

July 7, 1989

Docket No. 50-220

DISTRIBUTION

Mr. Lawrence Burkhardt III
Executive Vice President, Nuclear Operations
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

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Dear Mr. Burkhardt:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. 71138)

The Commission has issued the enclosed Amendment No. 107 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station Unit No. 1 (NMP-1). The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated October 19, 1988.

This amendment revises Technical Specifications 3.2.6 and 4.2.6 and their Bases to bring them into conformance with the staff positions delineated in NRC Generic Letter 88-01, "NRC Position on IGSCC in BWR Austenitic Stainless Steel Piping."

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Original signed by

Marylee M. Slosson, Project Manager
Project Directorate I-1
Division of Reactor Projects, I/II

Enclosures:

1. Amendment No. 107 to DPR-63
2. Safety Evaluation

cc: w/enclosures
See next page

[AMEND 220 TAC 71138]

OFC	:PDI-1	:PDI-1	:PDI-1	:OGC	:PDI-1	:	:
NAME	:CVogan	:RBenedict/vr/bah	:MSlosson	:RCapra	:	:	:
DATE	:5/26/89	:5/30/89	:6/5/89	:6/18/89	:6/19/89	:	:

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Handwritten signatures and initials:
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Nine Mile Point Nuclear Station,
Unit No. 1

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.107
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated October 19, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 107, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Capra

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects, I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 7, 1989

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 107 TO FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise Appendix A as follows:

Remove Pages

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Insert Pages

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LIMITING CONDITION FOR OPERATION

SURVEILLANCE REQUIREMENT

b. Inservice Testing

1. To be considered operable, Quality Group A, B and C pumps and valves, shall satisfy the requirements contained in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda for continued service of ASME Code Class 1, 2 and 3 components, respectively, except where relief has been granted by the Commission pursuant to 10CFR50, Section 50.55a(g)(6)(i).

c. Performance of the above inservice inspection and testing activities shall be in addition to other specified Surveillance Requirements.

d. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any Technical Specification.

2. The Inservice Inspection Program for piping identified in NRC Generic Letter 88-01 shall be performed in accordance with the staff positions on schedule, methods, personnel and sample expansion included in this generic letter.

b. Inservice Testing

1. Inservice testing of Quality Group A, B and C pumps and valves shall be performed in accordance with the requirements for ASME Code Class 1, 2 and 3 components contained in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10CFR50, Section 50.55a(g), except where relief has been granted by the Commission pursuant to 10CFR Part 50, Section 50.55a(g)(6)(i).

BASES FOR 3.2.6 AND 4.2.6 INSERVICE INSPECTION AND TESTING

The inservice inspection and testing programs⁽¹⁾⁽²⁾ for the Nine Mile Point Unit 1 plant conform to the requirements of 10CFR50, Section 50.55a(g). Where practical, the inspection of components, pumps and valves classified into NRC Quality Groups A, B and C conforms to the requirements of ASME Code Class 1, 2 and 3 components, pumps and valves, respectively, contained in Section XI of the ASME Boiler and Pressure Vessel Code. If a Code required inspection is impractical for the Nine Mile Point Unit 1 facility, a request for relief from that requirement is submitted to the Commission in accordance with 10CFR50, Section 50.55a(g)(6)(i).

Request for relief from the requirements of Section XI of the ASME Code and applicable Addenda will be submitted to the Commission prior to the beginning of each 10-year inspection interval if they are known to be required at the time. Requests for relief which are identified during the course of inspection will be submitted quarterly throughout the inspection interval.

The inservice inspection program for piping conforms to the staff positions on schedules, methods, personnel and sample expansion contained in Generic Letter 88-01.⁽³⁾ It is performed in order to detect and survey intergranular stress corrosion cracking of BWR austenitic stainless steel piping that is four inches or larger in nominal diameter and contains reactor coolant at a temperature above 200°F during power operation. Inspections shall be performed by individuals qualified to: (A) the ASME Boiler and Pressure Vessel Code, Section XI, and (B) Ultrasonic Testing Operator Training for the Detection of Intergranular Stress Corrosion Cracking developed by the EPRI Non-Destructive Examination Center. As an alternate, Niagara Mohawk may use other qualification programs approved by the NRC.

References

- (1) Letter from the Nuclear Regulatory Commission (D. B. Vassallo) to Niagara Mohawk Power Corporation (G. K. Rhode), dated September 19, 1983.
- (2) Letter from Niagara Mohawk Power Corporation (D. P. Dise) to the Nuclear Regulatory Commission (T. A. Ippolito), dated August 7, 1981.
- (3) Generic Letter 88-01 endorses NUREG 0313 Revision 2, "Technical Report on Material Selection and Processing Guidelines for BWR Coolant Pressure Boundary Piping," dated January 1988.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 107 TO FACILITY OPERATING LICENSE NO. DPR-63
NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT NUCLEAR STATION, UNIT NO. 1
DOCKET NO. 50-220

INTRODUCTION

NRC Generic Letter 88-01 was sent to BWR licensees on January 25, 1988. This Generic Letter (GL) presented NRC staff positions on intergranular stress corrosion cracking (IGSCC) in austenitic stainless steel piping used in boiling water reactors. The GL requested BWR licensees to furnish their current plans relating to piping replacement, inspection, repair, and leakage detection.

In partial response to GL 88-01, Niagara Mohawk Power Company (NMPC) submitted, by letter dated October 19, 1988, an application for an amendment to the Nine Mile Point Unit 1 (NMP-1) operating license. The amendment would revise the NMP-1 Technical Specifications 3.2.6 and 4.2.6 and their Bases, which concern Inservice Inspection and Testing. The NRC staff's evaluation of the licensee's proposal is given below.

EVALUATION

The licensee has proposed to add two requirements to Technical Specification 3.2.6:

- c. Performance of the above inservice inspection and testing activities shall be in addition to other specified Surveillance Requirements.
- d. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any Technical Specification.

These additional requirements are identical to those of Specifications 4.0.5.d and 4.0.5.e of the NRC Model BWR Standard Technical Specification for Item 3 of Generic Letter 88-01; they are, therefore, acceptable.

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The licensee has also proposed to revise Technical Specification 4.2.6.a.2 to read as follows:

2. The Inservice Inspection Program for piping identified in NRC Generic Letter 88-01 shall be performed in accordance with the staff positions on schedule, methods, personnel and sample expansion included in this generic letter.

This requirement is the same as that of Specification 4.0.5.f of the NRC Model BWR standard Technical Specification for Item 3 of Generic Letter 88-01; it is, therefore, acceptable.

The licensee has also proposed to update the Bases for Specifications 3.2.6 and 4.2.6. The Bases will now refer to Generic Letter 88-01 which supersedes earlier guidance. By oral agreement between the staff and the licensee, the phrase "temperature of 200°F" in the fourth line of the proposed revised paragraph has been changed to "temperature above 200°F" in order to be consistent with the wording of GL 88-01. With that change, the proposed Bases appropriately support the applicable specifications and are acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the surveillance requirements for the facility components located within the restricted areas as defined in 10 CFR 20. The staff has determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: July 7, 1989

PRINCIPAL CONTRIBUTOR:

R. Benedict