

January 11, 1991

Docket No. 50-220

Mr. B. Ralph Sylvia
Executive Vice President, Nuclear
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Dear Mr. Sylvia:

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SUBJECT: ISSUANCE OF AMENDMENT FOR NINE MILE 1 ON THE REMOVAL OF THE
3.25 LIMIT (TAC NO. 76073)

The Commission has issued the enclosed Amendment No. 121 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station Unit No. 1 (NMP-1). The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated February 26, 1990, as superseded October 26, 1990, and supplemented November 30, 1990.

This amendment removes a restriction that limits the combined time interval for three consecutive surveillances to less than 3.25 times the specified interval, and also adds the Bases applicable to a new Section 4.0.1, "Surveillance Intervals." These changes are consistent with the guidance provided in Generic Letter 89-14, "Line Item Improvements in Technical Specifications - Removal of the 3.25 Limit on Extending Surveillance Intervals."

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

Donald S. Brinkman, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.121 to DPR-63
2. Safety Evaluation

cc: w/enclosures
See next page

PDI-1:LA	PDI-1:PE	PDI-1:PM	NRR/OTSP	OGC	PDI-1D
CVogan	DOudinot:av1	DBrinkman	JCalvo	Bemis	RACapra
12/27/90	12/28/90	12/28/90	12/31/90	1/2/90	01/11/90

DOCUMENT NAME: AMENDMENT NO. 76073

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Mr. B. Ralph Sylvia
Niagara Mohawk Power Corporation

Nine Mile Point Nuclear Station,
Unit No. 1

cc:

Mr. Mark J. Wetterhahn, Esquire
Bishop, Cook, Purcell & Reynolds
1400 L. Street, N.W.
Washington, D. C. 20005-3502

Mr. Kim Dahlberg
Unit 1 Station Superintendent
Nine Mile Point Nuclear Station
Post Office Box 32
Lycoming, New York 13093

Supervisor
Town of Scriba
R. D. #4
Oswego, New York 13126

Mr. Peter E. Francisco, Licensing
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Mr. Joseph F. Firlit
Vice President - Nuclear Generation
Niagara Mohawk Power Corporation
Nine Mile Point Nuclear Station
Post Office Box 32
Lycoming, New York 13093

Charlie Donaldson, Esquire
Assistant Attorney General
New York Department of Law
120 Broadway
New York, New York 10271

Resident Inspector
U.S. Nuclear Regulatory Commission
Post Office Box 126
Lycoming, New York 13093

Mr. Paul D. Eddy
State of New York
Department of Public Service
Power Division, System Operations
3 Empire State Plaza
Albany, New York 12223

Mr. Gary D. Wilson, Esquire
Niagara Mohawk Power Corporation
300 Erie Boulevard West
Syracuse, New York 13202

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

Ms. Donna Ross
New York State Energy Office
2 Empire State Plaza
16th Floor
Albany, New York 12223



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

January 11, 1991

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Niagara Mohawk Power Corporation
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This amendment removes a restriction that limits the combined time interval for three consecutive surveillances to less than 3.25 times the specified interval, and also adds the Bases applicable to a new Section 4.0.1, "Surveillance Intervals." These changes are consistent with the guidance provided in Generic Letter 89-14, "Line Item Improvements in Technical Specifications - Removal of the 3.25 Limit on Extending Surveillance Intervals."

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Donald S. Brinkman".

Donald S. Brinkman, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.121 to DPR-63
2. Safety Evaluation

cc: w/enclosures
See next page



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 121
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated February 26, 1990, as superseded October 26, 1990 and supplemented November 30, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 121, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 11, 1991

ATTACHMENT TO LICENSE AMENDMENT
AMENDMENT NO. 121 TO FACILITY OPERATING LICENSE NO. DPR-63
DOCKET NO. 50-220

Revise Appendix A as follows:

Remove Pages

4
25
-

Insert Pages

4
25
25a

1.12 Reactor Building Integrity

Reactor Building Integrity means that the reactor building is closed and the following conditions are met:

- a. At least one door at each access opening is closed.
- b. The standby gas treatment system is operable.
- c. All Reactor Building ventilation system automatic isolation valves are operable or are secured in the closed position.

1.13 Core Alteration

A core alteration is the addition, removal, relocation, or other manual movement of fuel or controls in the reactor core. Control rod movement with the control rod drive hydraulic system is not considered to be a core alteration.

1.14 Rated Flux

Rated flux is the neutron flux that corresponds to a steady-state power level of 1850 thermal megawatts. The use of the term 100 percent also refers to the 1850 thermal megawatt power level.

1.15 Surveillance

Surveillance means that process whereby systems and components which are essential to plant nuclear safety during all modes of operation or which are necessary to prevent or mitigate the consequences of incidents are checked, tested, calibrated and/or inspected, as warranted, to verify performance and availability at optimum intervals.

3.0 LIMITING CONDITIONS FOR OPERATION

3.0.1 OPERABILITY REQUIREMENTS

When a system, subsystem, train, component or device is determined to be inoperable solely because its emergency power source is inoperable, or solely because its normal power source is inoperable, it may be considered operable for the purpose of satisfying the requirements of its applicable Limiting Condition for Operation, provided: (1) its corresponding normal or emergency power source is operable; and (2) all of its redundant system(s), subsystem(s), train(s), component(s) and device(s) are operable, or likewise satisfy the requirements of this specification. Unless both conditions (1) and (2) are satisfied, the unit shall be placed in a condition stated in the individual specification.

In the event a Limiting Condition for Operation and/or associated surveillance requirements cannot be satisfied because of circumstances in excess of those addressed in the specification, the unit shall be placed in a condition consistent with the individual specification unless corrective measures are completed that permit operation under the permissible surveillance requirements for the specified time interval as measured from initial discovery or until the reactor is placed in an operational condition in which the specification is not applicable.

4.0 SURVEILLANCE REQUIREMENTS

4.0.1 SURVEILLANCE INTERVALS

Each Surveillance Requirement shall be performed within the specified surveillance interval with a maximum allowable extension not to exceed 25 percent of the specified surveillance interval.

BASES

Specification 4.0.1 establishes the limit for which the specified time interval for Surveillance Requirements may be extended. It permits an allowable extension of the normal surveillance interval to facilitate surveillance scheduling and consideration of plant operating conditions that may not be suitable for conducting the surveillance; e.g., transient conditions or other ongoing surveillance or maintenance activities. It also provides flexibility to accommodate the length of a fuel cycle for surveillances that are performed at each refueling outage and are specified with a 24 month surveillance interval. It is not intended that this provision be used repeatedly as a convenience to extend surveillance intervals beyond that specified for surveillances that are not performed during refueling outages. The limitation of Specification 4.0.1 is based on engineering judgement and the recognition that the most probable result of any particular surveillance being performed is the verification of conformance with the Surveillance Requirements. This provision is sufficient to ensure that the reliability ensured through surveillance activities is not significantly degraded beyond that obtained from the specified surveillance interval.

3.1.0 FUEL CLADDING

A) GENERAL APPLICABILITY

Applies to the power level regulation, control rod system, liquid poison system, emergency cooling system, and core spray system. LCO's for the minimum allowable circuits corresponding to the LS3 settings are included in the Reactor Protection System LCO (3.6.2).

B) GENERAL OBJECTIVE

LIMITING CONDITIONS FOR OPERATION - To define the lowest functional capability or performance level of the systems and associated components which will assure the integrity of the fuel cladding as a barrier against the release of radioactivity.

SURVEILLANCE REQUIREMENTS - To define the tests or inspections required to assure the functional capability or performance level of the required systems or components.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 121 TO FACILITY OPERATING LICENSE NO. DPR-63
NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT NUCLEAR STATION, UNIT NO. 1
DOCKET NO. 50-220

INTRODUCTION

By letter dated February 26, 1990, as superseded October 26, 1990, and supplemented November 30, 1990, Niagara Mohawk Power Corporation, the licensee, proposed changes to the Technical Specifications (TS) for Nine Mile Point Nuclear Station, Unit No. 1 (NMP-1). The proposed changes would remove the provision of Definition 1.15 that limits the combined time interval for three consecutive surveillances to less than 3.25 times the specified interval. This change would also revise and relocate to a new section, 4.0.1 Surveillance Intervals, the maximum allowable extension to surveillance intervals which is currently contained in Section 1.15 - Definition. The Bases applicable to the new Section 4.0.1 would also be added. Guidance on this proposed change to TS was provided to all power reactor licensees and applicants by Generic Letter 89-14 dated August 21, 1989.

EVALUATION

Specification 1.15 includes the provision that allows a surveillance interval to be extended by 25 percent of the specified time interval. This extension provides flexibility for scheduling the performance of surveillances and to permit consideration of plant operating conditions that may not be suitable for conducting a surveillance at the specified time intervals. Such operating conditions include transient plant operation or ongoing surveillance or maintenance activities. Specification 1.15 further limits the allowance for extending surveillance intervals by requiring that the combined time interval for any three consecutive surveillances not exceed 3.25 times the specified time interval. The purpose of this provision is to assure that surveillances are not extended repeatedly as an operational convenience to provide an overall increase in the surveillance interval.

Experience has shown that the 24 month surveillance interval, with the provision to extend it by 25 percent, is usually sufficient to accommodate normal variations in the length of a fuel cycle. However, the NRC staff has routinely granted requests for one-time exceptions to the 3.25 limit on

extending refueling surveillances because the risk to safety is low in contrast to the alternative of a forced shutdown to perform these surveillances. Therefore, the 3.25 limitation on extending surveillances has not been a practical limit on the use of the 25-percent allowance for extending surveillances that are performed on a refueling outage basis.

Extending surveillance intervals during plant operation can also result in a benefit to safety when a scheduled surveillance is due at a time that is not suitable for conducting the surveillance. This may occur when transient plant operating conditions exist or when safety systems are out-of-service for maintenance or other surveillance activities. In such cases, the benefit to safety of extending a surveillance interval would exceed any safety benefit derived by limiting the use of the 25-percent allowance to extend a surveillance. Furthermore, there is the administrative burden associated with tracking the use of the 25-percent allowance to ensure compliance with the 3.25 limit.

In view of these findings, the staff concluded that the 3.25 limit should be removed for all surveillances because its removal will have an overall positive effect on safety. The guidance provided in Generic Letter 89-14 included the following change to this specification and removes the 3.25 limit on three consecutive surveillances with the following statement which will be relocated, for consistency, in a newly created section, 4.0.1, Surveillance Intervals:

"Each Surveillance Requirement shall be performed within the specified surveillance interval with a maximum allowable extension not to exceed 25 percent of the specified surveillance interval."

Also, the Bases applicable to Specification 4.0.1 will be added. These Bases note that it is not the intent of the allowance for extending surveillance intervals that it be used repeatedly merely as an operational convenience to extend surveillance intervals beyond that specified.

In addition, for consistency with technical specifications nomenclature, the section entitled, "Operability Requirement" on page 25 will be given the number 3.0.1, and a new heading, 4.0, Surveillance Requirements, will contain the newly created Section 4.0.1 and associated Bases. A new page, 25a, will be added due to space limitations on page 25.

The licensee, by letter dated November 30, 1990, provided supplemental information. This supplemental submittal did not affect the proposed TS change noticed in the Federal Register and did not affect the staff's proposed determination that no significant hazards would result from these changes. The supplemental submittal included the following corrections per staff request: Section 3.0.2 was renumbered as 4.0 and a heading, 4.0.1, Surveillance Intervals, was added.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in a requirement with respect to the installation or use of the facility components located within the restricted areas as defined in 10 CFR Part 20. The staff has determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: January 11, 1991

PRINCIPAL CONTRIBUTOR:

T. Dunning