UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

July 12, 1993

Docket No. 50-410

Mr. B. Ralph Sylvia
Executive Vice President, Nuclear
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Dear Mr. Sylvia:

SUBJECT: ISSUANCE OF AMENDMENT FOR NINE MILE POINT NUCLEAR STATION, UNIT 2 (TAC NO. M86448)

The Commission has issued the enclosed Amendment No. 43 to Facility Operating License No. NPF-69 for the Nine Mile Point Nuclear Station, Unit 2. The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated May 18, 1993.

The amendment revises TS 4.6.1.2.a to provide a one-time extension of the required test interval for overall integrated containment leak rate tests (Type A tests). This extension will allow the second Type A test of the first 10-year service period to be performed during the fourth refueling outage and expires upon completion of that refueling outage.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

John E. Menning, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 43 to NPF-69
2. Safety Evaluation

cc w/enclosures:
See next page

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Mr. B. Ralph Sylvia
Niagara Mohawk Power Corporation

Nine Mile Point Nuclear Station
Unit 2

cc:

Mark J. Wetterhahn, Esquire
Winston & Strawn
1400 L Street, NW.
Washington, DC 20005-3502

Mr. Richard Goldsmith
Syracuse University
College of Law
E. I. White Hall Campus
Syracuse, New York 12223

Resident Inspector
Nine Mile Point Nuclear Station
P. O. Box 126
Lycoming, New York 13093

Gary D. Wilson, Esquire
Niagara Mohawk Power Corporation
300 Erie Boulevard West
Syracuse, New York 13202

Mr. David K. Greene
Manager Licensing
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Ms. Donna Ross
New York State Energy Office
2 Empire State Plaza
16th Floor
Albany, New York 12223

Supervisor
Town of Scriba
Route 8, Box 382
Oswego, New York 13126

Regional Administrator, Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Charles Donaldson, Esquire
Assistant Attorney General
New York Department of Law
120 Broadway
New York, New York 10271

Mr. Richard M. Kessel
Chair and Executive Director
State Consumer Protection Board
99 Washington Avenue
Albany, New York 12210

Mr. Martin J. McCormick Jr.
Plant Manager, Unit 2
Nine Mile Point Nuclear Station
Niagara Mohawk Power Corporation
P. O. Box 32
Lycoming, New York 13093

Vice President - Nuclear Generation
Nine Mile Point Nuclear Station
Niagara Mohawk Power Corporation
P. O. Box 32
Lycoming, New York 13093

DATED: July 12, 1993

AMENDMENT NO. 43 TO FACILITY OPERATING LICENSE NO. NPF-69-NINE MILE POINT UNIT 2

Docket File
NRC & Local PDRs
PDI-1 Reading
S. Varga, 14/E/4
J. Calvo, 14/A/4
R. Capra
C. Vogan
J. Menning
OGC-WF
D. Hagan, 3302 MNBB
G. Hill (2), P1-22
Wanda Jones, P-370
C. Grimes, 11/F/23
ACRS (10)
OPA
OC/LFMB
PD plant-specific file
C. Cowgill, Region I
R. Barrett, 8/H/7

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-410

NINE MILE POINT NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 43
License No. NPF-69

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated May 18, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-69 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 43 are hereby incorporated into this license. Niagara Mohawk Power Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Capra

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 12, 1993

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 43 TO FACILITY OPERATING LICENSE NO. NPF-69

DOCKET NO. 50-410

Revise Appendix A as follows:

Remove Page	Insert Page
3/4 6-4	3/4 6-4

SURVEILLANCE REQUIREMENTS

4.6.1.2 The primary containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of ANSI N45.4-1972:

a. Three Type A overall integrated containment leakage rate tests shall be conducted at 40 ± 10-month intervals during shutdown at Pa, 39.75 psig or at Pt, 20.0 psig, during each 10-year service period.* The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.

b. If any periodic Type A test fails to meet 0.75 La or 0.75 Lt, as applicable, the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet 0.75 La or 0.75 Lt, as applicable, a Type A test shall be performed at least every 18 months until two consecutive Type A tests meet 0.75 La or 0.75 Lt, as applicable, at which time the above test schedule may be resumed.

c. The accuracy of each Type A test shall be verified by a supplemental test which:

   1. Confirms the accuracy of the test by verifying that the difference between the supplemental data and the Type A test data is within 0.25 La or 0.25 Lt, as applicable.

   2. Has duration sufficient to establish accurately the change in leakage rate between the Type A test and the supplemental test.

   3. Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test to be equivalent to at least 25% of the total measured leakage at Pa, 39.75 psig, or Pt, 20.0 psig, as applicable.

d. Type B and C tests shall be conducted with gas at Pa, 39.75 psig, at intervals no greater than 24 months except for tests involving:

   1. Air locks

   2. Main steam line isolation valves and the remainder of the valves specified in Table 3.6.1.2-1.

   3. Containment isolation valves in hydrostatically tested lines which penetrate the primary containment, and

   4. Purge supply and exhaust isolation valves with resilient seals.

* The test interval for conducting the second Type A test of the first 10-year service period shall be extended to 54 months to allow the Type A test to be performed during the 4th refueling outage. This extension expires upon completion of the 4th refueling outage.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 43 TO FACILITY OPERATING LICENSE NO. NPF-69

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION, UNIT 2

DOCKET NO. 50-410

1.0 INTRODUCTION

By letter dated May 18, 1993, Niagara Mohawk Power Corporation (the licensee) submitted a request for changes to the Nine Mile Point Nuclear Station, Unit 2 (NMP-2), Technical Specifications (TSs). The requested changes would revise TS 4.6.1.2.a to provide a one-time extension of the required test interval for overall integrated containment leak rate tests (Type A tests)..

TS 4.6.1.2.a requires that three Type A tests be conducted at 40 plus or minus 10-month intervals during each 10-year service period, and that the third test of each set of three be conducted during the shutdown for the 10-year plant inservice inspection. Part 50 of Title 10 of the *Code of Federal Regulations* (CFR), Appendix J, Section III.D.1, requires that a set of three approximately equally spaced Type A tests be performed during each 10-year service period, and that the third test of each set be conducted during the plant shutdown for the 10-year plant inservice inspection. The licensee indicated in its submittal of May 18, 1993, that the first Type A test of the first 10-year service period was conducted on January 14, 1991, during the first refueling outage. The next (third) refueling outage is scheduled to begin in October 1993. If the second Type A test is performed during the third refueling outage to comply with the 40 plus or minus 10 months requirement, a third Type A test would be required during the fifth refueling outage in 1996 or 1997. In addition, a fourth Type A test would be required during the sixth refueling outage in 1998 or 1999 to coincide with the end of the 10-year inservice inspection period. This would result in four tests being performed in the first 10-year inservice period, instead of the three Type A tests required by the TSs and 10 CFR Part 50, Appendix J.

The licensee stated in the submittal of May 18, 1993, that the need to conduct four Type A tests during the first 10-year inservice period resulted from a short second fuel cycle (12 months). The licensee has proposed that the maximum allowed test interval of 50 months be extended by 4 months on a one-time basis to permit performance of the second Type A test during the fourth refueling outage that is scheduled to start in May or June 1995. The licensee stated that the proposed changes would provide for three approximately equally spaced Type A tests within the first 10-year inservice period, consistent with 10 CFR Part 50, Appendix J. The licensee has proposed that the one-time extension of the test interval be incorporated into the TSs

by adding a footnote to TS 4.6.1.2.a. The extension would expire upon completion of the fourth refueling outage. Since the TS change proposed by the licensee is consistent with the requirements of 10 CFR Part 50, Appendix J, no exemption from the requirements of the regulations would be needed.

2.0 EVALUATION

The licensee has proposed that the following be added to the TSs as a footnote to TS 4.6.1.2.a:

> "The test interval for conducting the second Type A test of the first 10-year service period shall be extended to 54 months to allow the Type A test to be performed during the 4th refueling outage. This extension expires upon completion of the 4th refueling outage."

The intent of the established test interval is that three approximately equally spaced Type A tests be conducted within a given 10-year inservice period. The proposed extension remains consistent with that intent. The alternative of conducting the second periodic Type A test during the third refueling outage in order to meet the 40 plus or minus 10 months requirement would necessitate conducting four tests during the first 10-year inservice period. This is clearly contrary to the intent of the regulations in 10 CFR Part 50, Appendix J, to have equally spaced Type A tests. The licensee has estimated that performance of an additional test would add 4 days to the outage schedule with associated costs and 2.25 man-rem of exposure to test personnel.

The licensee has stated that the results of previous Type A tests indicate that an extension of the maximum test interval by 4 months would not jeopardize the ability of the containment to maintain the leakage rate at or below the applicable limit. The allowable leakage rate stated in TS 3.6.1.2.a is 1.1 percent by weight of the containment air per day when testing is performed at a pressure of 39.75 psig. The preoperational Type A test performed on April 14, 1986, resulted in an integrated leakage rate of 0.6585 percent per day. The Type A test performed on January 14, 1991, resulted in an integrated leakage rate of 0.623 percent per day. The results of these two previous Type A tests showed that the actual average integrated leakage of primary containment was less than 58 percent of the TS limit. In addition, the results of the first inservice Type A test showed that the ability of the containment to maintain leakage at or below the TS limits had not degraded during the previous 57 months. Actual testing at NMP-2 has shown that approximately two thirds of the total primary containment leakage is from penetrations that receive Type B and C local leak rate tests. These Type B and C tests will continue to be performed at the frequency required by the TSs with repairs being performed as necessary. The demonstrated operability of these penetrations will provide added assurance that overall containment leakage remains satisfactory.

The licensee also indicated that since the last Type A test there have been no permanent modifications to the containment structure, liner, or penetrations nor other temporary alterations that would adversely affect Type A test results. No such modifications to the containment boundary are planned prior to the fourth refueling outage when the next Type A test would be conducted under the proposed TS change. Any unplanned modifications to the containment prior to the next scheduled Type A test would be subject to the special testing requirements of 10 CFR Part 50, Appendix J, Section IV.A. The licensee also stated that there have been no pressure or temperature excursions in the containment that could have adversely affected containment integrity.

Based on the past Type A test results and the absence of modifications to the containment and its penetrations, the staff finds that the proposed amendment for a one-time extension of the required test interval for Type A tests would not adversely affect plant safety and is, therefore, acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (58 FR 32387). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor:
John E. Menning

Date: July 12, 1993

July 12, 1993

Docket No. 50-410

Mr. B. Ralph Sylvia
Executive Vice President, Nuclear
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

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A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by:

John E. Menning, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 43 to NPF-69
2. Safety Evaluation

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