

October 16, 1995

Mr. B. Ralph Sylvia
Executive Vice President, Nuclear
Niagara Mohawk Power Corporation
Nine Mile Point Nuclear Station
P.O. Box 63
Lycoming, NY 13093

SUBJECT: ISSUANCE OF AMENDMENT FOR NINE MILE POINT NUCLEAR STATION UNIT
NO. 1 (TAC NO. M91499)

Dear Mr. Sylvia:

The Commission has issued the enclosed Amendment No. 155 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station Unit No. 1. The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated February 1, 1995.

The amendment revises TS 3.6.13 and associated Bases to permit the controls and instruments from both Remote Shutdown Panels to be considered when assuring that one complete set of controls and instruments is operable. The changes also allow 30 days to restore an inoperable function to operable status, remove MODE 3 (hot shutdown) from the existing requirement for operability, and revise the LIMITING CONDITION FOR OPERATION ACTION to require achieving hot shutdown in 12 hours instead of cold shutdown in 36 hours. An additional change permits the operator 30 days to establish an alternate method of monitoring a parameter, located remote from the control room and consistent with Appendix R, (and 90 days to restore the function) when the function is inoperable.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by:

Gordon E. Edison, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-220

Enclosures: 1. Amendment No. 155 to DPR-63
2. Safety Evaluation

cc w/encls: See next page
Distribution: See attached sheet

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

October 16, 1995

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Executive Vice President, Nuclear
Niagara Mohawk Power Corporation
Nine Mile Point Nuclear Station
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Sincerely,

A handwritten signature in cursive script that reads "Gordon E. Edison".

Gordon E. Edison, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-220

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2. Safety Evaluation

cc w/encls: See next page

B. Ralph Sylvia
Niagara Mohawk Power Corporation

Nine Mile Point Nuclear Station
Unit No. 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 155
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated February 1, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 155, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION


Ledyard B. Marsh, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 16, 1995

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 155 TO FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise Appendix A as follows:

Remove Pages

277
278
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Insert Pages

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LIMITING CONDITION FOR OPERATION

3.6.13 REMOTE SHUTDOWN PANELS

Applicability:

Applies to the operating status of the remote shutdown panels.

Objective:

To assure the capability of the remote shutdown panels to provide 1) initiation of the emergency condensers independent of the main/auxiliary control room 2) control of the motor-operated steam supply valves independent of the main/auxiliary control room and 3) parameter monitoring outside the control room.

Specification:

- a. During power operation, the remote shutdown panels' Functions in Table 3.6.13-1 shall be operable.

SURVEILLANCE REQUIREMENT

4.6.13 REMOTE SHUTDOWN PANELS

Applicability:

Applies to the periodic testing requirements for the remote shutdown panels.

Objective:

To assure the capability of the remote shutdown panels to provide 1) initiation of the emergency condensers independent of the main/auxiliary control room 2) control of the motor-operated steam supply valves independent of the main/auxiliary control room and 3) parameter monitoring outside the control room.

Specification:

The remote shutdown panels surveillance shall be performed as indicated below:

- a. Each remote shutdown panel monitoring instrumentation channel shall be demonstrated operable by performance of the operations and frequencies shown in Table 4.6.13-1.
- b. During each major refueling outage
 - 1. Each remote shutdown panel shall be demonstrated to initiate the emergency condensers independent of the main/auxiliary control room.

LIMITING CONDITION FOR OPERATION

- b. With the valve control Function inoperable, restore the required Function to operable status within 30 days.
- c. With one or more required monitoring instrument Functions inoperable, restore the required Function to operable status within 30 days or establish an alternate method of monitoring the parameter within 30 days and restore the required Function to operable status within 90 days.
- d. If the required action and associated completion time is not met, be in hot shutdown within the next 12 hours.

SURVEILLANCE REQUIREMENT

- 2. Each remote shutdown panel shall be demonstrated to open both the motor-operated steam valves.

TABLE 3.6.13-1

REMOTE SHUTDOWN PANELS FUNCTIONS

Limiting Condition for Operation

<u>FUNCTION</u>	<u>MINIMUM NUMBER OF OPERABLE CHANNELS PER FUNCTION</u>
Reactor Pressure	1
Reactor Water Level	1
Reactor Water Temperature	1
Torus Water Temperature	1
Drywell Pressure	1
Emergency Condenser Water Level	1
Drywell Temperature	1
"All Rods In" Light	1
Emergency Condenser Condensate Return Valve and Motor-Operated Steam Supply Valves Control on the Same Panel	1

BASES FOR 3.6.13 AND 4.6.13 REMOTE SHUTDOWN PANELS

The remote shutdown panels provide 1) manual initiation of the emergency condensers 2) manual control of the steam supply valves and 3) parameters monitoring independent of the main/auxiliary control room. Two panels are provided, each located in a separate fire area, for added redundancy. Both panels are also in separate fire areas from the main/auxiliary control room. One channel of each Function provides the necessary capabilities consistent with 10CFR50 Appendix R. Therefore, only one channel of either remote shutdown panel monitoring instrument or control is required to be operable. The electrical design of the panels is such that no single fire can cause loss of both emergency condensers.

Each remote shutdown panel is provided with controls for one emergency condenser loop. The emergency condensers are designed such that automatic initiation is independently assured in the event of a fire 1) in the Reactor Building (principle relay logic located in the auxiliary control room or 2) in the main/auxiliary control room or Turbine Building (redundant relay logic located in the Reactor Building). Each remote shutdown panel also has controls to operate the two motor-operated steam supply valves on its respective emergency condenser loop. A key operated bypass switch is provided to override the automatic isolation signal to these valves. Once the bypass switch is activated, the steam supply valves can be manually controlled from the remote shutdown panels. Since automatic initiation of the emergency condenser is assured, the remote shutdown panels serve as additional manual controlling stations for the emergency condensers. In addition, certain parameters are monitored at each remote shutdown panel.

The remote shutdown panels are normally de-energized, except for the monitoring instrumentation, which is normally energized. To energize the remaining functions on a remote shutdown panel, a power switch located on each panel must be activated. Once the panels are completely energized, the emergency condenser condensate return valve and steam supply valve controls can be utilized.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 155 TO FACILITY OPERATING LICENSE NO. DPR-63

NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT NUCLEAR STATION UNIT NO. 1

DOCKET NO. 50-220

1.0 INTRODUCTION

By letter dated February 1, 1995, Niagara Mohawk Power Corporation (the licensee) submitted a request for changes to the Nine Mile Point Nuclear Station Unit No. 1 (NMP1), Technical Specifications (TSs). The proposed changes alter the requirements for the Limiting Conditions for Operation and the Bases for the instrumentation and controls for the remote shutdown capability at NMP1. The licensee proposes these changes to bring consistency between the NMP1 TSs and NUREG-1433, "Standard Technical Specifications - General Electric Boiling Water Reactors (BWR/4)." The licensee's submittal includes the justification for the proposed changes and provides the basis for the no significant hazards consideration.

2.0 BACKGROUND

The remote shutdown panels provide 1) manual initiation of the emergency condensers 2) manual control of the steam supply valves and 3) parameters monitoring independent of the main/auxiliary control room. Two panels are provided; each located in a separate fire area for added redundancy. Both panels are also in separate fire areas from the main/auxiliary control room. One channel of each Function provides the necessary capabilities consistent with 10CFR50 Appendix R. Therefore, only one channel of either remote shutdown panel monitoring instrument or control is required to be operable. The electrical design of the panels is such that no single fire can cause loss of both emergency condensers.

The existing TS requires that at least one remote shutdown panel be operable during power operation and when the reactor coolant temperature is above 212 °F. When this requirement cannot be met, an orderly plant shutdown must commence within 24 hours and cold shutdown must be reached within 36 hours. A remote shutdown panel is considered inoperable if any of the presently-required control or instrumentation functions is not operable in that panel. These required functions include the emergency condenser return valve control switch, either of the motor-operated steam supply valve control switches, and the required operable instrument channels as indicated in Table 3.6.13-1, "REMOTE SHUTDOWN PANEL MONITORING - Limiting Condition for Operation."

The licensee indicated that the existing TS requirements are overly conservative compared to NUREG-1433 and to Amendment No. 216 issued to the James A. FitzPatrick Nuclear Power Plant on August 31, 1994, and has proposed changes to relax the existing requirements and thereby preclude unnecessary plant shutdowns.

3.0 EVALUATION

The proposed changes relax the existing requirement that all of the required functions must be operable on the same remote shutdown panel, and require instead that between the two remote shutdown panels, one channel of each required function be operable. The operators communicate with one another and the remaining personnel from each panel using the plant Gaitronix system. These changes could potentially avoid an unnecessary plant shutdown without affecting an operator's ability to cope with a control room evacuation. One channel of each function is adequate to assure a safe shutdown. The NRC staff finds the above to be consistent with NUREG-1433 and is, therefore, acceptable.

The proposed TS changes also allow 30 days to restore an inoperable function to operable status. If this action is not completed, then the plant must be brought to a hot shutdown condition within 12 hours. The staff finds that this is consistent with NUREG-1433 and is, therefore, acceptable.

With one or more of the required monitoring instrument functions inoperable, the TS changes propose to allow the licensee an additional option. Specifically, the operator is allowed 30 days to establish an alternate method of monitoring the parameter and 90 days to restore the function to operable status. The staff finds that at NMP1 the temporary provision of an alternate method for monitoring the required instrumentation function, located remote from the control zone and consistent with Appendix R, is consistent with the intent of NUREG-1433 because it will provide the operator with indication of the parameter of interest, and is acceptable.

The proposed TS changes remove Mode 3 (Hot Shutdown) from the existing specification for operability by deleting the phrase "whenever the reactor coolant temperature is greater than 212 °F" from the existing specification. The staff finds that this change, removal of Mode 3 from the requirement for operability, is consistent with NUREG-1433 which requires operability only during Mode 1 (Power Operation) and Mode 2 (Startup/Hot Standby) and is, therefore, acceptable.

The proposed TS changes require that the plant be brought to a hot shutdown condition in 12 hours (versus a cold shutdown condition within 36 hours). As indicated in NUREG-1433, the 12-hour completion time is reasonable based on operating experience. The staff finds that this change is consistent with NUREG-1433 and is, therefore, acceptable.

The licensee proposes to revise Bases Sections 3.6.13 and 4.6.13 to provide consistency with the proposed changes to the TSs. The Bases Section currently indicates that one remote shutdown panel is required to be operable. In the proposed TS change, one channel of each function (between the two panels) is required to maintain remote shutdown operability. The staff finds that the revisions to the Bases Section are consistent with NUREG-1433 and are, therefore, acceptable.

Editorial changes are made to Table 3.6.13-1 for consistency with the proposed changes to the TSs. Specifically, the word "INSTRUMENT" is changed to "FUNCTION" and the words "PANEL MONITORING" are changed to the words "PANELS FUNCTIONS." These changes make clear the proposal that one operable channel of each function (between the two panels) is acceptable to maintain operability. The operability requirements for the emergency condenser condensate return valve control and motor-operated steam supply valves control are relocated from Specification 3.6.13.b to Table 3.6.13-1 for consistency with the proposed changes. The staff finds that these editorial changes are consistent with NUREG-1433 and are, therefore, acceptable.

Based on the review of the licensee's submittal, the staff concludes that the proposed TS changes are consistent with NUREG-1433 and previously approved TS changes and are, therefore, acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (60 FR 11135). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: K. Mortensen

Date: October 16, 1995

DATED: October 16, 1995

AMENDMENT NO. 155 TO FACILITY OPERATING LICENSE NO. DPR-63-NINE MILE POINT
UNIT NO. 1

Docket File
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