June 10, 1996

Mr. B. Ralph Sylvia
Executive Vice President
and Chief Nuclear Officer
Niagara Mohawk Power Corporation
Generation Business Group D-2
300 Erie Boulevard West
Syracuse, NY 13202

SUBJECT: ISSUANCE OF AMENDMENT FOR NINE MILE POINT NUCLEAR STATION, UNIT 2 (TAC NO. M94591)

Dear Mr. Sylvia:

The Commission has issued the enclosed Amendment No. 72 to Facility Operating License No. NPF-69 for the Nine Mile Point Nuclear Station, Unit 2 (NMP-2). The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated January 25, 1996.

The amendment revises TS 3/4.3.3, Emergency Core Cooling System Actuation Instrumentation, to more clearly define when, during shutdown and refueling, the Loss of Voltage and Degraded Voltage relays for the Loss of Power actuation trip functions are required to be operable.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

original signed by

Darl S. Hood, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-410

Enclosures: 1. Amendment No. 72 to NPF-69
2. Safety Evaluation

cc w/encls: See next page

*See previous concurrence
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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Office of Nuclear Reactor Regulation

Docket No. 50-410

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2. Safety Evaluation

cc w/encls: See next page

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DATED: June 10, 1996

AMENDMENT NO. 72 TO FACILITY OPERATING LICENSE NO. NPF-69-NINE MILE POINT UNIT 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-410

NINE MILE POINT NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 72
License No. NPF-69

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated January 25, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-69 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 72 are hereby incorporated into this license. Niagara Mohawk Power Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Jocelyn A. Mitchell

Jocelyn A. Mitchell, Acting Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 10, 1996

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 72 TO FACILITY OPERATING LICENSE NO. NPF-69

DOCKET NO. 50-410

Revise Appendix A as follows:

Remove Pages	Insert Pages
3/4 3-33	3/4 3-33
3/4 3-43	3/4 3-43*
3/4 3-44	3/4 3-44

*Overleaf page

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

TABLE NOTATIONS

* When the system is required to be OPERABLE per Specification 3.5.2 or 3.5.3.

** Required when the associated diesel generator is required to be OPERABLE.

(a) When a channel is placed in an inoperable status solely for performance of required surveillances, entry into associated Conditions and required ACTIONS may be delayed for up to 6 hours provided the associated function or the redundant function maintains ECCS initiation capability.

(b) Also actuates the associated division diesel generator.

(c) Not required to be OPERABLE when reactor steam dome pressure is less than or equal to 100 psig.

(d) The injection function of Drywell Pressure High and Manual Initiation is not required to be OPERABLE with indicated reactor vessel water level on the wide range instrument greater than level 8 setpoint coincident with the vessel pressure less than 600 psig because of hot calibration/cold operation level error.

(e) Provides signal to close HPCS pump injection valve only.

(f) Provides signal to HPCS pump suction valves only.

ACTION

ACTION 30 - With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement:

a. With one channel inoperable, place the inoperable channel in the tripped condition within 24 hours* or declare the associated system inoperable.

b. With more than one channel inoperable, declare the associated system inoperable.

ACTION 31 - With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement, place the inoperable channel in the tripped condition within 24 hours; restore the inoperable channel to OPERABLE status within 7 days or declare the associated system inoperable.

---

* The provisions of Specification 3.0.4 are not applicable.

TABLE 4.3.3.1-1 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TRIP FUNCTION	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE IS REQUIRED
D. Loss of Power (Divisions I & II)				
1. 4.16-kV Emergency Bus Undervoltage - Loss of Voltage	S	M	R	1, 2, 3, 4†, 5†
2. 4.16-kV Emergency Bus Undervoltage - Degraded Voltage	S	M	R	1, 2, 3, 4†, 5†
E. Loss of Power (Division III)				
1. 4.16-kV Emergency Bus Undervoltage - Loss of Voltage	S	M	R	1, 2, 3, 4†, 5†
2. 4.16-kV Emergency Bus Undervoltage - Degraded Voltage	S	M	R	1, 2, 3, 4†, 5†

TABLE NOTATIONS

* When the system is required to be OPERABLE per Specification 3.5.2.

** Not required to be OPERABLE when reactor steam dome pressure is less than or equal to 100 psig.

† Required when the associated diesel generator is required to be OPERABLE.

(a) Manual initiation switches shall be tested at least once per 18 months during shutdown. All other circuitry associated with manual initiation shall receive a CHANNEL FUNCTIONAL TEST at least once per 92 days as part of the circuitry required to be tested for automatic system actuation.

(b) The injection function of Drywell Pressure - High and Manual Initiation is not required to be OPERABLE with indicated reactor vessel water level on the wide range instrument greater than Level 8 setpoint coincident with the vessel pressure less than 600 psig due to the hot calibration/cold operation level error.

(c) Perform the calibration procedure for the Trip Setpoint at least once per 92 days.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 72 TO FACILITY OPERATING LICENSE NO. NPF-69

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION, UNIT 2

DOCKET NO. 50-410

1.0 INTRODUCTION

By letter dated January 25, 1996, Niagara Mohawk Power Corporation (the licensee) submitted a request for an operating license amendment to change the Technical Specifications (TSs) for Nine Mile Point Nuclear Station, Unit 2. The requested changes would revise footnotes in two tables referenced by TS 3/4.3.3, Emergency Core Cooling System Actuation Instrumentation, to more clearly define when, during cold shutdown and refueling, the Loss of Voltage and Degraded Voltage relays associated with the Loss of Power actuation trip functions are required to be operable. Specifically, the footnotes to Tables 3.3.3-1 and 4.3.3.1-1 currently state: "Required when ESF [Engineered Safety Features] equipment is required to be OPERABLE." These would be changed to state: "Required when the associated diesel generator is required to be OPERABLE."

2.0 BACKGROUND

Successful operation of the required safety functions of the Emergency Core Cooling Systems (ECCS) depends, in large measure, upon the availability of adequate power sources to energize the various components such as pump motors, motor-operated valves, and the associated control components. Loss of power (LOP) instrumentation monitors the 4.16 kV emergency buses. Offsite power is the preferred source for the 4.16 kV emergency buses. If the monitors sense that insufficient power is available, the buses are automatically disconnected from the offsite power sources and connected to the onsite diesel generator (DG) power sources.

Each 4.16 kV emergency bus has its own independent LOP instrumentation and associated trip logic circuits. The voltage for each bus is monitored at two levels, which can be considered as two different undervoltage functions: 4.16 kV Emergency Bus Undervoltage - Loss of Voltage, and 4.16 kV Emergency Bus Undervoltage - Degraded Voltage. Each function causes various bus transfers and disconnect operations. Each function is monitored by three undervoltage relays for each emergency bus, whose outputs are arranged in a two-out-of-three logic configuration. The channels include electronic equipment (e.g., trip units) that compares measured input signals with pre-established setpoints. When the setpoint is reached or exceeded, the

channel output relay actuates, which then sends a LOP trip signal to the associated trip logic circuits.

3.0 EVALUATION

Loss of voltage on a 4.16 kV emergency bus indicates that offsite power may be completely lost to the respective emergency bus. Therefore, the power supply to the bus is transferred from offsite power to DG power when the voltage on the bus drops below the loss of voltage function setpoint (loss of voltage with a short time delay). This ensures that adequate power will be available to the required equipment. Since the function of LOP instrumentation is to start and load the DGs, the 4.16 kV Emergency Bus Undervoltage - Loss of Voltage function associated with each emergency bus is only required to be operable when the associated DG is required to be operable. The proposed changes to the footnotes are consistent with this requirement.

A reduced voltage condition on a 4.16 kV emergency bus indicates that, while offsite power may not be completely lost to the respective emergency bus, available power may be insufficient for starting large ECCS motors without risking damage to the motors that could disable the ECCS function. Therefore, the power supply to the bus is transferred from offsite power to onsite DG power when the voltage on the bus drops below the degraded voltage function setpoint (degraded voltage with a time delay). This ensures that adequate power will be available to the required equipment. Since the function of LOP instrumentation is to start and load the DGs, the 4.16 kV Emergency Bus Undervoltage - Degraded Voltage function associated with each emergency bus is only required to be operable when the associated DG is required to be operable. The proposed changes to the footnotes are consistent with this requirement.

Accordingly, since the function of LOP instrumentation is to start and load the DGs, and LOP instrumentation associated with each emergency bus is only required to be operable when the associated DG is required to be operable, the NRC staff finds that the licensee's proposed changes to the footnotes of TS Tables 3.3.3-1 and 4.3.3.1-1 comply with applicable regulatory requirements and are acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the

amendment involves no significant hazards consideration, and there has been no public comment on such finding (61 FR 20851). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: V. Beaston
D. Hood

Date: June 10, 1996