



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

May 10, 1993

Docket No. 50-410

Mr. B. Ralph Sylvia
Executive Vice President, Nuclear
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Dear Mr. Sylvia:

SUBJECT: ISSUANCE OF AMENDMENT FOR NINE MILE POINT NUCLEAR STATION,
UNIT 2 (TAC NO. M85888)

The Commission has issued the enclosed Amendment No.40 to Facility Operating License No. NPF-69 for the Nine Mile Point Nuclear Station, Unit 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated February 12, 1993.

The amendment revises Technical Specifications (TS) Table 1.2, "Operational Conditions," and Section 3/4.9.1, "Reactor Mode Switch," to permit movement of a single control rod with the reactor in the Hot Shutdown or Cold Shutdown Conditions for post-maintenance and surveillance testing on control rod drives. The TS had previously permitted movement of a control rod in these operational conditions to recouple a control rod to its drive.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

John E. Menning, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 40 to NPF-69
2. Safety Evaluation

cc w/enclosures:
See next page

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Mr. B. Ralph Sylvia
Niagara Mohawk Power Corporation

Nine Mile Point Nuclear Station
Unit No. 2

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DATED: May 10, 1993

AMENDMENT NO. 40 TO FACILITY OPERATING LICENSE NO. NPF-69-NINE MILE POINT
UNIT 2

Docket File

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G. Hill (2), P1-22

Wanda Jones, P-370

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cc: Plant Service list



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-410

NINE MILE POINT NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 40
License No. NPF-69

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated February 12, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-69 is hereby amended to read as follows:

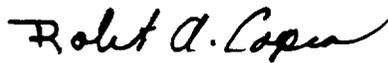
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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 40 are hereby incorporated into this license. Niagara Mohawk Power Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 10, 1993

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 40 TO FACILITY OPERATING LICENSE NO. NPF-69

DOCKET NO. 50-410

Revise Appendix A as follows:

Remove Pages

1-11
3/4 9-1

Insert Pages

1-11
3/4 9-1

TABLE 1.2

OPERATIONAL CONDITIONS

<u>CONDITION</u>	<u>MODE SWITCH POSITION</u>	<u>AVERAGE REACTOR COOLANT TEMPERATURE</u>
1. Power Operation	Run	Any temperature
2. Startup	Startup/Hot Standby	Any temperature
3. Hot Shutdown	Shutdown*,**	> 200°F
4. Cold Shutdown	Shutdown*,** †	≤ 200°F
5. Refueling † †	Shutdown or Refuel* #	≤ 140°F

TABLE NOTATIONS

- * The reactor mode switch may be placed in the Run or Startup/Hot Standby position to test the switch interlock functions provided that the control rods are verified to remain fully inserted by a second licensed operator or other technically qualified member of the unit technical staff.
- ** The reactor mode switch may be placed in the Refuel position while a single control rod is being moved provided that the one-rod-out interlock is OPERABLE.
- † The reactor mode switch may be placed in the Refuel position while a single control rod drive is being removed from the reactor pressure vessel per Specification 3.9.10.1
- † † Fuel in the reactor vessel with the vessel head closure bolts less than fully tensioned or with the head removed.
- # See Special Test Exceptions 3.10.1 and 3.10.3.

3/4.9 REFUELING OPERATIONS

3/4.9.1 REACTOR MODE SWITCH

LIMITING CONDITIONS FOR OPERATION

3.9.1 The reactor mode switch shall be OPERABLE and locked in the Shutdown or Refuel position. When the reactor mode switch is locked in the Refuel position:

- a. A control rod shall not be withdrawn unless the Refuel position one-rod-out interlock is OPERABLE.
- b. CORE ALTERATIONS shall not be performed using equipment associated with a Refuel position interlock unless at least the following associated Refuel position interlocks are OPERABLE for such equipment.
 1. All rods in.
 2. Refuel platform position.
 3. Refuel platform hoists fuel-loaded.
 4. Fuel grapple position.
 5. Service platform hoist fuel-loaded.

APPLICABILITY: OPERATIONAL CONDITION 5* #, OPERATIONAL CONDITIONS 3 and 4 when the reactor mode switch is in the Refuel position.

ACTION:

- a. With the reactor mode switch not locked in the Shutdown or Refuel position as specified, suspend CORE ALTERATIONS and lock the reactor mode switch in the Shutdown or Refuel position.
- b. With the one-rod-out interlock inoperable, lock the reactor mode switch in the Shutdown position.
- c. With any of the above required Refuel position equipment interlocks inoperable, suspend CORE ALTERATIONS with equipment associated with the inoperable Refuel position equipment interlock.

* See Special Test Exceptions 3.10.1 and 3.10.3.

The reactor shall be maintained in OPERATIONAL CONDITION 5 whenever fuel is in the reactor vessel with the vessel head closure bolts less than fully tensioned or with the head removed.



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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 40 TO FACILITY OPERATING LICENSE NO. NPF-69
NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT NUCLEAR STATION, UNIT 2
DOCKET NO. 50-410

1.0 INTRODUCTION

By letter dated February 12, 1993, Niagara Mohawk Power Corporation (NMPC or the licensee) submitted a request for changes to the Nine Mile Point Nuclear Station, Unit 2, (NMP-2) Technical Specifications (TS). The requested changes would revise a footnote in TS Table 1.2, "Operational Conditions," and the Applicability statement of TS 3/4.9.1, "Reactor Mode Switch."

The TS currently permit withdrawal of a single control rod in the Hot or Cold Shutdown Conditions for the purpose of recoupling a control rod to its drive. This is done by placing the mode switch in the Refuel position, provided the one-rod-out interlock (which limits withdrawal to one rod) is operable. Permission for this withdrawal for recoupling is provided in a footnote to the Conditions 3 and 4 mode switch position requirement statements in TS Table 1.2. The licensee has proposed that the word "recoupled" in this footnote be replaced with "moved." This change would provide permission for the movement of a single control rod in those operational conditions for purposes other than recoupling, e.g., for post-scrum venting, friction testing, or scram time testing.

There is currently no TS required surveillance related to the rod withdrawal for recoupling permitted for Conditions 3 and 4 in Table 1.2. The licensee proposes to augment the Applicability statement of TS 3/4.9.1 to include "Operational Conditions 3 and 4 when the reactor mode switch is in the Refuel position." This change would extend the applicability of the appropriate testing requirements for the one-rod-out interlock to Operational Conditions 3 and 4 when the reactor mode switch is in the Refuel position.

2.0 EVALUATION

The NRC staff has evaluated NMPC's submittal dated February 12, 1993, considering the factors that are discussed below.

The proposed change to TS Table 1.2 is similar to existing approved TS Table 1.2 specifications in other BWR reactors (e.g., Clinton, Grand Gulf, LaSalle, Perry, and River Bend). These were either in the initial TS or the result of approved changes similar to those proposed by NMPC.

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Since control rod movement is blocked when the mode switch is in the Shutdown position, movement of the switch to Refuel (or to Startup or Run) is necessary to move a rod for recoupling or any other purpose. When the mode switch is in the Refuel position, the redundant logic of the one-rod-out interlock limits rod movement to one rod. Because of the required shutdown margin with one control rod fully withdrawn, there is reasonable assurance that the reactor will remain subcritical with the mode switch in the Refuel position.

The proposed change to TS Table 1.2 does not change the current permission to withdraw a single control rod in Operational Conditions 3 and 4, but it does expand the permitted testing and maintenance activities for withdrawal. While this will increase the frequency of single control rod withdrawals in Operational Conditions 3 and 4, the probability of withdrawal events is not affected since these events would occur in Operational Conditions 1, 2, or 5.

Maintenance and testing on control rod drives are currently allowed for all BWRs in Operational Conditions 1 and 2 (Startup and Power Operation, respectively), where these activities are not under the control of the one-rod-out interlock, as well as in Operational Condition 5 (Refueling).

The proposed change to TS 3/4.9.1 provides appropriate surveillance of the one-rod-out interlock in Operational Conditions 3 and 4, as it currently does for Operational Condition 5.

The factors discussed above indicate that the proposed change to TS Table 1.2 is consistent with previous NRC staff approvals and existing TS for other BWR reactors, provides for needed maintenance and testing of rods, is not significantly different from currently permitted rod withdrawal operations, and does not increase the probability of a rod withdrawal event. The proposed change to TS 3/4.9.1 provides additional and appropriate surveillance requirements for rod withdrawal in Operational Conditions 3 and 4 not currently required for permitted withdrawals for control rod recoupling. Therefore, the staff concludes that the proposed changes to the NMP-2 TS are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a

proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (58 FR 16866). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor:
John E. Menning

Date: May 10, 1993

Docket No. 50-410

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Executive Vice President, Nuclear
Niagara Mohawk Power Corporation
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Sincerely,
Original Signed By:
John E. Menning, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

- Enclosures:
1. Amendment No. 40 to NPF-69
2. Safety Evaluation

cc w/enclosures:
See next page

OFFICE	PDI-1:LA	PDI-1:PM	SRXB	OGC/ASB	PDI-1:D
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DATE	4/20/93 <i>ai</i>	4/21/93	4/26/93	4/29/93	5/10/93

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FILENAME: G:\NMP2\NM285888.AMD