

October 11, 1990

Docket No. 50-410

Mr. Lawrence Burkhardt III
Executive Vice President, Nuclear Operations
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Dear Mr. Burkhardt:

Distribution:

Docket File	EJordan
NRC/Local PDRs	GHill(4)
PDI-1 Rdg	Wanda Jones
SVarga	JCalvo
BBoger	ACRS(10)
CVogan	GPA/PA
RACapra	OC/LFMB
DOudinot	REMartin
OGC	DHagan
JLinville	Plant File
CMcCracken	JHarold

SUBJECT: ISSUANCE OF AMENDMENT ON PRIMARY CONTAINMENT ISOLATION VALVES
LEAK RATE TESTING FOR NINE MILE POINT UNIT 2 (TAC NO. 77197)

The Commission has issued the enclosed Amendment No. 22 to Facility Operating License No. NPF-69 for the Nine Mile Point Nuclear Station Unit No. 2 (NMP-2). The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated July 19, 1990.

This amendment revises Table 3.6.3-1, "Primary Containment Isolation Valves," to permit hydrostatic leak rate testing of the Emergency Core Cooling System and Reactor Core Isolation Cooling System Suppression Pool Isolation Valves in lieu of the leak rate air testing.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Robert E. Martin, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 22 to NPF-69
2. Safety Evaluation

cc: w/enclosures

See next page

PDI-1:LA
CVogan
10-9-90

DOudinot
PDI/1:PE
DOudinot:rsc
10/09/90

REMartin
PDI/D:PM
REMartin
10/9/90

RACapra
PDI-1:D
RACapra
10/11/90

OGC
OGC 816
DHagan
10/11/90

DOCUMENT NAME: AMENDMENT 77197

9010240076 901011
PDR ADDCK 05000410
PDC

JFO
11
C/P
11



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

October 11, 1990

Docket No. 50-410

Mr. Lawrence Burkhardt III
Executive Vice President, Nuclear Operations
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Dear Mr. Burkhardt:

SUBJECT: ISSUANCE OF AMENDMENT ON PRIMARY CONTAINMENT ISOLATION VALVES
LEAK RATE TESTING FOR NINE MILE POINT UNIT 2 (TAC NO. 77197)

The Commission has issued the enclosed Amendment No. 22 to Facility Operating License No. NPF-69 for the Nine Mile Point Nuclear Station Unit No. 2 (NMP-2). The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated July 19, 1990.

This amendment revises Table 3.6.3-1, "Primary Containment Isolation Valves," to permit hydrostatic leak rate testing of the Emergency Core Cooling System and Reactor Core Isolation Cooling System Suppression Pool Isolation Valves in lieu of the leak rate air testing.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Robert E. Martin".

Robert E. Martin, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 22 to NPF-69
2. Safety Evaluation

cc: w/enclosures
See next page

Mr. Lawrence Burkhardt III
Niagara Mohawk Power Corporation

Nine Mile Point Nuclear Station
Unit 2

cc:

Mr. Mark J. Wetterhahn, Esquire
Bishop, Cook, Purcell & Reynolds
1400 L. Street, N.W.
Washington, D. C. 20005-3502

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

Mr. Richard Goldsmith
Syracuse University
College of Law
E. I. White Hall Campus
Syracuse, New York 12223

Charlie Donaldson, Esquire
Assistant Attorney General
New York Department of Law
120 Broadway
New York, New York 10271

Resident Inspector
Nine Mile Point Nuclear Power Station
P. O. Box 99
Lycoming, New York 13093

Mr. Richard M. Kessel
Chair and Executive Director
State Consumer Protection Board
99 Washington Avenue
Albany, New York 12210

Mr. Gary D. Wilson, Esquire
Niagara Mohawk Power Corporation
300 Erie Boulevard West
Syracuse, New York 13202

Mr. Richard Abbott, Unit 2 Station
Superintendent
Nine Mile Point Nuclear Station
Niagara Mohawk Power Corporation
P. O. Box 32
Lycoming, NY 13093

Mr. Peter E. Francisco, Licensing
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Mr. Joseph F. Firlit, General Supt.,
Nuclear Generation
Nine Mile Point Nuclear Station
Niagara Mohawk Power Corporation
P. O. Box 32
Lycoming, New York 13093

Ms. Donna Ross
New York State Energy Office
2 Empire State Plaza
16th Floor
Albany, New York 12223

Supervisor
Town of Scriba
R. D. #4
Oswego, New York 13126



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-410

NINE MILE POINT NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 22
License No. NPF-69

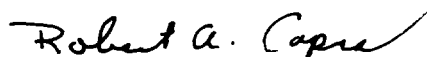
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated July 19, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-69 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 22 are hereby incorporated into this license. Niagara Mohawk Power Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 11, 1990



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 22 TO FACILITY OPERATING LICENSE NO. NPF-69

DOCKET NO. 50-410

Revise Appendix A as follows:

Remove Page

3/4 6-27
3/4 6-35

Insert Page

3/4 6-27
3/4 6-35

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

ISOLATION VALVE NO.	VALVE FUNCTION	VALVE GROUP	ISOLATION SIGNAL(a)	MAXIMUM CLOSING TIME (SECONDS)
2RHS*MOV142(j)(m)	RHS Drain to Radwaste Outside IV	4	A,Z,F,RM	30
2RHS*MOV149(j)(m)	RHS Drain to Radwaste Inside IV	4	A,Z,F,RM	25
2RHS*SOV35 A/B (j)(m)	RHS Sample HX Inside IVs	4	A,Z,F,RM	5
2RHS*SOV36 A/B (j)(m)	RHS Sample HX Outside IVs	4	A,Z,F,RM	5
2RDS*AOV124(k)	SCRAM Discharge Volume Vent	NA		NA
2RDS*AOV132(k)	SCRAM Discharge Volume Vent	NA		NA
2RDS*AOV123(k)	SCRAM Discharge Volume Drain	NA		NA
2RDS*AOV130(k)	SCRAM Discharge Volume Drain	NA		NA
B. Remote Manual				
2RHS*MOV15 A,B	Containment Spray to Drywell Outside IV's	12	RM	NA
2RHS*MOV 1 A,B,C(o)	RHS Pump Suction Outside IVs	12	RM	NA
2RHS*MOV30 A,B	RHS Test Line to SP Outside IVs	12	RM	NA
2RHS*MOV25 A,B (n)	Containment Spray to Drywell Outside IVs	12	RM	NA
2RHS*MOV24 A,B,C	RHS/LPCI to RPV Outside IVs	12	RM	NA
2CSH*MOV118(n)(o)	CSH Suction from SP Outside IV	12	RM	NA
2CSH*MOV105	HPCS Min Flow Bypass Outside IV	12	RM	NA
2CSH*MOV107	CSH to RPV Outside IV	12	RM	NA
2CSL*MOV112(o)	CSL Suction from SP Outside IV	12	RM	NA
2CSL*MOV104	CSL to RPV Outside IV	12	RM	NA
2ICS*MOV136(n)(o)	ICS Suction from SP Outside IV	12	RM	NA
2ICS*MOV143(n)	ICS Min flow to SP Outside IV	12	RM	NA

NINE MILE POINT - UNIT 2

3/4 6-27

TABLE 3.6.3-1 (Continued)
PRIMARY CONTAINMENT ISOLATION VALVES
TABLE NOTATION

- * Isolates on injection signal, not primary containment isolation signal.
- (a) See Specification 3.3.2, Table 3.3.2-4, for valve groups operated by isolation signal(s).
- (b) Deleted.
- (c) These valves are the RHR heat exchangers vent lines isolation valves. The vent line connects to the RHR safety relief valves (SRVs) Discharge Header before it penetrates the primary containment. The position indicators for these valves are provided in the Control Room for remote manual isolation.
- (d) Type C leakage tests not required.
- (e) The associated instrument lines shall not be isolated during Type A testing. Type C testing is not required. These valves shall be tested in accordance with Surveillance Requirement 4.6.3.4.
- (f) These valves are check valves, located on the vacuum breaker lines for RHR SRVs discharge headers. The SRV discharge header terminates under pool water and therefore has no containment isolation valves other than those on lines feeding into it.
- (g) 2SLS*MOV5A and B are globe stop check valves. These valves close upon reverse flow. The motor operator is provided to remote manually close the valve from the control room.
- (h) These valves are testable check valves. They close upon reverse flow. The air operator on each valve is provided only for periodic testing of the valve. These valves can only be tested against a zero d/p.
- (i) Valves are maintained closed. The FPW lines are capped. Valves are Type C tested.
- (j) Not primary containment penetration isolation valves. These valves close on an isolation signal to provide integrity of "A" and "B" LPCI loops.
- (k) Valves close on a SCRAM signal; not part of primary containment isolation system but are included here for Type C testing per Specification 3.6.1.2. These valves are not required to be OPERABLE per this specification but are required to be OPERABLE per Specification 3.1.3.1.
- (l) Not subject to Type A or Type C leak test because of constant monitoring under constant 1800 psig pressure and the possible detrimental effects of shutdown.
- (m) Not subject to Type C test per 10CFR50, Appendix J. A hydrostatic test is performed in accordance with Specification 4.6.1.2.d.3.
- (n) These valves are Type C tested and may be tested in the reverse direction.
- (o) Isolation barrier remains waterfilled post-LOCA. Isolation valve is tested with water in accordance with Specification 4.6.1.2.i.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 22 TO FACILITY OPERATING LICENSE NO. NPF-69
NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT NUCLEAR POWER STATION, UNIT NO. 2
DOCKET NO. 50-410

1.0 INTRODUCTION

By letter dated July 19, 1990, Niagara Mohawk Power Corporation, the licensee for Nine Mile Point Nuclear Station, Unit 2, proposed changes to the plant Technical Specification (TS) Table 3.6.3-1, "Primary Containment Isolation Valves." The proposed change to the table would allow hydrostatic leak rate testing for the emergency core cooling system (ECCS) and reactor core isolation cooling (RCIC) system isolation valves in lines that terminate below the suppression pool.

2.0 EVALUATION

Niagara Mohawk requested a change to the test method for the isolation valves associated with the following systems:

<u>System</u>	<u>Valve No.</u>
Residual Heat Removal Pump Suction	MOV1 A, B, C,
High Pressure Core Injection System	MOV 118
Low Pressure Core Injection System	MOV 112
Reactor Core Isolation Cooling System	MOV 136

These valves are presently Type C tested using air, and Niagara Mohawk has proposed to revise TS Table 3.6.3-1 to permit hydrostatic leak testing of the above listed ECCS and RCIC System suppression pool isolation valves in lieu of the air test, on the basis that these lines terminate below the minimum torus water level and total valve leakage will be limited to 1 gpm times the total number of the above containment isolation valves.

The Acceptance Criteria of NUREG-0800, Standard Review Plan, Section 6.2.6, "Containment Leakage Testing," states that "Hydrostatic testing of containment isolation valves is permissible if the line is not a potential containment atmosphere leak path, and may be found acceptable if it can be demonstrated

in accordance with the requirements of Section III.C of Appendix J, that a liquid inventory is available to maintain a water seal (while assuming the single failure of any active component) during the post accident period. Limits for liquid leakage should be assigned to these valves based on analysis and included in the plant Technical Specifications." The ECCS and RCIC suction lines terminate below the calculated minimum post accident suppression pool water level. Therefore, suppression pool water effectively seals the above containment isolation valves from the primary containment atmosphere thereby preventing gaseous releases to the primary containment during post-accident periods. Niagara Mohawk has committed to hydrostatically leak test these valves to at least 1.10 Pa (43.73 psig). The combined leakage value will be limited to less than or equal to 1 gpm times the total number of such containment isolation valves (as specified in the Technical Specifications). Leakage results from the hydrostatically tested valves will be excluded from the combined Types B and C leak rate calculations as allowed by Appendix J and the Technical Specifications. The proposed test pressure and the specification of leakage limit for the above containment isolation valves complies with 10 CFR Part 50, Appendix J, Item III.C.3(a) and (b).

The staff has reviewed the proposed changes to the Technical Specifications and finds these changes to be acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in a requirement with respect to the installation or use of the facility components located within the restricted areas as defined in 10 CFR Part 20. The staff has determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: October 11, 1990

PRINCIPAL CONTRIBUTOR:
J. Harold