Docket No. 50-220

Mr. Lawrence Burkhardt III Executive Vice President, Nuclear Operations Niagara Mohawk Power Corporation 301 Plainfield Road Syracuse, New York 13212

Dear Mr. Burkhardt:

DISTRIBUTION Docket file ACRS (10) NRC & LPDRs GPA/PA PDI-1 Rda OC/LFMB **RCapra** DHagan MNBB 3206 CVogan JLinville, R1 CMcCracken 8 D1 RMartin OGC 15 B18 EJodran MNBB 3701 P1 37 GHill JCalvo 11 F23

SUBJECT: ISSUANCE OF AMENDMENT ON HYDROGEN-OXYGEN SAMPLING SYSTEM

(TAC NO. 76662)

The Commission has issued the enclosed Amendment No. 116 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station Unit No. 1 (NMP-1). The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated March 27, 1990.

This amendment revises the Technical Specification to reflect that the normal position of the drywell and suppression chamber oxygen sampling line isolation valves is open.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely.

Original signed by

Robert E. Martin, Senior Project Manager Project Directorate I-1 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 116 to DPR-63

PDC

2. Safety Evaluation

cc: w/enclosures See next page

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Mr. L. Burkhardt III Niagara Mohawk Power Corp_ition

cc:

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Ms. Donna Ross New York State Energy Office 2 Empire State Plaza 16th Floor Albany, New York 12223 Nine Milr Point Nuclear Station, Unit No.

Mr. Kim Dahlberg Unit 1 Station Superintendent Nine Mile Point Nuclear Station Post Office Box 32 Lycoming, New York 13093

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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 116 License No. DPR-63

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated March 27, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I:
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.116, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert a. Capu

Robert A. Capra, Director Project Directorate I-1 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: May 24, 1990

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 116 TO FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise Appendix A as follows:

Remove Page

Insert Page

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LIMITING CONDITIONS FOR OPERATION

Table 3.3.4 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES LINES ENTERING FREE SPACE OF THE CONTAINMENT

Line or System	No. of Valves (Each Line)	Location Relative to Primary Containment	Normal Position	Motive Power	Maximum Oper. Time (Sec)	Action on Initiating Signal	Initiating Signal (All Valves Have Remote Manual Backup)
O ₂ Sampling							
Drywell (Three Lines)	2	Outside	Open	D.C. Sol.	60	Close (b)	Reactor Water Level Low-Low or High Drywell Pressure
Suppression Chamber (One Line)	2	Outside	Open	D.C. Sol.	60	Close (b)	

NOTES: (a) These valves may be open for containment fill with nitrogen.

(b) These valves will periodically be opened for sampling or nitrogen makeup.

Amendment No. 116



UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 116 TO FACILITY OPERATING LICENSE NO. DPR-63

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-220

INTRODUCTION

The Niagara Mohawk Power Corporation (NMPC) submitted an application dated March 27, 1990 to amend the Technical Specifications (TS) for the Nine Mile Point Nuclear Station, Unit 1 (NMP-1). The change would revise TS Table 3.3.4, "Primary Containment Isolation Valves Entering Free Space of the Containment," to reflect that the normal position of the isolation valves in the drywell and suppression chamber oxygen sampling line is "open". The licensee states that the prior listing of these isolation valves as normally closed was an administrative error in the TS.

EVALUATION

As discussed in Section VII of the FSAR, the hydrogen-oxygen sampling systems continuously monitor the hydrogen and oxygen concentrations within the drywell and suppression chamber. A continuous indication of concentrations is provided in the control room. As stated in the TS and its BASES the oxygen concentration is determined at least once per week to ensure that TS 3.3.1 on allowable concentration is met. As stated in the application, the hydrogen-oxygen sampling systems are an integral part of the containment atmospheric dilution (CAD) system. The CAD system, in conjunction with the containment inerting system is designed to prevent a combustible hydrogen-oxygen concentration from accumulating in the primary containment atmosphere immediately following or during a loss-of-coolant accident. Therefore, as discussed above, the hydrogen-oxygen sampling system is utilized during normal operation to ensure compliance with the TS limits and in response to potential accident conditions. In the event of conditions indicative of a loss-of-coolant accident (LOCA) (high drywell pressure or low-low reactor vessel water level) the isolation valves will be automatically actuated to close. The valves can be reopened to permit monitoring of the containment during LOCA conditions.

The operating characteristics of the system are such that an initial startup period would be required to initiate and to stabilize the system for the monitoring of oxygen concentrations if the system were normally maintained inoperable with the isolation valves closed. Therefore, to facilitate a more practical surveillance monitoring to meet the weekly TS surveillance requirement and to provide continuous indication in the control room, the system remains continuously operable while the plant is in operation with its isolation valve open to permit sampling of the containment atmosphere.

The staff has reviewed the licensee's application and on the basis discussed above finds the change in the normal valve position from "open" to "closed" to be acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in a requirement with respect to the installation or use of the facility components located within the restricted areas as defined in 10 CFR 20. The staff has determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: May 24, 1990

PRINCIPAL CONTRIBUTOR:

Robert E. Martin