

December 31, 1990

Docket Nos. 50-220
and 50-410

Distribution:

PDI-1 Reading	GHill(8)
Docket File	Wanda Jones
NRC/Local PDRs	JCalvo
SVarga	ACRS(10)
EGreenman	GPA/PA
RACapra	OC/LFMB
DBrinkman	DOudinot
EBrach	FAllenspach
OGC	DHagan
EJordan	Plant File
JLinville	

Mr. B. Ralph Sylvia
Executive Vice President, Nuclear
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Dear Mr. Sylvia:

SUBJECT: ISSUANCE OF AMENDMENTS FOR NINE MILE UNITS 1 AND 2
(TAC NOS. 77554 AND 77555)

The Commission has issued the enclosed Amendment No. 120 to Facility Operating License No. DPR-63 and Amendment No. 25 to Facility Operating License No. NPF-69 for the Nine Mile Point Nuclear Station Unit Nos. 1 and 2 (NMP-1 and NMP-2). The amendments consist of changes to the Technical Specifications in response to your applications transmitted by letter dated September 21, 1990, as supplemented October 4, 1990, and November 30, 1990, for NMP-1 and by letter dated September 21, 1990, as supplemented October 4, 1990, and December 10, 1990, for NMP-2.

These amendments revise Technical Specifications Section 6.0, Administrative Controls, to reflect management changes resulting from the reorganization of the Nuclear Division.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

Donald S. Brinkman, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - 1/11
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 120 to DPR-63
2. Amendment No. 25 to NPF-69
3. Safety Evaluation

cc: w/enclosures
See next page

PDI-1:LA
CVogan
12/31/90

PDI-1:PE
DOudinot:rsc
12/28

PDI-1:PM
DBrinkman
12/28/90

EWMA
LPEB
NRR/~~WIB~~
WBrach
12/28/90

OGC
CPW
12-28-90

PDI-1:D
RACapra
12/31/90

DOCUMENT NAME: NMP 1/2 AMEND 77554/77555

9101100397 901231
PDR ADOCK 05000220
P PDR

690077

DFol
11

10/6 *10*



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

December 31, 1990

Docket Nos. 50-220
and 50-410

Mr. B. Ralph Sylvia
Executive Vice President, Nuclear
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Dear Mr. Sylvia:

SUBJECT: ISSUANCE OF AMENDMENTS FOR NINE MILE UNITS 1 AND 2
(TAC NOS. 77554 AND 77555)

The Commission has issued the enclosed Amendment No. 120 to Facility Operating License No. DPR-63 and Amendment No. 25 to Facility Operating License No. NPF-69 for the Nine Mile Point Nuclear Station Unit Nos. 1 and 2 (NMP-1 and NMP-2). The amendments consist of changes to the Technical Specifications in response to your applications transmitted by letter dated September 21, 1990, as supplemented October 4, 1990, and November 30, 1990, for NMP-1 and by letter dated September 21, 1990, as supplemented October 4, 1990, and December 10, 1990, for NMP-2.

These amendments revise Technical Specifications Section 6.0, Administrative Controls, to reflect management changes resulting from the reorganization of the Nuclear Division.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Donald S. Brinkman".

Donald S. Brinkman, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 120 to DPR-63
2. Amendment No. 25 to NPF-69
3. Safety Evaluation

cc: w/enclosures
See next page

Mr. B. Ralph Sylvia
Niagara Mohawk Power Corporation

Mr. Mark J. Wetterhahn, Esquire
Bishop, Cook, Purcell & Reynolds
1400 L. Street, N.W.
Washington, D. C. 20005-3502

Supervisor
Town of Scriba
R. D. #4
Oswego, New York 13126

Mr. Joseph F. Firlit
Vice President - Nuclear Generation
Niagara Mohawk Power Corporation
Nine Mile Point Nuclear Station
Post Office Box 32
Lycoming, New York 13093

Resident Inspector
U. S. Nuclear Regulatory Commission
Post Office Box 126
Lycoming, New York 13093

Mr. Gary D. Wilson, Esquire
Niagara Mohawk Power Corporation
300 Erie Boulevard West
Syracuse, New York 13202

Regional Administrator, Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

Ms. Donna Ross
New York State Energy Office
2 Empire State Plaza
16th Floor
Albany, New York 12223

Nine Mile Point 1/2
Niagara Mohawk Power Corporation

Mr. Kim Dahlberg
Unit 1 Station Superintendent
Nine Mile Point Nuclear Station
Post Office Box 32
Lycoming, New York 13093

Mr. Richard Abbott
Unit 2 Station Superintendent
Nine Mile Point Nuclear Station
Post Office Box 32
Lycoming, New York 13093

Charlie Donaldson, Esquire
Assistant Attorney General
New York Department of Law
120 Broadway
New York, New York 10271

Mr. Paul D. Eddy
State of New York Department of
Public Service
Power Division, System Operations
3 Empire State Plaza
Albany, New York 12223

Mr. Peter E. Francisco, Licensing
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Mr. Richard Goldsmith
Syracuse University
College of Law
E. I. White Halle Campus
Syracuse, New York 12223

Mr. Richard M. Kessel
Chair and Executive Director
State Consumer Protection Board
99 Washington Avenue
Albany, New York 12210



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 120
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Niagara Mohawk Power Corporation (the licensee) dated September 21, 1990, as supplemented October 4, 1990, and November 30, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 120, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 31, 1990

ATTACHMENT TO LICENSE AMENDMENT
AMENDMENT NO. 120 TO FACILITY OPERATING LICENSE NO. DPR-63
DOCKET NO. 50-220

Revise Appendix A as follows:

Remove Pages

245
247
251
252
253
254
255
256
257
258
259
260
261
270

Insert Pages

245
247
251
252
253
254
255
256
257
258
259
260
261
270

6.0 ADMINISTRATIVE CONTROLS

6.1 Responsibility

- 6.1.1 The Plant Manager shall be responsible for overall unit operation and shall delegate in writing the succession to this responsibility during his absence.
- 6.1.2 The Station Shift Supervisor - Nuclear (or during his absence from the control room, a designated individual) shall be responsible for the control room command function. A management directive to this effect, signed by the Executive Vice President - Nuclear shall be re-issued to station personnel on an annual basis.

6.2 Organization

Onsite and Offsite Organization

- 6.2.1 An onsite and an offsite organization shall be established for unit operation and corporate management. The onsite and offsite organization shall include the positions for activities affecting the safety of the nuclear power plant.
 - a. Lines of authority, responsibility and communication shall be established and defined from the highest management levels through intermediate levels to and including all operating organization positions. Those relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions or in equivalent forms of documentation. The organization charts shall be documented in the Final Safety Analysis Report, and the functional descriptions of departmental responsibilities and relationships and job descriptions for key personnel positions are documented in procedures.
 - b. The Executive Vice President - Nuclear shall have corporate responsibility for overall plant nuclear safety. The Executive Vice President - Nuclear shall take any measures needed to assure acceptable performance of the staff in operating, maintaining, and providing technical support in the plant so that continued nuclear safety is assured.
 - c. The Plant Manager shall have responsibility for overall unit operation and shall have control over those resources necessary for safe operation and maintenance of the plant.

Facility Staff (Cont'd)

- 1) An individual should not be permitted to work more than 16 hours straight (excluding shift turnover time).
- 2) An individual should not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any 7 day period (all excluding shift turnover time).
- 3) A break of at least 8-hours should be allowed between work periods (including shift turnover time).
- 4) Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the Plant Manager, or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Controls shall be included in the procedures such that individual overtime shall be reviewed monthly by the Vice President - Nuclear Generation or designee to assure that excessive hours have not been assigned. Routine deviation from the above guidelines is not authorized.

- i. The General Supervisor Operations, Supervisor Operations, Station Shift Supervisor Nuclear and Assistant Station Shift Supervisor Nuclear shall hold senior reactor operator licenses.

6.3 Facility Staff Qualifications

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Manager Radiation Protection who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and the Shift Technical Advisor who shall have a bachelor's degree in a physical science or engineering or a professional engineer license issued by examination and shall have received specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 Training

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Manager Training and shall meet or exceed the recommendations and requirements of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Manager Training and Supervisor-Fire Protection, Nuclear and shall meet or exceed the requirements of Appendix R to 10CFR50.

6.5 Review and Audit

6.5.1 Station Operations Review Committee (SORC)

Function

6.5.1.1 The Station Operations Review Committee shall function to advise the Plant Manager on all matters related to nuclear safety.

Composition

6.5.1.2 The SORC shall be composed of the:

Chairman: Plant Manager
Member: Manager Operations
Member: Manager Maintenance
Member: Manager Technical Support
Member: Manager Health Physics
Member: Manager Chemistry
Member: Manager Radiation Protection
Member: General Supervisor Instrument and Control Maintenance
Member: General Supervisor System Engineering
Member: General Supervisor Operations
Member: Supervisor Reactor Engineering

Alternates

- 6.5.1.3 Alternate members shall be appointed in writing by the SORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in SORC activities at any one time.

Meeting Frequency

- 6.5.1.4 The SORC shall meet at least once per calendar month and as convened by the SORC Chairman.

Quorum

- 6.5.1.5 A quorum of the SORC shall consist of the Chairman or a designated alternate and five members including alternates.

Responsibilities

- 6.5.1.6 The SORC shall be responsible for:
- a. Review of all REPORTABLE EVENTS.
 - b. Review of unit operations to detect potential safety hazards.
 - c. Performance of special reviews, investigations or analyses and reports thereon as requested by the Plant Manager or the Safety Review and Audit Board.
 - d. Investigation of violations of the Technical Specifications and shall prepare and forward a report covering evaluation and recommendations to prevent recurrence to the Vice President - Nuclear Generation and to the Safety Review and Audit Board.

Authority

- 6.5.1.7 The SORC shall:
- a. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6 (a) through (d) above constitutes an unreviewed safety question.
 - b. Provide written notification within 24 hours to the Vice President - Nuclear Generation and the Safety Review and Audit Board of disagreement between the SORC and the Plant Manager; however, the Plant Manager shall have the responsibility for resolution of such disagreements pursuant to 6.1.1 above.

Records

- 6.5.1.8 The SORC shall maintain written minutes of each meeting and copies shall be provided to the Vice President - Nuclear Generation and the Safety Review and Audit Board.

6.5.2 Technical Review and Control

Activities

- 6.5.2.1 Each procedure and program required by Specification 6.8 and other procedures which affect nuclear safety, and changes thereto, shall be prepared by a qualified individual/organization. Each such procedure, and changes thereto, shall be reviewed by an individual/group other than the individual/group which prepared the procedure, or changes thereto, but who may be from the same organization as the individual/group which prepared the procedure, or changes thereto. Approval of procedures and programs and changes thereto and their safety evaluations, shall be controlled by administrative procedures.
- 6.5.2.2 Proposed changes to the Technical Specifications shall be prepared by a qualified individual/organization. The preparation of each proposed Technical Specifications change shall be reviewed by an individual/group other than the individual/group which prepared the proposed change, but who may be from the same organization as the individual/group which prepared the proposed change. Proposed changes to the Technical Specifications shall be approved by the Plant Manager.

Activities (Cont'd)

- 6.5.2.3 Proposed modifications to unit structures, systems and components that affect nuclear safety shall be designed by a qualified individual/organization. Each such modification shall be reviewed by an individual/group other than the individual/group which designed the modification, but who may be from the same organization as the individual/group which designed the modification. Proposed modifications to structures, systems and components and the safety evaluations shall be approved prior to implementation by the Plant Manager; or the Manager Technical Support as previously designated by the Plant Manager.
- 6.5.2.4 Individuals responsible for reviews performed in accordance with Specifications 6.5.2.1, 6.5.2.2 and 6.5.2.3 shall be members of the station supervisory staff, previously designated by the Plant Manager to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary such review shall be performed by the appropriate designated station review personnel.
- 6.5.2.5 Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications and their safety evaluations shall be reviewed by the Plant Manager, or the Manager Technical Support as previously designated by the Plant Manager.
- 6.5.2.6 The Plant Manager shall assure the performance of special reviews and investigations, and the preparation and submittal of reports thereon, as requested by the Vice President - Nuclear Generation.
- 6.5.2.7 The facility security program, and implementing procedures, shall be reviewed at least every 12 months. Recommended changes shall be approved by the Plant Manager and transmitted to the Vice President - Nuclear Generation and to the Chairman of the Safety Review and Audit Board.
- 6.5.2.8 The facility emergency plan, and implementing procedures shall be reviewed at least every 12 months. Recommended changes shall be approved by the Plant Manager and transmitted to the Vice President - Nuclear Generation and to the Chairman of the Safety Review and Audit Board.

Activities (Cont'd)

- 6.5.2.9 The Plant Manager shall assure the performance of a review by a qualified individual/organization of changes to the Radiological Waste Treatment systems.
- 6.5.2.10 Review of any accidental, unplanned, or uncontrolled radioactive release including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President - Nuclear Generation and to the Safety Review and Audit Board.
- 6.5.2.11 Review of changes to the Process Control Program and the Offsite Dose Calculation Manual. Approval of any changes shall be made by the Plant Manager or his designee before implementation of such changes.
- 6.5.2.12 Reports documenting each of the activities performed under Specifications 6.5.2.1 through 6.5.2.9 shall be maintained. Copies shall be provided to the Vice President-Nuclear Generation and the Safety Review and Audit Board.

6.5.3 Safety Review and Audit Board (SRAB)

Function

- 6.5.3.1 The Safety Review and Audit Board shall function to provide independent review and audit of designated activities in the areas of:
 - a. nuclear power plant operations
 - b. nuclear engineering
 - c. chemistry and radiochemistry
 - d. metallurgy
 - e. instrumentation and control
 - f. radiological safety
 - g. mechanical and electrical engineering
 - h. quality assurance practices
 - i. (other appropriate fields associated with the unique characteristics of the nuclear power plant)

Composition

6.5.3.2 The Safety Review and Audit Board shall be composed of the:

Chairman: Staff Engineer or Manager or Vice President
Member: Plant Manager or Designee
Member: Staff Engineer - Nuclear
Member: Staff Engineer - Mechanical or Electrical
Member: Consultant (See 6.5.3.4)

Alternates

6.5.3.3 Alternate members shall be appointed in writing by the SRAB Chairman to serve on a temporary basis; however, no more than two alternates shall participate in SRAB activities at any one time.

Consultants

6.5.3.4 Consultants shall be utilized as determined by the SRAB Chairman to provide expert advice to the SRAB.

Meeting Frequency

6.5.3.5 The SRAB shall meet at least once per six months.

Quorum

6.5.3.6 The quorum of the SRAB necessary for the performance of the SRAB review and audit functions of these Technical Specifications shall consist of the Chairman or the Chairman's designated alternate and at least three SRAB members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the facility.

Review

6.5.3.7 The SRAB shall review:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes in Technical Specifications or operating license.
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. ALL REPORTABLE EVENTS.
- h. Any indication of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- i. Reports and meeting minutes of the SORC.

Audits

- 6.5.3.8 Audits of facility activities shall be performed under the cognizance of the SRAB. These audits shall encompass:
- a. The conformance of facility operation to all provisions contained within the Technical Specifications and applicable license conditions at least once per year.
 - b. The performance, training and qualifications of the entire facility staff at least once per year.
 - c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per six months.
 - d. The performance of all activities required by the Quality Assurance Program to meet the criteria of Appendix "B", 10CFR50, at least once per two years.
 - e. The Facility Emergency Plan and implementing procedures at least once every 12 months.
 - f. The Facility Security Plan and implementing procedures at least once every 12 months.
 - g. The Facility Fire Protection Program and implementing procedures at least once per two years.
 - h. Any other area of facility operation considered appropriate by the SRAB or the Vice President - Nuclear Generation.
 - i. The radiological environmental monitoring program and the results thereof at least once per 12 months.
 - j. The Offsite Dose Calculation Manual and implementing procedures at least once per 24 months.
 - k. The Process Control Program and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months.

Authority

- 6.5.3.9 The SRAB shall report to and advise the Executive Vice President - Nuclear on those areas of responsibility specified in Section 6.5.3.7 and 6.5.3.8.

Records

- 6.5.3.10 Records of SRAB activities shall be prepared, approved and distributed as indicated below:
- a. Minutes of each SRAB meeting shall be prepared, approved and forwarded to the Executive Vice President - Nuclear within 30 days following each meeting.
 - b. Reports of reviews encompassed by Section 6.5.3.7 e, f, g and h above, shall be prepared, approved and forwarded to the Executive Vice President - Nuclear within 14 days following completion of the review.
 - c. Audit reports encompassed by Section 6.5.3.8 above, shall be forwarded to the Executive Vice President - Nuclear within 90 days following completion of the review.

6.6 Reportable Occurrence Action

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Sections 50.72 and 50.73 to 10CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the SORC and the results of this review submitted to the SRAB and the Vice President - Nuclear Generation.

6.7 Safety Limit Violation

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR 50.36(c)(1)(i) shall be complied with immediately.
- b. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Vice President - Nuclear Generation and the SRAB shall be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, within 30 days of the violation, and to the SRAB, and the Vice President - Nuclear Generation within 14 days.

6.8 Procedures

6.8.1 Written procedures and administrative policies shall be established, implemented and maintained that meet or exceed the requirements and recommendations of Sections 5.1 and 5.3 of ANSI N18.7-1972 and Appendix "A" of USAEC Regulatory Guide 1.33 except as provided in 6.8.2 and 6.8.3 below.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed and approved by the Plant Manager or designee prior to implementation and periodically as set forth in administrative procedures.

6.8 Procedures (Continued)

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed and approved by the Plant Manager or designee within 14 days of implementation.

6.9 Reporting Requirements

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following identified reports shall be submitted in accordance with 10 CFR 50.4.

6.9.1 Routine Reports

- a. Startup Report. A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant. The report shall address each of the tests identified in the FSAR and shall in general include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

6.12 High Radiation Area (Continued)

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rates in the area have been established and personnel have been made knowledgeable of them.
- c. An individual qualified in radiation protection, with a radiation dose rate monitoring device, who is responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the Manager Health Physics or designate in the Radiation Work Permit.

6.12.2 In addition to the requirements of 6.12.1 areas accessible to personnel with radiation levels such that a major portion of the body could receive in one hour a dose greater than 1000 mrem** shall be provided with locked doors*** to prevent unauthorized entry, and the hard keys or access provided by magnetic keycard shall be maintained under the administrative control of the Station Shift Supervisor or designate on duty and/or the Manager Health Physics or designate. Doors shall remain locked except during periods of access by personnel under an approved RWP which shall specify in accordance with site approved procedures accordingly, the dose rate levels in the immediate work area and the maximum allowable stay time for individuals in that area. In lieu of the stay time specification of the RWP, continuous surveillance, direct or remote, such as use of closed circuit TV cameras, may be made by personnel qualified in radiation protection procedures to provide positive exposure control over the activities within the area. For individual areas accessible to personnel with radiation levels such that a major portion of the body could receive in one hour a dose in excess of 1000 mrem** that are located within large areas, such as the drywell, where no enclosure exists for purposes of locking, and no enclosure can be reasonably constructed around the individual areas, then that area shall be roped off, conspicuously posted and a flashing light shall be activated as a warning device.

* by accessible passage and permanently fixed ladders

** measurement made at 18" from source of radioactivity

*** The requirement for locked doors to prevent unauthorized entry does not apply to areas which may temporarily exceed 1000 mrem/hr during the hydrogen water chemistry tests to be conducted during approximately a six-week period following startup from the spring 1986 refueling outage.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-410

NINE MILE POINT NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 25
License No. NPF-69

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated September 21, 1990, as supplemented October 4, 1990, and December 10, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-69 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 25 are hereby incorporated into this license. Niagara Mohawk Power Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 31, 1990

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 25 TO FACILITY OPERATING LICENSE NO. NPF-69

DOCKET NO. 50-410

Revise Appendix A as follows:

Remove Pages

6-1
6-3
6-7
6-8
6-9
6-10
6-11
6-14
6-15
6-16
6-25

Insert Pages

6-1
6-3
6-7
6-8
6-9
6-10
6-11
6-14
6-15
6-16
6-25

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Plant Manager shall be responsible for overall unit operation and shall delegate in writing the succession to this responsibility during the Plant Manager's absence.

6.1.2 The Station Shift Supervisor - Nuclear (or during the Supervisor's absence from the control room, a designated individual) shall be responsible for the control room command function. A management directive to this effect, signed by the Executive Vice President - Nuclear shall be reissued to all station personnel annually.

6.2 ORGANIZATION

6.2.1 Onsite and Offsite Organization

An onsite and an offsite organization shall be established for unit operation and corporate management. The onsite and offsite organization shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility and communication shall be established and defined from the highest management levels through intermediate levels to and including all operating organization positions. Those relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions or in equivalent forms of documentation. The organization charts shall be documented in the Final Safety Analysis Report, and the functional descriptions of departmental responsibilities and relationships and job descriptions for key personnel positions are documented in procedures.
- b. The Executive Vice President - Nuclear shall have corporate responsibility for overall plant nuclear safety. The Executive Vice President - Nuclear shall take any measures needed to assure acceptable performance of the staff in operating, maintaining, and providing technical support in the plant so that continued nuclear safety is assured.
- c. The Plant Manager shall have responsibility for overall unit operation and shall have control over those resources necessary for safe operation and maintenance of the plant.
- d. The persons responsible for the training, health physics and quality assurance functions may report to an appropriate manager onsite, but shall have direct access to responsible corporate management at a level where action appropriate to the mitigation of training, health physics and quality assurance concerns can be accomplished.

UNIT STAFF

6.2.2 The unit organization shall be subject to the following:

- a. Each on-duty shift shall be composed of at least the minimum shift crew shown in Table 6.2.2-1;

6.0 ADMINISTRATIVE CONTROLS

ORGANIZATION

UNIT STAFF

6.2.2.1 (Continued)

4. Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the Plant Manager, or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Controls shall be included in the procedures so that individual overtime shall be reviewed monthly by the Vice President - Nuclear Generation or a designee to assure that excessive hours have not been assigned. Routine deviation from the above guidelines is not authorized.

- j. The General Supervisor Operations, Supervisor Operations, Station Shift Supervisor Nuclear and Assistant Station Shift Supervisor Nuclear shall hold senior reactor operator licenses.

6.2.3 INDEPENDENT SAFETY ENGINEERING GROUP

FUNCTION

6.2.3.1 The Independent Safety Engineering Group (ISEG) shall function to examine unit operating characteristics, NRC issuances, industry advisories, Licensee Event Reports, and other sources of unit design and operating experience information, including units of similar design, which may indicate areas for improving unit safety. The ISEG shall make detailed recommendations for revised procedures, equipment modifications, maintenance activities, operations activities, or other means of improving unit safety to the Vice President - Nuclear Engineering.

COMPOSITION

6.2.3.2 The ISEG shall be composed of at least five, dedicated, full-time engineers located on site. Each shall have a bachelor's degree in engineering or related science and at least 2 years of professional level experience in his/her field, at least 1 year of which experience shall be in the nuclear field.

RESPONSIBILITIES

6.2.3.3 The principal function of the ISEG is to examine plant operating characteristics and the various NRC and industry licensing and service advisories, and to recommend areas for improving plant operations or safety. The ISEG will perform independent review of plant activities, including maintenance, modifications, operational concerns, and analysis and make recommendations to the Vice President - Nuclear Engineering.

ADMINISTRATIVE CONTROLS

ORGANIZATION

RECORDS

5.2.3.4 Records of activities performed by the ISEG shall be prepared, maintained, and forwarded each calendar month to the Vice President - Nuclear Engineering.

5.2.4 ASSISTANT STATION SHIFT SUPERVISOR/SHIFT TECHNICAL ADVISOR

Normally the Assistant Station Shift Supervisor (ASSS) shall function in a dual role (SRO/STA) and assume the duties of the Shift Technical Advisor (STA) when the Emergency Plan is activated in OPERATIONAL CONDITIONS 1,2 or 3. The STA shall provide advisory technical support to the Shift Supervisor in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to safe operation of the unit. The ASSS, when fulfilling the role of the STA, shall have a bachelor's degree in a physical science, engineering, or a PE license issued by examination, and shall have received specific training in the response and analysis of the unit for transients and accidents, and in unit design and layout, including the capabilities of instrumentation and controls in the control room.

6.3 FACILITY STAFF QUALIFICATIONS

Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions, except for the Manager Radiation Protection who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975. The Licensed Operators and Senior Operators shall also meet or exceed the minimum qualifications of the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980, NRC letter to all licensees.

6.4 TRAINING

A retraining and replacement training program for the unit staff shall be maintained under the direction of the Manager Training, shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI/ANS 3.1-1978 and Appendix A of 10 CFR 55, and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980, NRC letter to all licensees, and shall include familiarization with relevant industry operational experience. A training program for the Fire Brigade shall be maintained under the direction of the Manager Training and the Supervisor - Fire Protection Nuclear and shall meet or exceed the requirements of Appendix R to 10 CFR 50.

ADMINISTRATIVE CONTROLS

6.5 REVIEW AND AUDIT

6.5.1 STATION OPERATIONS REVIEW COMMITTEE

FUNCTION

6.5.1.1 The Station Operations Review Committee (SORC) shall function to advise the Plant Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The SORC shall be composed of the

Chairman:	Plant Manager
Member:	Manager Operations
Member:	Manager Maintenance
Member:	Manager Technical Support
Member:	Manager Health Physics
Member:	Manager Chemistry
Member:	Manager Radiation Protection
Member:	General Supervisor Instrument and Control Maintenance
Member:	General Supervisor System Engineering
Member:	General Supervisor Operations
Member:	Supervisor Reactor Engineering

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in SORC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The SORC shall meet at least once every calendar month and as convened by the SORC Chairman or a designated alternate.

QUORUM

6.5.1.5 The quorum of the SORC necessary for the performance of the SORC responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or a designated alternate and five members including alternates.

RESPONSIBILITIES

6.5.1.6 The SORC shall be responsible for:

- a. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence, to the Vice President - Nuclear Generation and to the Safety Review and Audit Board;

ADMINISTRATIVE CONTROLS

6.5 REVIEW AND AUDIT

6.5.1 STATION OPERATIONS REVIEW COMMITTEE

6.5.1.6 (Continued)

- b. Review of all REPORTABLE EVENTS;
- c. Review of unit operations to detect potential hazards to nuclear safety;
- d. Performance of special reviews, investigations, or analyses and reports thereon as requested by the Plant Manager or the Safety Review and Audit Board;
- e. Safety evaluations and analyses resulting from technical review and control activities 6.5.2.1, 6.5.2.2, 6.5.2.3, and 6.5.2.5.

DUTIES

6.5.1.7 The SORC shall:

- a. Render determinations in writing with regard to whether or not each item considered under Specification 6.5.1.6.a through e constitutes an unreviewed safety question.
- b. Provide written notification within 24 hours to the Vice President - Nuclear Generation and the Safety Review and Audit Board of disagreement between the SORC and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1.

RECORDS

6.5.1.8 The SORC shall maintain written minutes of each SORC meeting that at a minimum, document the result of all SORC activities performed under the responsibilities and authority provisions of these Technical Specifications. Copies shall be provided to the Vice President - Nuclear Generation and the Safety Review and Audit Board.

6.5.2 TECHNICAL REVIEW AND CONTROL ACTIVITIES

6.5.2.1 Each procedure and program required by Specification 6.8 and other procedures that affect nuclear safety, and changes thereto, shall be prepared by a qualified individual/organization. Each such procedure, and changes thereto, shall be reviewed by an individual/group other than the individual/group that prepared the procedure, or changes thereto, but who may be from the same organization as the individual/group that prepared the procedure, or changes thereto. Approval of procedures and programs and changes thereto and their safety evaluations, shall be controlled by administrative procedures.

6.5.2.2 Proposed changes to the Technical Specifications shall be prepared by a qualified individual/organization. The preparation of each proposed Technical Specifications change shall be reviewed by an individual/group other than the individual/group that prepared the proposed change, but who may be from the same organization as the individual/group that prepared the proposed change.

ADMINISTRATIVE CONTROLS

REVIEW AND AUDIT

TECHNICAL REVIEW AND CONTROL ACTIVITIES

6.5.2.2 (Continued)

Proposed changes to the Technical Specifications shall be approved by the Plant Manager.

6.5.2.3 Proposed modifications to unit structures, systems, and components that affect nuclear safety shall be designed by a qualified individual/organization. Each such modification shall be reviewed by an individual/group other than the individual/group that designed the modification, but who may be from the same organization as the individual/group that designed the modification. Proposed modifications to structures, systems, and components and the safety evaluations shall be approved before implementation by the Plant Manager, or the Manager Technical Support, as previously designated by the Plant Manager.

6.5.2.4 Individuals responsible for reviews performed in accordance with Specifications 6.5.2.1, 6.5.2.2, and 6.5.2.3 shall be members of the station supervisory staff, previously designated by the Plant Manager to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary such review shall be performed by the appropriate designated station review personnel.

6.5.2.5 Proposed tests and experiments that affect station nuclear safety and are not addressed in the FSAR or Technical Specifications and their safety evaluations shall be reviewed by the Plant Manager, or the Manager Technical Support, as previously designated by the Plant Manager.

6.5.2.6 The Plant Manager shall assure the performance of special reviews and investigations, and the preparation and submittal of reports thereon, as requested by the Vice President - Nuclear Generation.

6.5.2.7 The facility security program and implementing procedures shall be reviewed at least every 12 months. Recommended changes shall be approved by the Plant Manager and transmitted to the Vice President - Nuclear Generation, and to the Chairman of the Safety Review and Audit Board.

6.5.2.8 The facility emergency plan and implementing procedures shall be reviewed at least every 12 months. Recommended changes shall be approved by the Plant Manager and transmitted to the Vice President - Nuclear Generation and to the Chairman of the Safety Review and Audit Board.

ADMINISTRATIVE CONTROL

REVIEW AND AUDIT

TECHNICAL REVIEW AND CONTROL ACTIVITIES

6.5.2.9 The Plant Manager shall assure the performance of a review by a qualified individual/organization of changes to the Radiological Waste Treatment systems.

6.5.2.10 Review of any accidental, unplanned, or uncontrolled radioactive release, including the preparation of reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President - Nuclear Generation and to the Safety Review and Audit Board.

6.5.2.11 Review of changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL. Approval of any changes shall be made by the Plant Manager or a designee before implementation of such changes.

6.5.2.12 Reports documenting each of the activities performed under Specifications 6.5.2.1 through 6.5.2.9 shall be maintained. Copies shall be provided to the Vice President - Nuclear Generation and the Safety Review and Audit Board.

6.5.3 SAFETY REVIEW AND AUDIT BOARD

FUNCTION

6.5.3.1 The Safety Review and Audit Board (SRAB) shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations
- b. Nuclear engineering
- c. Chemistry and radiochemistry
- d. Metallurgy
- e. Instrumentation and control
- f. Radiological safety
- g. Mechanical and electrical engineering
- h. Quality assurance practices and
- i. Other appropriate fields associated with the unique characteristics of the nuclear power plant

The SRAB shall report to and advise the Executive Vice President - Nuclear on those areas of responsibility in Specifications 6.5.3.7 and 6.5.3.8.

COMPOSITION

6.5.3.2 The SRAB shall be composed of the:

- Chairman: Vice President, Manager or Staff Engineer
- Member: Plant Manager or Designee
- Member: Staff Engineer - Nuclear
- Member: Staff Engineer - Mechanical or Electrical
- Member: Consultant (Specification 6.5.3.4)

ADMINISTRATIVE CONTROLS

REVIEW AND AUDIT

SAFETY REVIEW AND AUDIT BOARD

AUDITS

6.5.3.8 (Continued)

- j. Any other area of unit operation considered appropriate by the SRAB or the Vice President - Nuclear Generation.
- k. The Fire Protection Program and implementing procedures at least once per 24 months.
- l. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- m. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 36 months.

AUTHORITY

6.5.3.9 The SRAB shall report to and advise the Executive Vice President - Nuclear on those areas of responsibility specified in Sections 6.5.3.7 and 6.5.3.8.

RECORDS

6.5.3.10 Records of SRAB activities shall be prepared, approved, and distributed as indicated below:

- a. Minutes of each SRAB meeting shall be prepared, approved, and forwarded to the Executive Vice President - Nuclear within 14 days following each meeting.
- b. Reports of reviews encompassed by Specification 6.5.3.7b, e, g, h shall be prepared, approved, and forwarded to the Executive Vice President - Nuclear within 14 days following completion of the review.
- c. Audit reports encompassed by Specification 6.5.3.8 shall be forwarded to the Executive Vice President - Nuclear and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

ADMINISTRATIVE CONTROLS

6.6 REPORTABLE EVENT ACTION

The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of 10 CFR 50.72 and 10 CFR 50.73, and
- b. Each REPORTABLE EVENT shall be reviewed by the SORC, and the results of this review shall be submitted to the SRAB and the Vice President - Nuclear Generation.

6.7 SAFETY LIMIT VIOLATION

The following actions shall be taken in the event a Safety Limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Vice President - Nuclear Generation and the SRAB shall be notified within 24 hours.
- b. A Safety Limit Violation Report shall be prepared in accordance with 10 CFR 50.73. The report shall be reviewed by the SORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon unit components, systems, or structures, and (3) corrective action taken to prevent recurrence.
- c. The Safety Limit Violation Report shall be submitted to the Commission within 30 days of the violation, and to the SRAB, and the Vice President - Nuclear Generation within 14 days.
- d. Critical operation of the unit shall not be resumed until authorized by the Commission.

6.8 PROCEDURES AND PROGRAMS

PROCEDURES

6.8.1 Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory guide 1.33, Revision 2, February 1978
- b. The applicable procedures required to implement the requirements of NUREG-0737
- c. Refueling operations

ADMINISTRATIVE CONTROLS

6.8 PROCEDURES AND PROGRAMS

PROCEDURES

6.8.1 (Continued)

- d. Surveillance and test activities of safety-related equipment
- e. Security Plan implementation
- f. Emergency Plan implementation
- g. Fire Protection Program implementation
- h. PROCESS CONTROL PROGRAM implementation
- i. OFFSITE DOES CALCULATION MANUAL implementation and
- j. Quality Assurance for effluent and environmental monitoring.

REVIEW AND APPROVAL

6.8.2 Each procedure of Specification 6.8.1 and changes thereto, shall be approved by the Plant Manager based on the recommendations of SORC* before implementation and reviewed periodically as set forth in administrative procedures.

TEMPORARY CHANGES

6.8.3 Temporary changes to procedures of Specification 6.8.1 may be made provided:

- a. The intent of the original procedure is not altered;
- b. The change is approved by two members of the unit management staff, at least one of whom holds a Senior Operator license on the unit affected; and
- c. The change is documented, approved by the Plant Manager based on the recommendations of the SORC* within 14 days of implementation.

*SORC recommendations to the Plant Manager are to be based on SORC responsibilities as identified in Specification 6.5.1.7.a.

ADMINISTRATIVE CONTROLS

6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by 10 CFR 20.203 (c) (2), each high radiation area in which the intensity of radiation is greater than 100 mrem/hr* but less than 1000 mrem/hr* shall be barricaded and conspicuously posted as a high radiation area, and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP)**. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area
- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate levels in the area have been established and personnel have been made knowledgeable of them
- c. An individual qualified in radiation protection (i.e., qualified in radiation protection procedures) with a radiation dose rate monitoring device who is responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the Manager Health Physics or the Manager Health Physic's designee in the RWP

6.12.2 In addition to the requirements of Specification 6.12.1, areas accessible to personnel with radiation levels such that a major portion of the body could receive in 1 hour a dose greater than 1000 mrem* shall be provided with locked doors to prevent unauthorized entry, and the keyed access shall be maintained under the administrative control of the Station Shift Supervisor or the designee on duty and/or the Manager Health Physics or designee. Doors shall remain locked except during periods of access by personnel under an approved RWP which shall specify the dose rate levels in the immediate work area and the maximum allowable stay time for individuals in that area. For individual areas accessible to personnel with such radiation levels that a major portion of the body could receive in 1 hour a dose in excess of 1000 mrem* that are located within large areas, such as the drywell, where no enclosure exists that can be locked, and no enclosure can be reasonably constructed around the individual areas, then that area shall be roped off.

*Measurements made at 18 inches from the source of radioactivity.

**Health Physics personnel or personnel escorted by health physics personnel shall be exempt from the RWP issuance requirement during the performance of their assigned radiation protection duties, provided they are otherwise following plant radiation protection procedures for entry into high radiation areas.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENTS NOS. 120 AND 25 TO FACILITY OPERATING
LICENSE NOS. DPR-63 AND NPF-69
NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT NUCLEAR STATION, UNIT NOS. 1 AND 2
DOCKET NOS. 50-220 AND 50-410

1.0 INTRODUCTION

By letters dated September 21, 1990, as supplemented October 4, 1990, for Nine Mile Point Units 1 and 2 (NMP-1 and NMP-2), November 30, 1990, for NMP-1, and December 10, 1990, for NMP-2, Niagara Mohawk Power Corporation, the licensee, requested changes to the Administrative Controls Section of the Technical Specifications, Section 6.0. The proposed amendment would reflect management changes due to the reorganization of the Nuclear Division. A Temporary Waiver of Compliance was issued on September 28, 1990, which permitted the licensee to implement the reorganization on October 1, 1990. Following is a brief description and our evaluation of the requested changes.

2.0 EVALUATION

General - The licensee has made revisions to the Corporate and the plant organizations that are responsible for the operation and support of NMP-1 and NMP-2. We have reviewed the title changes made in Sections 6.1 - Responsibility, 6.2.1 - Onsite and Offsite Organization, 6.2.2 - Unit Staff, 6.4 - Training, 6.5.2 - Technical Review and Control Activities, 6.6 - Reportable Event Action, 6.7 - Safety Limit Violation, 6.8 - Procedures and Programs, and 6.12 - High Radiation Area. We find these changes adequately reflect the revised organization and that the level of responsibility associated with the revised positions and titles are commensurate with the previous organization and are acceptable.

Section 6.5.1 - Station Operations Review Committee (SORC) - The title of this group has been changed from the Site Operations Review Committee to the Station Operations Review Committee. For Unit 1 the membership of the SORC has been increased from seven to eleven members to be consistent with Unit 2. In addition, the quorum for both units has been increased to six such as to constitute a majority of the members. Committee members have also been changed to reflect revised responsibilities and titles.

We find these changes acceptable as they reflect the revised organization and meet the applicable acceptance criteria of Section 13.4 of NUREG 0800, the Standard Review Plan.

Section 6.5.3 - Safety Review and Audit Board (SRAB) - Niagara Mohawk Power Corporation has requested to revise the composition of the SRAB by deleting the position of Staff Engineer-Environmental. In addition, for Unit 1, the licensee has revised the quorum to state that it shall consist of the Chairman or his alternate and at least three members in place of the Chairman or his alternate and a majority of SRAB members. Also title changes have been made consistent with the revised organization.

We find these requested changes acceptable as they reflect the revised organization and meet the acceptance criteria of Section 13.4 of NUREG-0800, the Standard Review Plan.

The licensee, by letters dated November 30, 1990, and December 10, 1990, provided supplemental information for NMP-1 and NMP-2, respectively. These supplemental submittals did not affect the proposed TS change noticed in the Federal Register on October 19, 1990 (55 FR 42524) and did not affect the staff's proposed determination that no significant hazards would result from these changes.

3.0 SUMMARY

The staff has reviewed the proposed changes to Technical Specifications Section 6.0 and finds these changes to be acceptable.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change in administrative procedures and requirements. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need to be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: December 31, 1990

PRINCIPAL CONTRIBUTOR:
F. Allenspach