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| ENGINEERED SAFETY FEATURE (ESF) RESPONSE TIME | The ESF RESPONSE TIME shall be that time interval from when the monitored parameter exceeds its ESF actuation setpoint at the channel sensor until the ESF equipment is capable of performing its safety function (i.e., the valves travel to their required positions, pump discharge pressures reach their required values, etc.). Times shall include diesel generator starting and sequence loading delays, where applicable. The response time may be measured by means of any series of sequential, overlapping, or total steps so that the entire response time is measured. In lieu of measurement, response time may be verified for selected components provided that the components and methodology for verification have been previously reviewed and approved by the NRC. |
|---|---|
| K _{n-1} | K_{n-1} is the K effective calculated by considering the actual CEA configuration and assuming that the fully or partially inserted full-length CEA of highest worth is fully withdrawn. |
| LEAKAGE | LEAKAGE shall be: |
| | a. <u>Identified LEAKAGE</u> |
| | LEAKAGE, such as that from pump seals or valve packing (except reactor coolant pump (RCP) seal water injection or leakoff), that is captured and conducted to collection systems or a sump or collecting tank; |
| | LEAKAGE into the containment atmosphere from sources that are both specifically located and known either not to interfere with the operation of leakage detection systems or not to be pressure boundary LEAKAGE; or |
| | Reactor Coolant System (RCS) LEAKAGE through a steam generator (SG) to the Secondary System. |
| | |
| | (continued) |

PALO VERDE UNITS 1,2,3

1.1-4

AMENDMENT NO. 117, 135

| LEAKAGE (continued) | b. Unidentified LEAKAGE |
|---------------------------------------|--|
| | All LEAKAGE that is not identified LEAKAGE; |
| | c. Pressure Boundary LEAKAGE |
| | LEAKAGE (except SG LEAKAGE) through a nonisolable fault in an RCS component body, pipe wall, or vessel wall. |
| MODE | A MODE shall correspond to any one inclusive combination of core reactivity condition, power level, cold leg reactor coolant temperature, and reactor vessel head closure bolt tensioning specified in Table 1.1-1 with fuel in the reactor vessel. |
| NEUTRON RATED THERMAL POWER (NRTP) | The indicated neutron flux at RTP. |
| OPERABLE - OPERABILITY | A system, subsystem, train, component, or device shall be OPERABLE or have OPERABILITY when it is capable of performing its specified safety function(s) and when all necessary attendant instrumentation, controls, normal or emergency electrical power, cooling and seal water, lubrication, and other auxiliary equipment that are required for the system, subsystem, train, component, or device to perform its specified safety function(s) are also capable of performing their related support function(s). |
| PHYSICS TESTS | PHYSICS TESTS shall be those tests performed to measure the fundamental nuclear characteristics of the reactor core and related instrumentation. These tests are: |
| | a. Described in Chapter 14, Initial Test Program of the UFSAR; |
| | Authorized under the provisions of 10 CFR 50.59; or |
| | c. Otherwise approved by the Nuclear Regulatory Commission. |
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PALO VERDE UNITS 1,2,3 1.1-5 AMENDMENT NO.119, 135

1.1 Definitions (continued)

RATED THERMAL POWER (RTP)

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REACTOR PROTECTIVE SYSTEM (RPS) RESPONSE TIME The RPS RESPONSE TIME shall be that time interval from when the monitored parameter exceeds its RPS trip setpoint at the channel sensor until electrical power to the CEAs drive mechanism is interrupted. The response time may be measured by means of any series of sequential, overlapping, or total steps so that the entire response time is measured. In lieu of measurement, response time may be verified for selected components provided that the components and methodology for verification have been previously reviewed and

RTP shall be a total reactor core heat transfer

rate to the reactor coolant of 3876 MWt.

SHUTDOWN MARGIN (SDM)

SDM shall be the instantaneous amount of reactivity by which the reactor is subcritical or would be subcritical from its present condition assuming:

- a. All full length CEAs (shutdown and regulating) are fully inserted except for the single CEA of highest reactivity worth, which is assumed to be fully withdrawn. With any full length CEAs not capable of being fully inserted, the withdrawn reactivity worth of these CEAs must be accounted for in the determination of SDM and
- b. There is no change in part length CEA position.

(continued)

approved by the NRC.