



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

April 24, 1995

Mr. B. Ralph Sylvia
Executive Vice President, Nuclear
Niagara Mohawk Power Corporation
Nine Mile Point Nuclear Station
P.O. Box 63
Lycoming, NY 13093

SUBJECT: ISSUANCE OF AMENDMENT FOR NINE MILE POINT NUCLEAR STATION, UNIT 2
(TAC NO. M91766)

Dear Mr. Sylvia:

The Commission has issued the enclosed Amendment No. 65 to Facility Operating License No. NPF-69 for the Nine Mile Point Nuclear Station, Unit 2. This amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated March 9, 1995.

The amendment revises TS Section 4.6.1.2.a, Primary Containment/Containment Leakage. This change allows the second Type A containment leak rate test to be performed at refueling outage 5 instead of refueling outage 4, consistent with an exemption to 10 CFR Part 50, Appendix J which has been granted.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

A handwritten signature in black ink that reads "Gordon E. Edison".

Gordon E. Edison, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-410

Enclosures: 1. Amendment No. 65 to NPF-69
2. Safety Evaluation

cc w/encls: See next page

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CP

B. Ralph Sylvia
Niagara Mohawk Power Corporation

Nine Mile Point Nuclear Station
Unit 2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-410

NINE MILE POINT NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 65
License No. NPF-69

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated March 9, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2. of Facility Operating License No. NPF-69 is hereby amended to read as follows:

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2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 65, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Ledyard B. Marsh, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 24, 1995

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 65 FACILITY OPERATING LICENSE NO. NPF-69

DOCKET NO. 50-410

Revise Appendix A as follows:

Remove Page
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Insert Page
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CONTAINMENT SYSTEM

PRIMARY CONTAINMENT

PRIMARY CONTAINMENT LEAKAGE

SURVEILLANCE REQUIREMENTS

4.6.1.2 The primary containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of ANSI N45.4-1972:

- a. Three Type A overall integrated containment leakage rate tests shall be conducted at 40 ± 10 -month intervals during shutdown at Pa, 39.75 psig or at Pt, 20.0 psig, during each 10-year service period.* The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.
- b. If any periodic Type A test fails to meet 0.75 La or 0.75 Lt, as applicable, the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet 0.75 La or 0.75 Lt, as applicable, a Type A test shall be performed at least every 18 months until two consecutive Type A tests meet 0.75 La or 0.75 Lt, as applicable, at which time the above test schedule may be resumed.
- c. The accuracy of each Type A test shall be verified by a supplemental test which:
 1. Confirms the accuracy of the test by verifying that the difference between the supplemental data and the Type A test data is within 0.25 La or 0.25 Lt, as applicable.
 2. Has duration sufficient to establish accurately the change in leakage rate between the Type A test and the supplemental test.
 3. Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test to be equivalent to at least 25% of the total measured leakage at Pa, 39.75 psig, or Pt, 20.0 psig, as applicable.
- d. Type B and C tests shall be conducted with gas at Pa, 39.75 psig, at intervals no greater than 24 months except for tests involving:
 1. Air locks,
 2. Main steam line isolation valves and the remainder of the valves specified in Table 3.6.1.2-1.
 3. Containment isolation valves in hydrostatically tested lines which penetrate the primary containment, and
 4. Purge supply and exhaust isolation valves with resilient seals.

* Exemption to 10CFR50 Appendix J

The test interval for conducting the second Type A test of the first 10-year service period shall be extended to 72 months to allow the Type A test to be performed during the 5th refueling outage. This extension expires upon completion of the 5th refueling outage.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 65 TO FACILITY OPERATING LICENSE NO. NPF-69

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION, UNIT 2

DOCKET NO. 50-410

1.0 INTRODUCTION

By letter dated March 9, 1995, Niagara Mohawk Power Corporation (the licensee) submitted a request for a change to the Nine Mile Point Nuclear Station, Unit 2 Technical Specifications (TSs). The requested change would revise Section 4.6.1.2.a of the TS, Primary Containment/Containment Leakage, to reference 10 CFR Part 50, Appendix J, as modified by approved exemptions. Specifically, TS 4.6.1.2.a would be modified to allow the second Primary Containment Integrated Leakage Rate Test (Type A) to be performed at the fifth refueling outage (RF-05) or 72 months after the first Type A test instead of the fourth refueling outage (RF-04) as currently scheduled.

2.0 EVALUATION

The proposed change is administrative in nature. TS Section 4.6.1.2.a currently paraphrases 10 CFR Part 50, Appendix J, for the required frequency of the containment Integrated Leakage Rate Test (ILRT). The proposed change would revise Section 4.6.1.2.a to reference 10 CFR Part 50, Appendix J, as modified by approved exemptions. Specifically, TS 4.6.1.2.a would be modified to allow the second Primary Containment Integrated Leakage Rate Test (Type A) to be performed at the fifth refueling outage (RF-05) or 72 months after the first Type A test instead of the fourth refueling outage (RF-04) as currently scheduled. The revised wording is consistent with that used in the revised Standard TSs for the required frequency of the ILRT. Since the change is administrative in nature, in that it references the controlling regulations directly and recognizes approved exemptions, rather than paraphrasing the regulation, the NRC staff finds the proposed change to be acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

DATED: April 24, 1995

AMENDMENT NO. 65 TO FACILITY OPERATING LICENSE NO. NPF-69-NINE MILE POINT
UNIT 2

Docket File

PUBLIC

PDI-1 Reading

S. Varga, 14/E/4

J. Zwolinski, 14/H/3

L. Marsh

S. Little

J. Harold

G. Edison

OGC

D. Hagan, T-4 A43

G. Hill (2), T-5 C3

C. Grimes, 11/E/22

ACRS (4)

OPA

OC/LFDCB

PD plant-specific file

C. Cowgill, Region I

cc: Plant Service list

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (60 FR 15310). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Harold

Date: April 24, 1995

April 24, 1995

Mr. B. Ralph Sylvia
Executive Vice President, Nuclear
Niagara Mohawk Power Corporation
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A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by:

Gordon E. Edison, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-410

Enclosures: 1. Amendment No. 65 to NPF-69
2. Safety Evaluation

cc w/encs: See next page

Distribution: See attached sheet

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