

January 26, 1989

Docket No. 50-220

DISTRIBUTION

Mr. Lawrence Burkhardt III  
Executive Vice President, Nuclear Operations  
Niagara Mohawk Power Corporation  
301 Plainfield Road  
Syracuse, New York 13212

Docket file NRC PDR  
Local PDR PDI-1 Rdg  
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RCapra MHaughey  
RBenedict OGC  
DHagan EJordan  
BGrimes TBarnhart (4)  
WJones EButcher  
ACRS (10) GPA/PA  
ARM/LFMB JJohnson, RI  
WHodges MJohnson, EDO

Dear Mr. Burkhardt:

The Commission has issued the enclosed Amendment No. 103 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station Unit 1 (NMP-1). The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated January 14, 1988 (TAC 67175).

This amendment revises the Technical Specifications to eliminate a contradiction between Specification 3.1.1.b(3)(b) and Specification 3.1.1.e and to require verification in Specification 3.1.1.b(3)(b) that the control rod program is being followed appropriately.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Joseph D. Neighbors for  
Mary F. Haughey, Project Manager  
Project Directorate I-1  
Division of Reactor Projects, I/II

Enclosures:

- 1. Amendment No. 103 to DPR-63
- 2. Safety Evaluation

cc: w/enclosures  
See next page

\*SEE PREVIOUS CONCURRENCE

OFC	:PDI-1	:PDI-1	:PDI-1	:SRXB	:PDI-1	:OGC	:
NAME	:CVogan	:RBenedict	:vr:MHaughey	:WHodges	:RCapra	:MYoung	:
DATE	:11/17/88*	:12/2/88	:11/15/88*	:11/17/88*	:12/5/88	:11/29/88*	:

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C/P-1

Docket No. 50-220

Mr. Charles V. Mangan  
Senior Vice President  
Niagara Mohawk Power Corporation  
301 Plainfield Road  
Syracuse, New York 13212

Dear Mr. Mangan:

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Mary F. Haughey, Project Manager  
Project Directorate I-1  
Division of Reactor Projects, I/II

Enclosures:

1. Amendment No. to DPR-63
2. Safety Evaluation

cc: w/enclosures  
See next page

*Haughey*

*Rob*

*Smith*

*OGC/PA/ak secy*

*M Jones*

OFC	:PDI-1	:PDI-1	:PDI-1	:SRXB	:PDI-1	:OGC/PA/ak secy
NAME	:CVogan	:RBenedict	:vr:MHaughey	:WHodges	:RCapra	:M Jones
DATE	:11/17/88	:11/17/88	:11/15/88	:11/17/88	:11/ /88	:11/29/88

Mr. L. Burkhardt III  
Niagara Mohawk Power Corporation

Nine Mile Point Nuclear Station,  
Unit No. 1

cc:

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Ms. Donna Ross  
New York State Energy Office  
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16th Floor  
Albany, New York 12223



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 103  
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated January 14, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

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PDR ADOCK 05000220  
P PDC

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 103, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*for Joseph D. Neighbores*

Robert A. Capra, Director  
Project Directorate I-1  
Division of Reactor Projects, I/II

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: January 26, 1989

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 103 TO FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise Appendix A as follows:

Remove Page

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Insert Page

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LIMITING CONDITION FOR OPERATION

SURVEILLANCE REQUIREMENT

- (b) Whenever the reactor is in the startup or run mode below 20% rated thermal power, no control rods shall be moved unless the rod worth minimizer is operable, except as noted in 4.1.1.b(3)(a)(iv), or a second independent operator or engineer verifies that the operator at the reactor console is following the control rod program. The second operator may be used as a substitute for an inoperable rod worth minimizer during a startup only if the rod worth minimizer fails after withdrawal of at least twelve control rods.

If the rod worth minimizer fails prior to the complete withdrawal of the first twelve rods, then the withdrawn rods shall be inserted in the reverse order in which they were withdrawn. A second independent operator or engineer shall verify that the operator at the reactor controls is following the control rod program in reverse order.

- (4) Control rods shall not be withdrawn for approach to criticality unless at least three source range channels have an observed count rate equal to or greater than three counts per second.

- (iv) The rod block function of the rod worth minimizer shall be verified by attempting to withdraw an out-of-sequence control rod beyond the block point.

- (b) If the rod worth minimizer is inoperable while the reactor is in the startup or run mode below 20% rated thermal power and a second independent operator or engineer is being used he shall verify that all rod positions are correct prior to commencing withdrawal of each rod group.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 103 TO FACILITY OPERATING LICENSE NO. DPR-63

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR POWER STATION, UNIT NO. 1

DOCKET NO. 50-220

INTRODUCTION

By letter dated January 14, 1988, Niagara Mohawk Power Corporation (the licensee) has proposed a change to Technical Specification 3.1.1.b(3)(b) to eliminate a contradiction between that Specification and Specification 3.1.1.e. Currently, Specification 3.1.1.b(3)(b) requires that, when reactor power is less than 20% and fewer than 12 control rods have been withdrawn, no control rods shall be moved without an operable rod worth minimizer. However, Specification 3.1.1.e requires that, when Technical Specifications 3.1.1.a through d are not met, (including an operable rod worth minimizer), the reactor shall be placed in hot shutdown within 10 hours. Thus, the rod movement requirement of Specification 3.1.1.e is prohibited by the contradiction of Specification 3.1.1.b(3)(b).

In order to resolve this contradiction, the licensee has proposed to revise Specification 3.1.1.b(3)(b) to require that, if the rod worth minimizer fails prior to the complete withdrawal of the first 12 rods, the rods are to be inserted in a reverse order from which they have been withdrawn. The revision would also require a second independent operator or engineer to verify that the operator at the reactor controls is following the control rod program in reverse order.

DISCUSSION AND EVALUATION

The rod worth minimizer (RWM) is installed to enhance safety, during reactor startup, by blocking the withdrawal of control rods if they are pulled in improper order. If the RWM is inoperable while control rods are being withdrawn, this safety feature is not available to prevent unacceptable consequences from a potential control rod drop accident. By eliminating the contradiction noted above, the proposed revision would make Specification 3.1.1.b(3)(b) consistent with the original intent that the reactor would be shut down if the RWM became inoperable. The revision is, therefore, acceptable.

The inclusion, in Specification 3.1.1.b(3)(b), of verification that the control rod program is being followed properly, adds to safety by assuring that control rods will be inserted in proper order. It is, therefore, acceptable.

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ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of the facility components located within the restricted areas as defined in 10 CFR 20. The staff has determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: January 26, 1989

PRINCIPAL CONTRIBUTOR:

R. Benedict