



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

November 12, 1993

Docket No. 50-410

Mr. B. Ralph Sylvia  
Executive Vice President, Nuclear  
Niagara Mohawk Power Corporation  
301 Plainfield Road  
Syracuse, New York 13212

Dear Mr. Sylvia:

SUBJECT: ISSUANCE OF AMENDMENT FOR NINE MILE POINT NUCLEAR STATION,  
UNIT 2 (TAC NO. M86378)

The Commission has issued the enclosed Amendment No. 53 to Facility Operating License No. NPF-69 for the Nine Mile Point Nuclear Station, Unit 2. The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated May 7, 1993, as superseded September 28, 1993.

The amendment adds a new TS 3/4.10.7, "Inservice Leak and Hydrostatic Testing," to the Nine Mile Point Nuclear Station, Unit 2, TSs. The amendment also includes corresponding changes to the TS Index, Table 1.2, and provides Bases for TS 3/4.10.7. The added TS 3/4.10.7 permits the unit to remain in OPERATIONAL CONDITION 4 with average reactor coolant temperature being increased above 200 °F during reactor coolant system inservice leak or hydrostatic tests provided the maximum reactor coolant temperature does not exceed 212 °F and the following OPERATIONAL CONDITION 3 TSs are being met: (a) TS 3.3.2, "Isolation Actuation Instrumentation," Functions 1.a.2, 1.b, and 3.a and b of Table 3.3.2-1; (b) TS 3.6.5.1, "Secondary Containment Integrity;" (c) TS 3.6.5.2, "Secondary Containment Automatic Isolation Dampers;" and (d) TS 3.6.5.3, "Standby Gas Treatment System."

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

John E. Menning, Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 53 to NPF-69
2. Safety Evaluation

cc w/enclosures:  
See next page

9311300100 931112  
PDR ADOCK 05000410  
P PDR

*CP-1*  
*DFC*  
**NRC FILE CENTER COPY**

Mr. B. Ralph Sylvia  
Niagara Mohawk Power Corporation

Nine Mile Point Nuclear Station  
Unit 2

cc:

Mark J. Wetterhahn, Esquire  
Winston & Strawn  
1400 L Street, NW.  
Washington, DC 20005-3502

Regional Administrator, Region I  
U. S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, PA 19406

Mr. Richard Goldsmith  
Syracuse University  
College of Law  
E. I. White Hall Campus  
Syracuse, New York 12223

Charles Donaldson, Esquire  
Assistant Attorney General  
New York Department of Law  
120 Broadway  
New York, New York 10271

Resident Inspector  
Nine Mile Point Nuclear Station  
P. O. Box 126  
Lycoming, New York 13093

Mr. Richard M. Kessel  
Chair and Executive Director  
State Consumer Protection Board  
99 Washington Avenue  
Albany, New York 12210

Gary D. Wilson, Esquire  
Niagara Mohawk Power Corporation  
300 Erie Boulevard West  
Syracuse, New York 13202

Mr. John H. Mueller  
Plant Manager, Unit 2  
Nine Mile Point Nuclear Station  
Niagara Mohawk Power Corporation  
P. O. Box 32  
Lycoming, New York 13093

Mr. David K. Greene  
Manager Licensing  
Niagara Mohawk Power Corporation  
301 Plainfield Road  
Syracuse, New York 13212

Vice President - Nuclear Generation  
Nine Mile Point Nuclear Station  
Niagara Mohawk Power Corporation  
P. O. Box 32  
Lycoming, New York 13093

Ms. Donna Ross  
New York State Energy Office  
2 Empire State Plaza  
16th Floor  
Albany, New York 12223

Supervisor  
Town of Scriba  
Route 8, Box 382  
Oswego, New York 13126

Mr. B. Ralph Sylvia  
Executive Vice President, Nuclear  
Niagara Mohawk Power Corporation  
301 Plainfield Road  
Syracuse, New York 13212

Dear Mr. Sylvia:

SUBJECT: ISSUANCE OF AMENDMENT FOR NINE MILE POINT NUCLEAR STATION,  
UNIT 2 (TAC NO. M86378)

The Commission has issued the enclosed Amendment No. 53 to Facility Operating License No. NPF-69 for the Nine Mile Point Nuclear Station, Unit 2. The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated May 7, 1993, as superseded September 28, 1993.

The amendment adds a new TS 3/4.10.7, "Inservice Leak and Hydrostatic Testing," to the Nine Mile Point Nuclear Station, Unit 2, TSs. The amendment also includes corresponding changes to the TS Index, Table 1.2, and provides Bases for TS 3/4.10.7. The added TS 3/4.10.7 permits the unit to remain in OPERATIONAL CONDITION 4 with average reactor coolant temperature being increased above 200 °F during reactor coolant system inservice leak or hydrostatic tests provided the maximum reactor coolant temperature does not exceed 212 °F and the following OPERATIONAL CONDITION 3 TSs are being met: (a) TS 3.3.2, "Isolation Actuation Instrumentation," Functions 1.a.2, 1.b, and 3.a and b of Table 3.3.2-1; (b) TS 3.6.5.1, "Secondary Containment Integrity;" (c) TS 3.6.5.2, "Secondary Containment Automatic Isolation Dampers;" and (d) TS 3.6.5.3, "Standby Gas Treatment System."

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by:  
John E. Menning, Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 53 to NPF-69
- 2. Safety Evaluation

cc w/enclosures:  
See next page

Distribution:  
See attached sheet

LA: PDI-1 <i>MMB</i>	PM: PDI-1	PM: PDI-1	SEXB <i>SEXB</i>	OGC <i>ELB</i>	D: PDI-1 <i>D: PDI-1</i>
CVogan <i>CV</i>	DBrinkman <i>DB</i>	JMenning <i>JM</i>	RJones <i>RJ</i>	EHolder <i>EH</i>	RACapra <i>RA</i>
10/29/93	10/29/93	10/29/93	11/01/93	11/1/93	11/12/93

DATED: November 12, 1993

AMENDMENT NO. 53 TO FACILITY OPERATING LICENSE NO. NPF-69-NINE MILE POINT  
UNIT 2

Docket File  
NRC & Local PDRs  
PDI-1 Reading  
S. Varga, 14/E/4  
J. Calvo, 14/A/4  
R. Capra  
C. Vogan  
J. Menning  
OGC  
D. Hagan, 3302 MNBB  
G. Hill (2), P1-22  
C. Grimes, 11/F/23  
D. Brinkman  
ACRS (10)  
OPA  
OC/LFDCB  
PD plant-specific file  
C. Cowgill, Region I  
R. Jones, 8/E/23  
M. Razzaque, 8/E/23

cc: Plant Service list



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-410

NINE MILE POINT NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 53  
License No. NPF-69

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated May 7, 1993, as superseded September 28, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-69 is hereby amended to read as follows:

9311300102 931112  
PDR ADOCK 05000410  
P PDR

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 53 are hereby incorporated into this license. Niagara Mohawk Power Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 12, 1993

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 53 TO FACILITY OPERATING LICENSE NO. NPF-69

DOCKET NO. 50-410

Revise Appendix A as follows:

Remove Pages

xiv  
xx  
1-11  
--  
B 3/4 10-1

Insert Pages

xiv  
xx  
1-11  
3/4 10-7 (added page)  
B 3/4 10-1

**INDEX**

**LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS**

	<u>PAGE</u>
<b>3/4.9.10 CONTROL ROD REMOVAL</b>	
Single Control Rod Removal . . . . .	3/4 9-12
Multiple Control Rod Removal . . . . .	3/4 9-14
<b>3/4.9.11 RESIDUAL HEAT REMOVAL AND COOLANT CIRCULATION</b>	
High Water Level . . . . .	3/4 9-16
Low Water Level . . . . .	3/4 9-17
<b><u>3/4.10 SPECIAL TEXT EXCEPTIONS</u></b>	
3/4.10.1 PRIMARY CONTAINMENT INTEGRITY . . . . .	3/4 10-1
3/4.10.2 ROD SEQUENCE CONTROL SYSTEM . . . . .	3/4 10-2
3/4.10.3 SHUTDOWN MARGIN DEMONSTRATIONS . . . . .	3/4 10-3
3/4.10.4 RECIRCULATION LOOPS . . . . .	3/4 10-4
3/4.10.5 OXYGEN CONCENTRATION . . . . .	3/4 10-5
3/4.10.6 TRAINING STARTUPS . . . . .	3/4 10-6
3/4.10.7 INSERVICE LEAK AND HYDROSTATIC TESTING . . . . .	3/4 10-7
<b><u>3/4.11 RADIOACTIVE EFFLUENTS</u></b>	
3/4.11.1 LIQUID EFFLUENTS	
Concentration . . . . .	3/4 11-1
<b>Table 4.11.1-1 Radioactive Liquid Waste Sampling and Analysis</b>	
Program . . . . .	3/4 11-2
Dose . . . . .	3/4 11-5
Liquid Radwaste Treatment System . . . . .	3/4 11-6
Liquid Holdup Tanks . . . . .	3/4 11-7
3/4.11.2 GASEOUS EFFLUENTS	
Dose Rate . . . . .	3/4 11-8

INDEX

BASES FOR SECTIONS 3.0/4.0

	<u>PAGE</u>
<u>3/4.9 REFUELING OPERATIONS</u>	
3/4.9.1 REACTOR MODE SWITCH .....	B3/4 9-1
3/4.9.2 INSTRUMENTATION .....	B3/4 9-1
3/4.9.3 CONTROL ROD POSITION .....	B3/4 9-1
3/4.9.4 DECAY TIME .....	B3/4 9-2
3/4.9.5 COMMUNICATIONS .....	B3/4 9-2
3/4.9.6 REFUELING PLATFORM .....	B3/4 9-2
3/4.9.7 CRANE TRAVEL - SPENT FUEL STORAGE POOL .....	B3/4 9-2
3/4.9.8 WATER LEVEL - REACTOR VESSEL AND WATER LEVEL - SPENT 3/4.9.9 FUEL STORAGE POOL .....	B3/4 9-2
3/4.9.10 CONTROL ROD REMOVAL .....	B3/4 9-2
3/4.9.11 RESIDUAL HEAT REMOVAL AND COOLANT CIRCULATION .....	B3/4 9-3
<u>3/4.10 SPECIAL TEST EXCEPTIONS</u>	
3/4.10.1 PRIMARY CONTAINMENT INTEGRITY .....	B3/4 10-1
3/4.10.2 ROD SEQUENCE CONTROL SYSTEM .....	B3/4 10-1
3/4.10.3 SHUTDOWN MARGIN DEMONSTRATIONS .....	B3/4 10-1
3/4.10.4 RECIRCULATION LOOPS .....	B3/4 10-1
3/4.10.5 OXYGEN CONCENTRATION .....	B3/4 10-1
3/4.10.6 TRAINING STARTUPS .....	B3/4 10-1
3/4.10.7 INSERVICE LEAK AND HYDROSTATIC TESTING .....	B3/4 10-1
<u>3/4.11 RADIOACTIVE EFFLUENTS</u>	
3/4.11.1 LIQUID EFFLUENTS	
Concentration .....	B3/4 11-1
Dose .....	B3/4 11-1
Liquid Radwaste Treatment System .....	B3/4 11-2
Liquid Holdup Tanks .....	B3/4 11-2

TABLE 1.2  
OPERATIONAL CONDITIONS

<u>CONDITION</u>	<u>MODE SWITCH POSITION</u>	<u>AVERAGE REACTOR COOLANT TEMPERATURE</u>
1. Power Operation	Run	Any temperature
2. Startup	Startup/Hot Standby	Any temperature
3. Hot Shutdown	Shutdown*,**	> 200°F
4. Cold Shutdown	Shutdown*,** †	≤ 200°F ##
5. Refueling ††	Shutdown or Refuel* #	≤ 140°F

TABLE NOTATIONS

- \* The reactor mode switch may be placed in the Run or Startup/Hot Standby position to test the switch interlock functions provided that the control rods are verified to remain fully inserted by a second licensed operator or other technically qualified member of the unit technical staff.
- \*\* The reactor mode switch may be placed in the Refuel position while a single control rod is being moved provided that the one-rod-out interlock is OPERABLE.
- † The reactor mode switch may be placed in the Refuel position while a single control rod drive is being removed from the reactor pressure vessel per Specification 3.9.10.1.
- †† Fuel in the reactor vessel with the vessel head closure bolts less than fully tensioned or with the head removed.
- # See Special Test Exceptions 3.10.1 and 3.10.3.
- ## See Special Test Exception 3.10.7.

## SPECIAL TEST EXCEPTIONS

### 3/4.10.7 INSERVICE LEAK AND HYDROSTATIC TESTING

#### LIMITING CONDITIONS FOR OPERATION

---

3.10.7 When conducting inservice leak or hydrostatic testing, the average reactor coolant temperature specified in Table 1.2 for OPERATIONAL CONDITION 4 may be increased above 200°F, and operation considered not to be in OPERATIONAL CONDITION 3, to allow performance of an inservice leak or hydrostatic test provided the maximum reactor coolant temperature does not exceed 212°F and the following OPERATIONAL CONDITION 3 LCO's are met:

- a. 3.3.2, "Isolation Actuation Instrumentation", Functions 1.a.2, 1.b, and 3.a and b of Table 3.3.2-1;
- b. 3.6.5.1, "Secondary Containment Integrity";
- c. 3.6.5.2, "Secondary Containment Automatic Isolation Dampers"; and
- d. 3.6.5.3, "Standby Gas Treatment System."

APPLICABILITY: OPERATIONAL CONDITION 4, with average reactor coolant temperature > 200°F.

#### ACTION:

With the requirements of the above specification not satisfied, immediately enter the applicable condition of the affected specification or immediately suspend activities that could increase the average reactor coolant temperature or pressure and reduce the average reactor coolant temperature to  $\leq 200^\circ\text{F}$  within 24 hours.

#### SURVEILLANCE REQUIREMENTS

---

4.10.7 Verify applicable OPERATIONAL CONDITION 3 surveillances for specifications listed in 3.10.7 are met.

## 3/4.10 SPECIAL TEST EXCEPTIONS

### BASES

---

#### 3/4.10.1 PRIMARY CONTAINMENT INTEGRITY

The requirement for PRIMARY CONTAINMENT INTEGRITY is not applicable during the period when open vessel tests are being performed during the low-power PHYSICS TESTS.

#### 3/4.10.2 ROD SEQUENCE CONTROL SYSTEM

In order to perform the tests required in the Technical Specifications it is necessary to bypass the sequence restraints on control rod movement. The additional surveillance requirements ensure that the specifications on heat generation rates and shutdown margin requirements are not exceeded during the period when these tests are being performed and that individual rod worths do not exceed the values assumed in the safety analysis.

#### 3/4.10.3 SHUTDOWN MARGIN DEMONSTRATIONS

Performance of shutdown margin demonstrations with the vessel head removed requires additional restrictions in order to ensure that criticality does not occur. These additional restrictions are specified in this Limiting Condition for Operation.

#### 3/4.10.4 RECIRCULATION LOOPS

This special test exception permits reactor criticality under no-flow conditions and is required to perform certain startup and PHYSICS TESTS while at low THERMAL POWER levels.

#### 3/4.10.5 OXYGEN CONCENTRATION

Relief from the oxygen concentration specifications is necessary in order to provide access to the primary containment during the initial startup and testing phase of operation. Without this access, the startup and test program could be restricted and delayed.

#### 3/4.10.6 TRAINING STARTUPS

This special test exception permits training startups to be performed with the reactor vessel depressurized at low THERMAL POWER and temperature while controlling RCS temperature with one RHR subsystem aligned in the shutdown cooling mode in order to minimize the discharge of contaminated water to the radioactive waste disposal system.

#### 3/4.10.7 INSERVICE LEAK AND HYDROSTATIC TESTING

This special test exception allows reactor vessel inservice leak and hydrostatic testing to be performed in OPERATIONAL CONDITION 4 with the maximum reactor coolant temperature not exceeding 212°F. The additionally imposed OPERATIONAL CONDITION 3 requirement for secondary containment operability provides conservatism in the response of the unit to an operational event. This allows flexibility since temperatures approach 190°F during the testing and can drift higher because of decay and mechanical heat. Additionally, because reactor vessel fluence increases over time, this testing will require coolant temperatures > 200°F.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 53 TO FACILITY OPERATING LICENSE NO. NPF-69  
NIAGARA MOHAWK POWER CORPORATION  
NINE MILE POINT NUCLEAR STATION, UNIT 2  
DOCKET NO. 50-410

1.0 INTRODUCTION

By letter dated May 7, 1993, as superseded September 28, 1993, Niagara Mohawk Power Corporation (the licensee) submitted a request for changes to the Nine Mile Point Nuclear Station, Unit 2 (NMP-2), Technical Specifications (TSs). The requested changes would add a new TS 3/4.10.7, "Inservice Leak and Hydrostatic Testing." The proposed changes would also include corresponding changes to the TS Index, Table 1.2, and provides Bases for TS 3/4.10.7. The proposed changes would permit NMP-2 to remain in OPERATIONAL CONDITION 4 with average reactor coolant temperature being increased above 200 °F during reactor coolant system leak or hydrostatic tests provided the maximum reactor coolant temperature does not exceed 212 °F and the following OPERATIONAL CONDITION 3 TSs are being met: (a) TS 3.3.2, "Isolation Actuation Instrumentation," Functions 1.a.2, 1.b, and 3.a and b of Table 3.3.2-1; (b) TS 3.6.5.1, "Secondary Containment Integrity;" (c) TS 3.6.5.2, "Secondary Containment Automatic Isolation Dampers;" and (d) TS 3.6.5.3, "Standby Gas Treatment System." The September 28, 1993, submittal superseded in its entirety a previously proposed amendment which was similar but would not have included the 212 °F limit.

2.0 EVALUATION

The NMP-2 TS define five OPERATIONAL CONDITIONS. OPERATIONAL CONDITION 4 requires the reactor mode switch to be in the shutdown position (reactor subcritical) and the average reactor coolant temperature to be less than or equal to 200 °F. OPERATIONAL CONDITION 3 also requires the reactor mode switch to be in the shutdown position (reactor subcritical) but with the average reactor coolant temperature greater than 200 °F.

The NMP-2 TS require that various TSs be applicable in one or more of the five OPERATIONAL CONDITIONS. Additional TSs become applicable when NMP-2 enters OPERATIONAL CONDITION 3 from OPERATIONAL CONDITION 4. This change in OPERATIONAL CONDITIONS occurs when the average reactor coolant temperature is increased above 200 °F. Two TSs of particular concern for entry into OPERATIONAL CONDITION 3 are TS 3.5.1 and TS 3.6.1.1. TS 3.5.1 requires ECCS Divisions 1, 2, and 3 to be OPERABLE in OPERATIONAL CONDITION 3 while TS 3.5.2 only requires three of five ECCS systems to be OPERABLE in OPERATIONAL CONDITIONS 4 and 5 thereby permitting outage related maintenance to be performed on the ECCS systems not required to be OPERABLE.

TS 3.6.1.1 requires PRIMARY CONTAINMENT INTEGRITY to be maintained in OPERATIONAL CONDITION 3 but PRIMARY CONTAINMENT INTEGRITY is not required in OPERATIONAL CONDITION 4. The requirements of TS 3.6.1.1 significantly restrict unobstructed access within the primary containment during operations in OPERATIONAL CONDITION 3. The licensee desires to be able to perform certain outage activities on the ECCS equipment during the performance of the reactor coolant system leak or hydrostatic tests so as to minimize the duration of outages. Unobstructed access within the primary containment is desirable to perform inspections of the reactor coolant system (RCS) during leak or hydrostatic tests of the reactor coolant system.

The RCS is isolated during leak or hydrostatic tests. This isolation makes RCS temperature control difficult since the RCS is isolated from its heat sinks and heat input to the RCS is caused by both decay heat and mechanical heat from the recirculation pumps. TS 3.4.6, "Pressure/Temperature Limits," currently requires the reactor pressure vessel temperature to be above approximately 160 °F when the RCS is pressurized for leak or hydrostatic testing. This minimum temperature for performing leak or hydrostatic tests will increase over time as fast neutron fluence to the reactor vessel increases with operating time. The leak or hydrostatic tests require several hours for completion; operating experience has shown that the RCS temperature slowly increases during these tests and dependent upon the amount of decay heat present, the RCS may approach the 200 °F limit of OPERATIONAL CONDITION 4. Therefore, NMPC has proposed to increase the OPERATIONAL CONDITION 4 temperature limit to provide some additional margin within which to complete the leak or hydrostatic tests.

Permitting the average reactor coolant temperature to be increased above 200 °F and limiting the maximum reactor coolant temperature to 212 °F while performing leak or hydrostatic tests will not substantially affect the results of potential accidents which might occur with the increased average reactor coolant temperature since the leak and hydrostatic tests are performed with the RCS near water solid and with all control rods fully inserted (reactor subcritical). Therefore, the stored energy in the reactor core would be very low and the potential for causing fuel failures with a subsequent increase in coolant activity is minimal. The restrictions provided in the proposed new TS 3.10.7 would require secondary containment integrity as well as OPERABLE automatic isolation dampers, OPERABLE standby gas treatment system, and OPERABLE automatic actuation instrumentation for this equipment. Therefore, any leakage of radioactive materials from the RCS would be filtered by the standby gas treatment system prior to release to the atmosphere. Furthermore, since the maximum reactor coolant temperature would be limited to a maximum of 212 °F, there would be no flashing of coolant to steam and therefore, any releases of radioactive materials from the coolant would be minimized.

In the event of a large loss-of-coolant accident during a leak or hydrostatic test, the RCS would rapidly depressurize thereby permitting the low pressure ECCS equipment, required OPERABLE by TS 3.5.2, to actuate and thereby keep the core flooded. This action would prevent the fuel from overheating and releasing radioactive materials. The RCS inspections required to be performed

as part of the leak or hydrostatic tests would be expected to detect small leaks before they would develop into large leaks and before a significant inventory of coolant was lost.

Based on the foregoing analyses, we conclude that the proposed TS changes will ensure acceptable consequences of any postulated accidents, are enveloped by the previously accepted analyses, and are, therefore, acceptable.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (58 FR 32386 and renoticed 58 FR 52990). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor:  
Donald S. Brinkman

Date: November 12, 1993