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JUN 12 1972

Docket No. 50-219

Jersey Central Power & Light Company
ATTN: Mr. R. H. Sims, Vice President
Madison Avenue at Punch Bowl Road
Morristown, New Jersey 07960

Gentlemen:

By letters dated April 6, April 19, and May 26, 1972, you submitted Facility Change Request No. 2, Facility Change Request No. 3, and Change Request No. 12 to the Technical Specifications for Oyster Creek Nuclear Power Plant. In support of these proposed changes, you also submitted Amendment 69 to the Oyster Creek Station Facility Description and Safety Analysis Report.

We have reviewed the above requests and supporting analyses and we have concluded that operation with the cycle 2 core, as described in Facility Change Requests No. 2 and No. 3, and implementation of the changes to the Technical Specifications, as proposed in Technical Specification Change Request No. 12, do not present significant hazards considerations not described or implicit in the Oyster Creek Safety Analysis Report and that there is reasonable assurance that the health and safety of the public will not be endangered.

Accordingly, pursuant to Section 50.59 of 10 CFR Part 50, you are hereby authorized to operate the Oyster Creek Plant with the cycle 2 core as described in Facility Change Requests Nos. 2 and 3 and the Technical Specifications of Provisional Operating License No. DPR-16 are hereby changed as indicated in Attachment A.

Sincerely,

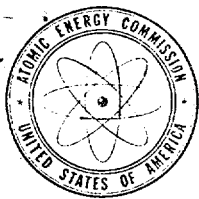
/s/

Donald J. Skovholt
Assistant Director
for Operating Reactors
Directorate of Licensing

Enclosure:
Attachment A - Change to the
Technical Specifications

cc: George F. Trowbridge, Esquire

OFFICE ▶	L:OR-1	L:OR-1	L:OR-1	D:OR		
SURNAME ▶	<i>J. Wambach</i> TWambach:pd1	<i>SATeets</i> SATeets	<i>SDM for</i> RJSchemel	<i>DJSkovholt</i> DJSkovholt		<i>App'l</i> <i>RW</i>
DATE ▶	6/9/72	6/12/72	6/12/72	6/12/72		



UNITED STATES
ATOMIC ENERGY COMMISSION

WASHINGTON, D.C. 20545

JUN 12 1972

Docket No. 50-219

Jersey Central Power & Light Company
ATTN: Mr. R. H. Sims, Vice President
Madison Avenue at Punch Bowl Road
Morristown, New Jersey 07960

Gentlemen:

By letters dated April 6, April 19, and May 26, 1972, you submitted Facility Change Request No. 2, Facility Change Request No. 3, and Change Request No. 12 to the Technical Specifications for Oyster Creek Nuclear Power Plant. In support of these proposed changes, you also submitted Amendment 69 to the Oyster Creek Station Facility Description and Safety Analysis Report.

We have reviewed the above requests and supporting analyses and we have concluded that operation with the cycle 2 core, as described in Facility Change Requests No. 2 and No. 3, and implementation of the changes to the Technical Specifications, as proposed in Technical Specification Change Request No. 12, do not present significant hazards considerations not described or implicit in the Oyster Creek Safety Analysis Report and that there is reasonable assurance that the health and safety of the public will not be endangered.

Accordingly, pursuant to Section 50.59 of 10 CFR Part 50, you are hereby authorized to operate the Oyster Creek Plant with the cycle 2 core as described in Facility Change Requests Nos. 2 and 3 and the Technical Specifications of Provisional Operating License No. DPR-16 are hereby changed as indicated in Attachment A.

Sincerely,

A handwritten signature in cursive script, appearing to read "Donald J. Skovholt", is written over a horizontal line.

Donald J. Skovholt
Assistant Director
for Operating Reactors
Directorate of Licensing

Enclosure:
Attachment A - Change to the
Technical Specifications

cc: George F. Trowbridge, Esquire

ATTACHMENT A

CHANGE NO. 12 TO THE TECHNICAL SPECIFICATIONS

PROVISIONAL OPERATING LICENSE NO. DPR-16

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

<u>ITEM</u>	<u>LOCATION</u>	<u>CHANGE</u>
Basis statement for 2.2	Page 2.2-1 last paragraph	In the second sentence, change the last part of the sentence to read ". . . in combination with the turbine bypass system and the relief valves limit the reactor pressure to approximately 1110 psig (2)".
Basis statement for 2.2	Page 2.2-2 first paragraph	Change in second sentence "1183 psig" to "1188 psig". At end of paragraph, add footnote notation "(2)".
Reference (2)	Page 2.2-2	Change present reference to read "License Application Amendment 69, Section III".
Specification 2.3.3	Page 2.3-2	Change the Reactor High Pressure Scram Setpoint from " \leq 1070 psig" to " \leq 1060 psig".
Specification 2.3.4	Page 2.3-2	Change Reactor High Pressure Relief Valves setpoint from "1125 psig" to "1070 psig".
Specification 2.3.5	Page 2.3-2	Change Reactor High Pressure Isolation Condenser initiation setpoint from "1070 psig" to "1060 psig".
Basis statement for 2.3	Page 2.3-5 paragraph 2	In the 10th line, change "1070 psig" to "1060 psig".

<u>ITEM</u>	<u>LOCATION</u>	<u>CHANGE</u>
Basis statement for 2.3	Page 2.3-5 paragraph 2	In the 17th line, change "1183 psig" to "1188 psig".
Basis statement for 2.3	Page 2.3-6 paragraph 1	In the 4th line, change reference "(9)" to "(10)".
Basis statement for 2.3	Page 2.3-6 paragraph 2	In the last line, change "1136 psig (10)" to "1110 psig (9)".
Basis statement for 2.3	Page 2.3-7 paragraph 1	In the last line, change references from "(7,12)" to "(7,10)".
References	Page 2.3-7 Ref. (7)	Change "B.VII.4" to "B.VII.7".
	Ref. (9)	Change to "License Application Amendment 69, Section III-D-5"
	Ref. (12)	Delete.
Specification 3.2.3	Tables on page 3.2-1	Change the two tables in sequence, to those shown below

<u>Percent of Rod Length Inserted</u>	<u>Seconds</u>
5	0.375
20	0.900
50	2.00
90	5.00

<u>Percent of Rod Length Inserted</u>	<u>Seconds</u>
5	0.398
20	0.954
50	2.120
90	5.300

<u>ITEM</u>	<u>LOCATION</u>	<u>CHANGE</u>
Basis statement for 3.2	Page 3.2-4 paragraph 5	Delete 2nd and 3rd sentences and replace with "Figure III-1 of Amendment 69 to the FDSAR shows the control rod scram reactivity used in the transient analyses
Basis statement for 3.2	Page 3.2-5 paragraph 1, 1st sentence	Change "390" to "290".
Basis statement for 3.2	Page 3.2-5 paragraph 1, 2nd sentence	Change the last part of the sentence to read: ". . . compared to the typical time delay of about 210 milliseconds estimated from scram test results".
Basis statement for 3.2	Page 3.2-5 paragraph 1, 3rd sentence	Change to read: "Approximately the first 90 milliseconds of each of these time intervals result from the sensor and circuit delays; then the pilot scram solenoid de-energizes. Approximately 120 milliseconds later, the control rod motion is estimated to actually begin. However, 200 milliseconds is conservatively assumed for this time interval in the transient analyses and this is also included in the allowable scram insertion times of specification of 3.2.3."
Specification 3.4.B.2	Page 3.4-2	In the 1st paragraph, change "1865 Mwt" to "1740 Mwt" and "1690 Mwt" to "1640 Mwt".