



**UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION IV
611 RYAN PLAZA DRIVE, SUITE 400
ARLINGTON, TEXAS 76011-8064**

April 16, 2001

J. H. Swailes, Vice President of
Nuclear Energy
Nebraska Public Power District
P.O. Box 98
Brownville, Nebraska 68321

**SUBJECT: COOPER NUCLEAR STATION - REQUEST FOR ADDITIONAL INFORMATION
REGARDING A POTENTIAL YELLOW FINDING (EA 00-248)**

Dear Mr. Swailes:

On March 29, 2001, a regulatory conference was conducted in the Region IV office between you, your staff, and the Nuclear Regulatory Commission (NRC) staff. The participants discussed the risk significance of a potential yellow finding and associated apparent violations, identified at your Cooper Nuclear Station, involving programmatic environmental qualification design, implementation, and documentation deficiencies.

As discussed during the conference, in order for the NRC staff to complete its evaluation, additional information is necessary. The staff has reviewed the information provided in your March 22, 2001, letter as augmented by your presentation during the conference. With that in mind, a response to the enclosed Request for Additional Information is required.

The contents of this request has been discussed with Mr. John McDonald of your staff on April 16, 2001. As discussed at that time, we have established a response time frame of 30 days from the date of this letter. If for any reason this date becomes unreasonable, please contact me at your earliest opportunity.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," Part 2, Title 10, Code of Federal Regulations, a copy of this letter and its enclosures will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/NRC/ADAMS/index.html> (the Public Reading Room).

Should you have any questions concerning this matter, we will be pleased to discuss them with you.

Sincerely,

/RA/

Charles S. Marschall, Chief
Project Branch C
Division of Reactor Projects

Docket No.: 50-298
License No.: DPR-46

Enclosure:
Request for Additional Information

cc w/enclosure:
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Jim Isom, Pilot Plant Program (**JAI**)

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ENCLOSURE

REQUEST FOR ADDITIONAL INFORMATION

REGARDING THE RISK SIGNIFICANCE OF A POTENTIAL YELLOW FINDING

NEBRASKA PUBLIC POWER DISTRICT

COOPER NUCLEAR STATION

DOCKET 50-298

Division of Reactor Projects, Project Branch C

1. Provide copies of environmental qualification (EQ) records and documentation for the safety-relief valve pilot solenoid valves. Specifically, the profiles to which the solenoids were qualified.
2. Provide the results of a probabilistic risk assessment sensitivity study (risk achievement worth) for the safety-relief valves in the overall plant configuration presented for the EQ issue.
3. Provide copies of all new test reports discussed during the conference and the associated data. Include as a minimum, the safety-relief valve tailpiece pressure switches, the fuses, and the airlock door indicating light.
4. Provide EQ records and documentation for the qualification of motor control centers on the reactor building 903-foot elevation, specifically, the profiles to which these centers were qualified.
5. Provide the controlled drawings and data used for your ground fault analysis and the 125 Vdc fuse/breaker coordination review.
6. Provide the definitions and scope of your category "medium" harsh environment EQ profile.
7. Provide a list of components that are considered to be in a "medium" harsh environment.
8. Provide the results of a sensitivity study evaluating the impact of the assumptions used in "medium" harsh environments.
9. Provide qualification data and records for the Weidmueller terminal blocks in the drywell that were associated with the safety-relief valves.
10. Provide the likelihood and risk impact of distribution system level fuses (20 amp) in the 125 Vdc system opening as a result of the postulated ground faults caused by the unqualified EQ treatments.
11. Provide your assessment of the likelihood that either the Distribution Panel A fuse (100 amp) or the Switchgear 1A fuse (450 amp) would open as a result of the postulated ground faults.