



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

December 29, 1994

Mr. B. Ralph Sylvia  
Executive Vice President, Nuclear  
Niagara Mohawk Power Corporation  
Nine Mile Point Nuclear Station  
P.O. Box 63  
Lycoming, NY 13093

SUBJECT: ISSUANCE OF AMENDMENT FOR NINE MILE POINT NUCLEAR STATION UNIT NO. 1  
(TAC NO. M90278)

Dear Mr. Sylvia:

The Commission has issued the enclosed Amendment No. 151 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station Unit No. 1 (NMP-1). The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated August 26, 1994.

The amendment revises TS 4.3.3.c.(1) to permit a one-time extension of the second 10-year service period for the primary containment integrated leakage rate (Type A) test. The one-time extension permits delaying the third Type A test of the second 10-year service period from the 1995 refueling outage until the 1997 refueling outage. This delay will result in an interval of approximately 46 months between the second and third Type A tests of the second 10-year service period.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Donald S. Brinkman, Senior Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-220

Enclosures: 1. Amendment No. 151 to DPR-63  
2. Safety Evaluation

cc w/encls: See next page

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B. Ralph Sylvia  
Niagara Mohawk Power Corporation

Nine Mile Point Nuclear Station  
Unit No. 1

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DATED: December 29, 1994

AMENDMENT NO. 151 TO FACILITY OPERATING LICENSE NO. DPR-63-NINE MILE POINT  
UNIT NO. 1

Docket File

PUBLIC

PDI-1 Reading

S. Varga, 14/E/4

J. Zwolinski, 14/H/3

M. J. Case

C. Vogan

D. Brinkman

OGC

D. Hagan, T4 A43

G. Hill (2), T5 C3

C. Grimes, 11/E/22

ACRS (4)

OPA

OC/LFDCB

PD plant-specific file

C. Cowgill, Region I

R. Barrett

cc: Plant Service list



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 151  
License No. DPR-63

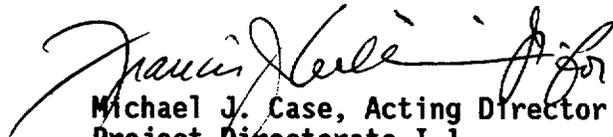
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated August 26, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 151, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael J. Case, Acting Director  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: December 29, 1994

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 151 TO FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise Appendix A as follows:

Remove Page

134

Insert Page

134

**LIMITING CONDITION FOR OPERATION**

**SURVEILLANCE REQUIREMENT**

(3) When adding the leakage rate measured during a Type C test to the results of a Type A test, the leakage rate shall be determined using minimum pathway analysis.

c. Frequency

(1) Three Type A tests shall be conducted during each ten year service interval at approximately equal intervals. The third test will be conducted when the plant is shutdown for the 10 year inservice inspections.\*

(2) Retesting

(a) If a Type A test fails to meet the acceptance criteria of 4.3.3.b.(1), a Corrective Action Plan that focuses attention on the cause of the problem shall be developed and implemented. A Type A test that meets the requirements of 4.3.3.a.(3) and 4.3.3.b.(2) is required prior to plant start-up. A report of the Corrective Action following the failed Type A shall be submitted to the NRC for review and approval with the Containment Leak Test Report.

\* The duration of the second ten year service interval will be extended to correspond with the end of the ISI interval.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 151 TO FACILITY OPERATING LICENSE NO. DPR-63  
NIAGARA MOHAWK POWER CORPORATION  
NINE MILE POINT NUCLEAR STATION UNIT NO. 1  
DOCKET NO. 50-220

1.0 INTRODUCTION

By letter dated August 26, 1994, Niagara Mohawk Power Corporation (the licensee or NMPC) submitted a request for changes to the Nine Mile Point Nuclear Station Unit No. 1 (NMP-1), Technical Specifications (TSs). The requested changes would revise TS 4.3.3.c.(1) to permit a one-time extension of the second 10-year service period for the primary containment integrated leakage rate (Type A) test. The one-time extension would permit delaying the third Type A test of the second 10-year service period from the 1995 refueling outage until the 1997 refueling outage. This delay would result in the interval between the second and third Type A tests of the second 10-year service period being approximately 46 months.

By a second letter dated August 26, 1994, NMPC also requested a one-time only schedular exemption to delay performance of the third Type A test of the second 10-year service period for the primary containment until the 1997 refueling outage. The NRC staff issued an environmental assessment in support of the requested exemption on November 30, 1994. The exemption was issued on December 20, 1994. These actions were noticed in the Federal Register on December 6, 1994 (59 FR 62752) and December 27, 1994 (59 FR 66568), respectively.

2.0 EVALUATION

Section III.D.1.(a) of Appendix J to 10 CFR Part 50 and TS 4.3.3.c.(1) each require the performance of three Type A containment leakage rate tests, at approximately equal intervals during each 10-year service period of the primary containment. The third test of each set is required to be conducted when the plant is shutdown for the 10-year inspection of the primary containment. These tests provide assurance that the containment leakage rate will remain within acceptable limits during plant operation.

Type A leakage rate tests were performed as required during the first 10-year containment service period (1974-1984). The second 10-year containment service period is scheduled to end in December 1994 and; therefore, the third Type A test would normally be performed during the 1995 refueling outage which is currently scheduled to begin in February 1995.

To minimize the duration of refueling outages, the licensee desires to have the containment 10-year service period coincide with the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) required 10-year inservice inspection (ISI) periods since the licensee estimates that performance of a Type A test adds 4 days to a refueling outage schedule, results in an estimated occupational exposure of 2.25 person-rem, and costs approximately \$2.5 million dollars. However, due to a lengthy outage for the replacement of reactor recirculation piping, the first ISI period was extended to June 1986. An extended refueling outage (January 1987 to July 1990) resulted in the second 10-year ISI interval being extended to December 1998. The extensions of ISI periods were made in accordance with the provisions of Section XI of the ASME Code and have resulted in the ISI intervals being decoupled from the Type A leakage test intervals since Appendix J does not contain any provisions for adjusting the 10-year intervals for extended outages as does the ASME Code.

Two Type A tests have already been performed (May 1990 and April 1993) during the second 10-year containment service interval. Since the interval is scheduled to end in December 1994, the third Type A test for this interval would be required during the next refueling outage (February 1995). However, as noted above and in an attempt to recouple the schedule for Type A tests with the ISI schedule, the licensee has proposed to extend the third Type A test of the second containment service interval until the 1997 refueling outage. The licensee has also proposed to perform the final outage tests of the second ISI interval during the 1997 refueling outage. (Some non-outage tests may be performed after the 1997 refueling outage but prior to the end of the 10-year ISI intervals). This action would eliminate the need to perform an extra Type A test which could otherwise be required (one test in 1995 and another in 1997) while recoupling the Type A test schedule with the ISI schedule.

The last Type A test was performed in April 1993. Performance of another Type A test in February 1995 would result in a test interval of only 22 months. Whereas if the 10-year interval was equally divided into three intervals, each test interval would be 40 months. Delaying the next Type A test until the 1997 refueling outage would result in a test interval of approximately 46 months since NMP-1 is currently operating on a 24-month fuel cycle.

The licensee has also provided the following information to support extending the containment service period and hence delaying the next Type A test until the 1997 refueling outage:

1. The two most recent Type A test data show that the "as left" leakage rates (0.4634% wt/day and 0.4634% wt/day, respectively) were well within the acceptance limit of 0.75 Lt (0.892% wt/day).

2. There have been no permanent or temporary modifications to the containment structure, liner, or penetrations since the last Type A test that could adversely affect the Type A test results.
3. No modifications that require a Type A test are planned prior to the 1997 refueling outage.
4. There have been no pressure or temperature excursions in the containment which could have adversely affected containment integrity.

This information indicates the containment's recent performance has been acceptable, that there have been no indications of abnormal degradation, and no actions are planned which would adversely affect containment leakage rates.

Since the containment's recent performance does not indicate any abnormal degradation and since the proposed interval for the next Type A test would be consistent with the NRC staff's current position regarding surveillance frequencies (typically nominal interval plus a maximum of 25 percent of the nominal interval with the nominal interval in this case being 40 months), we have concluded that the proposed one-time extension of the second containment service period (and delay of the third Type A set for this period) until the 1997 refueling outage is acceptable.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and a surveillance requirement. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (59 FR 49431). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

## 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

**Principal Contributor:** Donald S. Brinkman

**Date:** December 29, 1994

December 29, 1994

Mr. B. Ralph Sylvia  
Executive Vice President, Nuclear  
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A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by

Donald S. Brinkman, Senior Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-220

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