirator user that he may leave the area at any time for relief from respirator use in case of equipment malfunction, physical or psychological discomfort, or any other condition that might cause reduction in the protection afforded the wearer.
- d. The licensee maintains a respiratory protective program adequate to assure that the requirements above are met and incorporates practices for respiratory protection consistent with those recommended by the American National Standards Institute (ANSI-Z88.2-1969). Such a program shall include:
 1. Air sampling and other surveys sufficient to identify the hazard, to evaluate individual exposures, and to permit proper selection of respiratory protective equipment.
 2. Written procedures to assure proper selection, supervision, and training of personnel using such protective equipment.
 3. Written procedures to assure the adequate fitting of respirators, and the testing of respiratory protective equipment for operability immediately prior to use.
 4. Written procedures for maintenance to assure full effectiveness of respiratory protective equipment, including issuance, cleaning and decontamination, inspection, repair, and storage.

5. Written operational and administrative procedures for proper use of respiratory protective equipment including provisions for planned limitations on working times as necessitated by operation conditions.
 6. Bioassays and/or whole body counts of individuals (and other surveys, as appropriate) to evaluate individual exposures and to assess protection actually provided.
- e. The licensee shall use equipment approved by the U. S. Bureau of Mines^{5/} under its appropriate approval schedules as set forth in Table 6.12.1. Equipment not approved under U. S. Bureau of Mines^{5/} approval schedules shall be used only if the licensee has evaluated the equipment and can demonstrate by testing, or on the basis of reliable test information, that the material and performance characteristics of the equipment are at least equal to those afforded by U. S. Bureau of Mines-approved equipment of the same type, as specified in Table 6.12.1.
 - f. Unless otherwise authorized by the Commission, the licensee shall not assign protection factors in excess of those specified in Table 6.12.1 in selecting and using respiratory protective equipment.

REVOCATION

6.12.5 The specifications of Section 6.12 shall be revoked in their entirety upon adoption of the proposed change to 10 CFR 20, Section 20.103, which would make such provisions unnecessary.

6.15 HIGH RADIATION AREA

6.15.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20:

- a. Each High Radiation Area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Radiation Work Permit and any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. Each High Radiation Area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 6.13.1(a) above, and in addition locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the shift supervisor and radiation protection foreman on duty.

5/ and/or the National Institute of Health and Occupational Safety

TABLE 6.12.1
PROTECTION FACTORS FOR RESPIRATORS

DESCRIPTION	MODES ¹	PROTECTION FACTORS ² PARTICULATES AND VAPOURS AND GASES EXCEPT TRITIUM OXIDE ³	GUIDES TO SELECTION OF EQUIPMENT BUREAU OF MINES ^{5/} APPROVAL SCHEDULES* FOR EQUIPMENT CAPABLE OF PROVIDING AT LEAST EQUIVALENT PROTECTION FACTORS *or schedule superseding for equipment of type listed
I. AIR-PURIFYING RESPIRATORS Facepiece, half-mask ^{4,7} Facepiece, full ⁷	NP NP	5 100	21B 30 CFR § 14.4(b)(4) 21B 30 CFR § 14.4(b)(5); 14F 30 CFR 13
II. ATMOSPHERE-SUPPLYING RESPIRATOR 1. <u>Airline respirator</u> Facepiece, half-mask Facepiece, full Facepiece, full ⁷ Facepiece, full Hood Suit	CF CF D PD CF CF	100 1,000 100 1,000 5 5	19B 30 CFR § 12.2(c)(2) Type C(i) 19B 30 CFR § 12.2(c)(2) Type C(i) 19B 30 CFR § 12.2(c)(2) Type C(ii) 19B 30 CFR § 12.2(c)(2) Type C(iii) 5 6
2. <u>Self-contained breathing apparatus (SCBA)</u> Facepiece, full ⁷ Facepiece, full Facepiece, full	D PD R	100 1,000 100	13E 30 CFR § 11.4(b)(2)(i) 13E 30 CFR § 11.4(b)(2)(ii) 13E 30 CFR § 11.4(b)(1)
III. COMBINATION RESPIRATOR Any combination of air- purifying and atmosphere- supplying respirator		Protection factor for type and mode of operation as listed above	19B CFR § 12.2(e) or applicable schedules as listed above

6-31

1, 2, 3, 4, 5, 6, 7 [These notes are on the following pages]

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TABLE 6.12.1 (Continued)

¹ See the following symbols:

- CF: continuous flow
- D: demand
- NP: negative pressure (i.e., negative phase during inhalation)
- PD: pressure demand (i.e., always positive pressure)
- R: recirculating (closed circuit)

² (a) For purposes of this specification the protection factor is a measure of the degree of protection afforded by a respirator, defined as the ratio of the concentration of airborne radioactive material outside the respiratory protective equipment to that inside the equipment (usually inside the facepiece) under conditions of use. It is applied to the ambient airborne concentration to estimate the concentration inhaled by the wearer according to the following formula:

$$\text{Concentration Inhaled} = \frac{\text{Ambient Airborne Concentration}}{\text{Protection Factor}}$$

(b) The protection factors apply:

- (i) only for trained individuals wearing properly fitted respirators used and maintained under supervision in a well-planned respiratory protective program.
- (ii) for air-purifying respirators only when high efficiency [above 99.9% removal efficiency by U.S. Bureau of Mines type dioctyl phthalate (DOP) test] particulate filters and/or sorbents appropriate to the hazard are used in atmospheres not deficient in oxygen.
- (iii) for atmosphere-supplying respirators only when supplied with adequate respirable air.

³ Excluding radioactive contaminants that present an absorption or submersion hazard. For tritium oxide approximately half of the intake occurs by absorption through the skin so that an overall protection factor of not more than approximately 2 is appropriate when atmosphere-supplying respirators are used to protect against tritium oxide. Air-purifying respirators are not recommended for use against tritium oxide. See also footnote ⁵, below, concerning supplied-air suits and hoods.

⁴ Under chin type only. Not recommended for use where it might be possible for the ambient airborne concentration to reach instantaneous values greater than 50 times the pertinent values in Appendix B, Table I, Column 1 of 10 CFR Part 20.

TABLE 6.12.1 (Continued)

- ⁵ Appropriate protection factors must be determined taking account of the design of the suit or hood and its permeability to the contaminant under conditions of use. No protection factor greater than 1,000 shall be used except as authorized by the Commission.
- ⁶ No approval schedules currently available for this equipment. Equipment must be evaluated by testing or on basis of available test information.
- ⁷ Only for shaven faces.

NOTE 1: Protection factors for respirators, as may be approved by the U.S. Bureau of Mines ^{5/} according to approval schedules for respirators to protect against airborne radionuclides, may be used to the extent that they do not exceed the protection factors listed in this Table. The protection factors in this Table may not be appropriate to circumstances where chemical or other respiratory hazards exist in addition to radioactive hazards. The selection and use of respirators for such circumstances should take into account approvals of the U.S. Bureau of Mines ^{5/} in accordance with its applicable schedules.

NOTE 2: Radioactive contaminants for which the concentration values in Appendix B, Table I of this part are based on internal dose due to inhalation may, in addition, present external exposure hazards at high concentrations. The external exposure hazard is based on dose, which may be limited to that given by external dose limits.

5/ and/or the National Institute of Health and Occupational Safety



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 10 TO FACILITY LICENSE NO. DPR-16

JERSEY CENTRAL POWER AND LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

Introduction

By letter dated January 10, 1975, and modified by your letter dated May 5, 1975, Jersey Central Power and Light Company proposed changes to the Technical Specifications appended to Facility Operating License No. DPR-16, for the Oyster Creek Nuclear Generating Station, Unit 1. The proposed changes involve changes to the administrative controls including changes to the reporting requirements.

Discussion

The proposed changes would be administrative in nature and would affect the conduct of operation. The proposed changes are intended to provide uniform license requirements. Areas covered by the proposed uniform specifications include management procedures involved with operating the reactor, reporting requirements, abnormal occurrence definition change, and a respiratory protection program.

In Section 208 of the Energy Reorganization Act of 1974 "abnormal occurrences" is defined as an unscheduled incident or event which the Commission determines is significant from the standpoint of public health or safety. The term "abnormal occurrence" is reserved for usage by NRC. Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4, enumerates required reports consistent with Section 208. The proposed change to required reports identifies the reports required of all licensees not already identified by the regulations and those unique to this facility. The proposal would formalize present reporting and would delete any reports no longer needed for assessment of safety related activities. In addition, a radiation protection program delineates use of respiratory equipment in the event personnel are to be exposed to concentrations in excess of Part 20 concentrations.

Evaluation

The new guidance for reporting operating information does not identify any event as an "abnormal occurrence". The proposed reporting requirements also delete reporting of information no longer required and duplication of reported information. The standardization of required reports and desired format for the information will permit more rapid recognition of potential problems.

Incorporating the currently accepted respiratory protection program at this time assures that a consistent method of using respiratory equipment is immediately available whenever needed. Similar changes are being approved for all power reactor licensees, so all licensees will have the same requirements presented in a uniform manner.

During our review of the proposed changes, we found that certain modifications to the proposal were necessary to have conformance with the desired regulatory position. These changes were discussed with your staff and have been incorporated into the proposal.

We have concluded that the proposal as modified improves the licensee's program for evaluating plant performance and the reporting of the operating information needed by the Commission to assess safety related activities and is acceptable. The modified reporting program is consistent with the guidance provided by Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4. The administrative controls are consistent with requirements being incorporated in Technical Specifications for new licensed facilities.

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental statement, negative declaration, or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that:
(1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does

not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: 7 11 68

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-219

JERSEY CENTRAL POWER AND LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO PROVISIONAL
OPERATING LICENSE

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 10 to Provisional License No. DPR-16 issued to Jersey Central Power and Light Company which revised Technical Specifications for operation of the Oyster Creek Nuclear Generating Station, Unit 1, located in Ocean County, New Jersey. The amendment is effective as of its date of issuance.

The amendment incorporate changes related to Administrative Controls into the Technical Specifications for Oyster Creek Nuclear Generating Station, Unit 1.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental statement, negative declaration or

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environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the applications for amendment dated January 10, 1975 and modified by your letter dated May 5, 1975, (2) Amendment No. 10 to License No. DPR-16, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Ocean County Library, 15 Hooper Avenue, Toms River, New Jersey 08753.

A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this 16th day of January 1976

FOR THE NUCLEAR REGULATORY COMMISSION

George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

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