

December 1, 1989

Docket No. 50-220

Mr. Lawrence Burkhardt III
Executive Vice President, Nuclear Operations
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Dear Mr. Burkhardt:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. 72746)

The Commission has issued the enclosed Amendment No. 112 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station Unit No. 1 (NMP-1). The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated March 16, 1989.

This amendment revises the Technical Specifications to correct an error in Table 3.3.4 "Primary Containment Isolation Valve Lines Entering Free Space of the Containment." The change clarifies that the containment spray isolation valves do not receive automatic initiation signals to open on reactor low-low water level and high drywell pressure.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Original signed by

Robert E. Martin, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 112 to DPR-63
2. Safety Evaluation

cc w/enclosures:
See next page

OFC	:PDI-1	:PDI-1	:PDI-1	:SPLB	:OGC	:PDI-1	:
NAME	:CVogan	:D. [unclear]	:Martin	:CMcCracken	: [unclear]	: [unclear]	:
DATE	10/12/89	10/12/89	11/01/89	11/2/89	11/01/89	11/01/89	:

OFFICIAL RECORD COPY
Document Name: Amend 72746

8912140327 891201
PDR ADOCK 05000220
P PNU

R/01
11

CP-1



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

December 1, 1989

Docket No. 50-220

Mr. Lawrence Burkhardt III
Executive Vice President, Nuclear Operations
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Dear Mr. Burkhardt:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. 72746)

The Commission has issued the enclosed Amendment No. 112 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station Unit No. 1 (NMP-1). The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated March 16, 1989.

This amendment revises the Technical Specifications to correct an error in Table 3.3.4 "Primary Containment Isolation Valve Lines Entering Free Space of the Containment." The change clarifies that the containment spray isolation valves do not receive automatic initiation signals to open on reactor low-low water level and high drywell pressure.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Robert E. Martin
Robert E. Martin, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 112 to DPR-63
2. Safety Evaluation

cc: w/enclosures
See next page

Mr. L. Burkhardt III
Niagara Mohawk Power Corporation

Nine Mile Point Nuclear Station,
Unit No. 1

cc:

Mr. Troy B. Conner, Jr., Esquire
Conner & Wetterhahn
Suite 1050
1747 Pennsylvania Avenue, N. W.
Washington, D. C. 20006

Mr. Kim Dahlberg
Unit 1 Station Superintendent
Nine Mile Point Nuclear Station
Post Office Box 32
Lycoming, New York 13093

Supervisor
Town of Scriba
R. D. #4
Oswego, New York 13126

Mr. Peter E. Francisco, Licensing
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Mr. James L. Willis
General Supt.-Nuclear Generation
Niagara Mohawk Power Corporation
Nine Mile Point Nuclear Station
Post Office Box 32
Lycoming, New York 13093

Charlie Donaldson, Esquire
Assistant Attorney General
New York Department of Law
120 Broadway
New York, New York 10271

Resident Inspector
U. S. Nuclear Regulatory Commission
Post Office Box 126
Lycoming, New York 13093

Mr. Paul D. Eddy
State of New York
Department of Public Service
Power Division, System Operations
3 Empire State Plaza
Albany, New York 12223

Mr. Gary D. Wilson, Esquire
Niagara Mohawk Power Corporation
300 Erie Boulevard West
Syracuse, New York 13202

Regional Administrator, Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

Ms. Donna Ross
New York State Energy Office
2 Empire State Plaza
16th Floor
Albany, New York 12223



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 112
License No. DPR-63

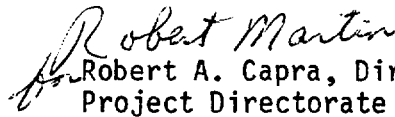
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated March 16, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.112, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 1, 1989

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 112 TO FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise Appendix A as follows:

Remove Page

148

Insert Page

148

LIMITING CONDITION FOR OPERATION

Table 3.3.4 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES
LINES ENTERING FREE SPACE OF THE CONTAINMENT

<u>Line or System</u>	<u>No. of Valves (Each Line)</u>	<u>Location Relative to Primary Containment</u>	<u>Normal Position</u>	<u>Motive Power</u>	<u>Maximum Oper. Time (Sec)</u>	<u>Action on Initiating Signal</u>	<u>Initiating Signal (All Valves Have Remote Manual Backup)</u>
<u>Core Spray (c)</u>							
<u>Pump Suction (Four Lines from Suppression Chamber)</u>	1	Outside	Open	AC Motor	90	-	Remote manual
<u>Pump Discharge (Two Test Lines to Suppression Chamber)</u>	1	Outside	Closed	AC Motor	90	Close	Reactor water level low-low
<u>LINES WITH A CLOSED LOOP INSIDE CONTAINMENT VESSELS</u>							
<u>Recir. Pump Cooling Water Supply (c)</u>							
<u>Supply Line</u>	1	Outside	Open	Self Act. Ck.	--	-	--
<u>Return Line</u>	1	Outside	Open	DC Motor	30	-	Remote manual
<u>Drywell Cooler Water Supply (c)</u>							
<u>Supply Line</u>	1	Outside	Open	Self Act. Ck.	--	-	--
<u>Return Line</u>	1	Outside	Open	DC Motor	30	-	Remote manual
<u>LINES WITH A CLOSED LOOP OUTSIDE CONTAINMENT VESSELS</u>							
<u>Containment Spray (c)</u>							
<u>Drywell & Suppression Chamber Common Supply (Four Lines)</u>	1	Outside	Open	Air/DC Sol.	60	-	Remote manual
<u>Drywell Branch (Four Lines)</u>	1	Outside	--	Self Act. Ck.	--	-	--
<u>Suppression Chamber Branch (One Branch for Each System)</u>	2*	Outside	--	Self Act. Ck.	--	-	--
<u>Pump Suction From Suppression Chamber (Four Lines)</u>	1	Outside	Open	AC Motor	70	-	Remote manual

* One valve in each separate line and one valve in each common line.
(c) These are classified as not-testable valves and penetrations.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 112 TO FACILITY OPERATING LICENSE NO. DPR-63

NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-220

INTRODUCTION

By letter dated March 16, 1989, Niagara Mohawk Power Corporation (the licensee) submitted an application to amend the Nine Mile Point Unit No. 1 Technical Specifications to correct an error in Table 3.3.4 "Primary Containment Isolation Valves Lines Entering Free Space of the Containment." The purpose of the change is to clarify that the containment spray isolation valves do not receive automatic initiation signals to open on reactor low-low water level and high drywell pressure. This issue was also discussed in an earlier letter dated September 12, 1983. The earlier letter provides additional background information and is consistent, in this regard, with the March 16, 1989 application.

DISCUSSION AND EVALUATION

Table 3.3.4 provides Limiting Conditions for Operation (LCOs) for primary containment isolation valves in lines entering free space of the containment. The current specifications indicate that the containment spray drywell and suppression chamber common supply valves receive an automatic initiation signal to open on reactor level low-low and high drywell pressure signals. The proposed change corrects Table 3.3.4 to indicate that these valves do not receive automatic signals and that they are operated by remote manual action. The error was discovered through review of piping and instrument drawings and electrical diagrams which indicate that the automatic signals are not present.

The licensee and the NRC staff have evaluated this discrepancy with respect to its potential safety consequence and have determined that there is no safety impact. The subject valves are air/DC solenoid operated valves located in the primary and secondary loop of both trains of the containment spray system. The four valves are open during normal operation as required by station operating procedures. During an event which requires containment spray operation (isolated condition), the valves are also required to be open. Thus, these valves do not have to change position between their normal and isolated condition and no automatic opening signal is required.

The valves may be closed during surveillance testing, removal of water from the suppression pool, maintenance, or suppression pool cooling subject to the LCOs in the Technical Specifications. During maintenance, the affected loop is tagged out and operability of the other loop (primary or secondary) is demonstrated immediately and daily thereafter. Following surveillance testing,

suppression pool water removal and suppression pool cooling, the governing procedures direct that the system be realigned with these valves left in the open position. These procedures, in conjunction with the TS LCOs, also provide assurance that one or more of the remaining loops remain available for containment spray, if required.

During a design basis accident, only one of the containment spray loops is required to keep containment pressure and temperature within design limits. The design basis of the containment spray system is maintained even considering a worst case single failure.

Therefore, the safety function of the containment spray system is not affected if one containment spray loop is in the test mode (valve closed) when containment spray is required. Adequate administrative control exists to ensure that the valve is reopened following testing or maintenance. Additionally, the containment spray loop in the test mode can be manually placed in operation from the control room.

In response to the licensee's request and based on the above, the staff finds the request acceptable and approves the correction of Table 3.3.4 to reflect the existing diagrams and system configuration which indicate that the containment spray drywell and suppression chamber common supply valves do not receive an automatic initiation signal to open on reactor low-low water level and high drywell pressure.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the requirements with respect to installation or use of the facility components located within the restricted areas as defined in 10 CFR 20. The staff has determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: December 1, 1989

PRINCIPAL CONTRIBUTOR:

D. Oudinot