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Engineering managers are required by NEP-121 to determine if new, revised, or temporary changes to procedures affect job functions of their personnel. Managers will communicate change information appropriately and provide documentation of any training conducted to the Engineering Training Coordinator.

NO ACKNOWLEDGEMENT REQUIRED

A045

(End of Page)

Rev. 67

Effective Date 4/4/01

EMERGENCY PLAN IMPLEMENTING PROCEDURE

EM-202

FLORIDA POWER CORPORATION

CRYSTAL RIVER UNIT 3

DUTIES OF THE EMERGENCY COORDINATOR

APPROVED BY: Procedure Owner

(SIGNATURE ON FILE)

DATE: 4/4/01

PROCEDURE OWNER: Radiological Emergency Planning

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ENCLOSURES

1.0 PURPOSE

Provides instructions and guidelines used by the Emergency Coordinator during initiation of the Radiological Emergency Response Plan. Specific guidelines include emergency classification, reporting and notification requirements, and protective action recommendations for non-essential Energy Complex personnel and the general public.

2.0 REFERENCES

2.1 DEVELOPMENTAL REFERENCES

2.1.1 10 CFR 50.47, Emergency Plans

2.1.2 10 CFR 50, Appendix E, Emergency Planning and Preparedness for Production and Utilization Facilities

2.1.3 10 CFR 50.72, Immediate Notification Requirements for Operating Nuclear Power Reactors.

2.1.4 CR-3 Severe Accident Guideline

2.1.5 Emergency Action Level Bases Manual

2.1.6 Manual of Protective Action Guides And Protective Actions for Nuclear Incidents, EPA-400-R-92-001, Environmental Protection Agency (October, 1991).

2.1.7 NEI 91-04, Revision 1, Severe Accident Issue Closure Guidelines

2.1.8 NEI 97-03, Methodology for Development of Emergency Action Levels

2.1.9 NUREG-0654, Criteria for Preparation and Evaluation of Radiological Emergency Response Plans and Preparedness in Support of Nuclear Power Plants

2.1.10 Off-Site Dose Calculation Manual.

2.1.11 Radiological Emergency Response Plan

2.1.12 Safety Evaluation of FPC proposed EAL changes for CR-3 (TAC No. MA2231), NRC to FPC letter 3N0299-02.

3.0 PERSONNEL INDOCTRINATION

NOTE: **A Safety Assessment was performed for this procedure. A determination was made that this procedure is outside the scope of 10 CFR 50.59.**

3.1 DEFINITIONS

3.1.1 **Bomb** - An explosive device suspected of having sufficient force to damage plant systems or structures. (See EXPLOSION)

3.1.2 **Civil Disturbance**: - A group of ten (10) or more people violently protesting station operations or activities at the site. A civil disturbance is considered violent when force has been used in an attempt to injure site personnel or damage plant property.

3.1.3 **Committed Dose Equivalent (CDE)** - Dose to an organ due to the intake of radioactive materials.

3.1.4 **Deep Dose Equivalent (DDE)** - External whole body dose.

3.1.5 **Emergency Action Level (EAL)** - A pre-determined, observable threshold for plant conditions that places the plant in a given emergency classification.

3.1.6 **Emergency Classification** - A system of classification in which emergency occurrences are categorized according to specific protective action levels. The four emergency classification are:

3.1.6.1 **Unusual Event** - This classification refers to any event(s), in process or having occurred, indicating a potential degradation of the level of safety of the plant. No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety occurs. This classification brings the operating staff to a state of readiness if escalation to a more severe action level classification occurs.

3.1.6.2 **Alert** - This classification refers to event(s) that are in process, or have occurred, involving an actual or potentially substantial degradation of the level of safety of the plant. Any releases are expected to be limited to small fractions of the EPA Protective Action Guideline exposure levels. The Technical Support Center (TSC) is staffed and assembly and accountability are performed at local assembly areas.

3.1.6.3 **Site Area Emergency** - This classification refers to event(s) that are in process, or have occurred, involving actual or likely major failures of plant functions needed for the protection of the public. Any releases are not expected to result in exposure levels which exceed EPA Protective Action Guideline exposure levels at the SITE BOUNDARY. The TSC and the Emergency Operations Facility (EOF) are staffed and radiation monitoring teams may be dispatched. Protected Area evacuation and accountability is performed at CR-3. Assembly and accountability is performed at Units 1, 2, 4 and 5.

3.1.6.4 **General Emergency** - This classification refers to event(s) that are in process, or have occurred, involving actual or imminent substantial core degradation or nuclear fuel melting with the potential for loss of containment integrity. Releases can be reasonably expected to exceed EPA Protective Action Guidelines exposure levels at the SITE BOUNDARY. This classification initiates predetermined protective actions for the public, provides continuous assessment of information from on-site and off-site measurements, initiates additional measures indicated by the event, and provides current information and consultation with off-site authorities and the public. The Emergency Coordinator will probably decide to evacuate the Energy Complex.

3.1.7 **Emergency Coordinator (EC)** - The position with the highest level of authority for the CR-3 Emergency Organization and on-site emergency activities. This position is held by the Director Nuclear Plant Operations or designated alternate. The Nuclear Shift Manager assumes the position until the Director Nuclear Plant Operations or designated alternate arrives to assume Emergency Coordinator responsibilities.

3.1.8 **Emergency Response Data System (ERDS)** - NRC requirement {(10 CFR 50.72(a)(4)} to have the ability to acquire data from nuclear power plants in the event of an emergency at the plant. ERDS is a direct real-time transfer of data from FPC to NRC. Once initiated, ERDS operates automatically.

3.1.9 **Explosion** - A rapid, violent, unconfined combustion, or a catastrophic failure of pressurized equipment that imparts energy of sufficient force to potentially damage permanent structures, systems, or components.

3.1.10 **Extortion** - An attempt to cause an action at CR-3 by threat of force. Bomb threats that are unsubstantiated are not included in this definition.

3.1.11 **Fire** - Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do not constitute fires. Observation of flame is preferred but is NOT required if large quantities of smoke and heat are observed.

3.1.12 **Hostage** - A person or object held as leverage against the station to ensure that demands will be met by CR-3.

3.1.13 **Incident Report** - A report of the actual scenario of the emergency, the identified cause(s) of the emergency, and the radiological history of the emergency, including released quantities, existing radiological activity, abnormal doses for emergency worker and population doses.

3.1.14 **Intrusion/Intruder** - Suspected hostile individual present in a protected area without authorization.

3.1.15 **Local Assembly Area** - A pre-designated area personnel report for organization, roll call, and supervision following an "Alert" emergency classification.

3.1.16 **Main Assembly Area** - The place personnel report for organization and supervision following an evacuation of the CR-3 protected area. The main Assembly Area is the Site Administration Building Auditorium.

3.1.17 **Protected Area** - All areas within the CR-3 security perimeter fence that require badged authorization for entry.

3.1.18 **Protective Action Recommendations** - Emergency measures recommended for purposes of preventing or minimizing radiological exposures to Generating Complex personnel or members of the general public. Protective Action Recommendations are made using all available data, primarily plant conditions. Off-site dose projections and/or field survey results can also be factored in to Protective Action Recommendations if confidence in their accuracy is high (monitored release, confirmed field survey results).

3.1.19 **Release (State Form)** - Any of the following:

- Any increase in count rate on an effluent monitor that is a direct result of an event that has initiated an emergency declaration;
  OR
- Radioactivity detected by environmental monitoring;
  OR
- Radioactivity escaping unmonitored from the plant. (NOTE: Design Basis Leakage or other suspected leakage should not be categorized as a release until confirmed by environmental monitoring.)

3.1.20 **Release, Unplanned (NRC Event Notification Worksheet)** - Release is not authorized by a Release Permit or exceeds the conditions (e.g., minimum dilution flow, maximum discharge flow, alarm setpoints, etc.) on the applicable permit.

3.1.21 **Sabotage** - Deliberate damage, mis-alignment, or mis-operation of safe shutdown equipment with the intent to render the equipment unavailable.

3.1.22 **Safe Shutdown Equipment** - Equipment necessary to achieve and maintain the reactor subcritical with controlled decay heat removal.

3.1.23 **Security Emergency** - A Security related situation that poses a clear or imminent threat or danger to the plant and calls for prompt response and/or is confirmed as an act of sabotage.

3.1.24 **Severe Accident** - An accident beyond that assumed in the CR-3 design and licensing basis that results in catastrophic fuel rod failure, core degradation, and fission product release into the Rx vessel, Reactor Building, or the environment.

3.1.25 **Significant Transient**: An UNPLANNED event involving one or more of the following:

(1) Automatic turbine trip at >25% reactor thermal power
(2) Electrical load rejection >25% full electrical load
(3) Plant runback
(4) Reactor trip
(5) Safety injection system actuation
(6) >10% thermal power oscillations
(7) Loss of decay heat removal in Mode 4 ("Significant Transient" is not used in any Mode 5 or 6 EAL)

3.1.26 **Site Boundary**: - That area, including the PROTECTED AREA, that extends 4400 ft. or 0.83 miles in a circle around the Reactor Building. Also referred to as the Owner Controlled Area.

3.1.27 **Thyroid CDE Dose** - Dose to the thyroid due to intake of radioactive iodine.

3.1.28 **Total Effective Dose Equivalent (TEDE)** - The sum of external dose (DDE) and the equivalent amount of whole body dose due to individual organ uptakes.

3.1.29 **Unplanned** - An event or action is UNPLANNED if it is not the expected result of normal operations, testing, or maintenance. Events that result in corrective or mitigative actions being taken in accordance with abnormal or emergency procedures are UNPLANNED.

3.1.30 **Valid** - An indication or report or condition is considered VALID when it is conclusively verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment (e.g., within 15 minutes).

3.1.31 **Visible Damage** - Damage to equipment or structure that is readily observable without measurements, testing, or analyses. Damage is sufficient to cause concern regarding the continued operability or reliability of affected safety structure, system, or component. Example damage includes: deformation due to heat or impact, denting, penetration, rupture, cracking, paint blistering. Surface blemishing (e.g., paint chipping, scratches) should not be included.

3.2 RESPONSIBILITIES

3.2.1 The Emergency Coordinator controls all activities at CR-3 during activation of the Radiological Emergency Response Plan.

3.2.2 The Emergency Coordinator shall not delegate the decisions related to classification of the emergency condition.

3.2.3 The Emergency Coordinator shall not delegate the decisions related to notification and protective action recommendations to STATE and Local authorities who implement off-site emergency measures, until the EOF Director communicates to the Emergency Coordinator the EOF accepts the STATE notification and Protective Action Recommendations (PARs) responsibilities. At this time, the EOF completes the State of Florida Notification Form.

3.2.4 Upon arrival on-site, the Director Nuclear Plant Operations or designated alternate contacts the Control Room Emergency Coordinator or goes to the Control Room and receives a briefing about the status of the emergency condition and the implementation of the Radiological Emergency Response Plan. When ready to assume responsibility as the Emergency Coordinator, inform the Control Room Emergency Coordinator and Technical Support Center staff.

3.2.5 The Emergency Coordinator provides the Emergency Operations Facility Director an incident report when a sustained Site Area Emergency or General Emergency involves a Recovery Plan. This documents the emergency and serves as a basis for recovery phase operations.

3.2.6 During declared emergency conditions, the Emergency Coordinator is the sole contact for emergency regulatory directives. The Emergency Coordinator evaluates these directives for possible response to the emergency condition.

3.2.7 The Emergency Coordinator responsibilities in other Emergency Plan Implementing Procedures are implemented when plant conditions warrant.

3.2.8 Based on the evaluation of the emergency condition, the Emergency Coordinator has the authority to implement the following actions:

- Direct personnel to shelter or evacuate the Crystal River Energy Complex.
- Order Crystal River Energy Complex Plants placed in a safe shutdown condition.
- NOTIFY all applicable agencies of the plant status.
- Suspend security safeguards as appropriate. (10 CFR 50.54(x)(y)
- REQUEST outside assistance, if necessary.
- Make the necessary personnel assignments to PROVIDE continuing response for long-term activities.

3.2.9 The Emergency Coordinator reports to the EOF Director, once the EOF is operational.

3.2.10 The EOF Director is responsible for the direction and control of all emergency phase activities once the EOF is declared operational. The EOF Director has authority and responsibility for management of emergency response resources, coordination of radiological and environmental assessment, recommendations for public protective actions, and coordination of emergency response activities with Federal, State, and local agencies.

3.2.11 Nuclear Licensing is responsible for preparing a written summary of any Alert, Site Area or General emergency for the NRC and the State of Florida within twenty-four hours (or the next working day) from termination of the event.

3.2.12 During Severe Accident conditions, the EC is responsible for reviewing and providing final approval of all mitigation strategies developed by the Accident Assessment Team prior to implementation.

3.3 LIMITS AND PRECAUTIONS

3.3.1 Upon declaration of a GENERAL EMERGENCY, the minimum protective action recommendation is:

EVACUATE ZONE 1

3.3.2 During the initial phase of an emergency condition, the lack of information may prevent the Emergency Coordinator from completing the FLORIDA NUCLEAR PLANT EMERGENCY NOTIFICATION FORM. If information is not available, do not delay notification to State Warning Point Tallahassee. Indicate additional information will follow when it becomes available.

3.3.3 The NRC EVENT NOTIFICATION WORKSHEET is used as a guideline to PROVIDE adequate detail to the Headquarters Operations Officer to understand the event and its significance. All the information regarding an event may not be available at the time of notification, but at a minimum must PROVIDE the event classification and description as soon as possible after the State Notification, within the required time.

3.3.4 For all radiological, hazardous material spills, toxic gas releases or violent weather conditions, the EC determines the safe actions for plant personnel, which may include delaying the staffing of the TSC and EOF until it is safe to do so.

3.3.5 The Emergency Coordinator directly notifies the Director Nuclear Plant Operations and/or EOF Director to ensure the rationale of the emergency classification is understood.

3.3.6 Individuals assigned to make notifications are trained on how to make notifications and are familiar with communication systems. [NOCS 21207]

3.3.7 The Technical Support Center (TSC) continues to complete items on the FLORIDA NUCLEAR PLANT EMERGENCY NOTIFICATION FORM and transmits to the EOF until the EOF Director declares the EOF operational, AND informs the EC the EOF accepts responsibility for State notifications and Protective Action Recommendations. At this time, the EOF Director assumes full responsibility for completing the FLORIDA NUCLEAR PLANT EMERGENCY NOTIFICATION FORM.

3.3.8 Telephone notifications to the Nuclear Regulatory Commission (NRC), State of Florida, Citrus and Levy Counties are complete when direct voice contacts are made with the responsible representatives of the agencies notified. The leaving of a message with an agency's telephone operator, secretary, answering service, or message recording device is not a completed notification.

3.3.9 The Emergency Action Levels are not intended for maintenance and/or testing situations where abnormal instrument readings, alarms, and observations are expected. Some maintenance evolutions may require compensatory actions.

4.0 INSTRUCTIONS

4.0.1 RECORD significant information, events, and actions taken during the emergency condition and retain for later evaluation. Information substantiating the sequence of events is compiled from procedures, communication logs, tape recordings, flip charts, message copies, photographs (if available) and other pertinent documentation.

4.0.2 DETERMINE the emergency classification using Enclosure 1, EMERGENCY CLASSIFICATION TABLE.

Page 2	FISSION PRODUCT BARRIER MATRIX
Page 3	ABNORMAL RADIATION LEVELS/RADIOLOGICAL EFFLUENT
Page 5	NATURAL/MANMADE HAZARDS AND EC JUDGEMENT
Page 11	SYSTEM MALFUNCTION
Page 16	LOSS OF POWER

4.0.3 PERFORM steps from Emergency Coordinator Guide for each emergency classification as indicated in the following Sections:

4.1 UNUSUAL EVENT
4.2 ALERT
4.3 SITE AREA EMERGENCY
4.4 GENERAL EMERGENCY

4.0.4 USE the time blocks in Sections 4.1, 4.2, 4.3 and 4.4 to provide a reference of actions taken during the emergency condition. All actions, with the exception of decisions relating to classification and notification and protective actions recommendations made to State and Local authorities, can be performed in parallel by delegation from the EC.

4.0.5 IF an emergency classification is upgraded before the first notification is made,
THEN ENSURE notification is made within 15 minutes of original classification.

4.0.6 IF it is discovered that a condition previously existed that should have resulted in an emergency declaration,
AND the condition no longer exists,
THEN make notifications to the NRC Operations Center via ENS within one hour of discovering the undeclared event
AND NOTIFY the Emergency Planning staff to NOTIFY the State and Local Governments. An emergency declaration is not required.

4.0.7 Information requested for TSC turnover is contained in Enclosure 4 of EM-102, Operation of the Technical Support Center.

4.1 EMERGENCY COORDINATOR'S GUIDE FOR UNUSUAL EVENT
[NOCS 1129, 96042]

TIME

UNUSUAL EVENT DECLARED DATE _____ / _____

RECOMMENDED WITHIN 5 MINUTES

4.1.1 NOTIFY Control Room staff. _____

4.1.2 IF the emergency is due to a Security Event, THEN REFER TO Enclosure 3 before proceeding with the following steps. _____

4.1.3 NOTIFY Plant personnel using form from Section 4.1.12. _____

REQUIRED WITHIN 15 MINUTES

4.1.4 NOTIFY SWPT within 15 minutes of declaration using Enclosure 2. (Also refer to Section 4.1.9.) _____

RECOMMENDED WITHIN 15 MINUTES

4.1.5 IF a release is occurring as a result of this event, THEN COMPLETE EM-204A, as time permits. _____

4.1.6 NOTIFY DNPO or EC on-call. REQUEST he notify the EOF Director. _____

RECOMMENDED WITHIN 30 MINUTES

4.1.7 NOTIFY CR-3 NRC Resident Inspector (EM-206, Enclosure 3). _____

4.1.8 NOTIFY Units 1/2 & 4/5 Control Rooms per Enclosure 5. _____, _____

4.1.9 NOTIFY NRC via ENS immediately after the State per Enclosure 4. **REQUIRED WITHIN 60 MINUTES.** _____

UNUSUAL EVENT UPDATES/TERMINATION

TIME

4.1.10 PROVIDE periodic plant status updates to:

- SWPT (every 60 minutes or as agreed upon) per Enclosure 2
- NRC per Enclosure 4 (after STATE update, unless continuous communication established)
- 1,2,4 & 5 Control Rooms per Enclosure 5
- CR-3 Plant Personnel via announcements

4.1.11 If terminating, NOTIFY: DATE _____/ _____

- EC on call and REQUEST notification to EOF Director
- SWPT and document on State Form per Enclosure 2 _____
- NRC within one hour of termination with verbal summary per Enclosure 4 _____
- Unit 1,2,4 & 5 Control Rooms per Enclosure 5 _____
- CR-3 Plant personnel via PA announcement _____

4.1.12 **PA ANNOUNCEMENT FOR AN UNUSUAL EVENT**

ANNOUNCE or perform the following: Time: _____

1) SOUND the appropriate local evacuation alarm if required.

2) "ATTENTION ALL PERSONNEL, CRYSTAL RIVER 3 IS IN AN UNUSUAL EVENT BASED ON _____"

3) "THERE (IS OR IS NOT) A RADIOLOGICAL RELEASE TO THE ENVIRONMENT IN PROGRESS."

4) STATE any appropriate special instructions (areas to be avoided or evacuated, etc.)
(IF conditions warrant personnel accountability,
THEN REQUEST personnel to report to Local Assembly Areas)

_____

_____

5) REPEAT the announcement.

6) ESTABLISH continuous monitoring on PL-1.

4.2 EMERGENCY COORDINATOR'S GUIDE FOR AN ALERT [NOCS 1129, 96042]

TIME

ALERT DECLARED DATE _____ / _____

RECOMMENDED WITHIN 5 MINUTES

4.2.1 NOTIFY Control Room staff. _____

4.2.2 IF the emergency is due to a Security Event, THEN REFER TO Enclosure 3 before proceeding with the following steps. _____

4.2.3 IF safe conditions exist, THEN NOTIFY Security to activate the TSC. _____

4.2.4 NOTIFY Plant personnel using form from Section 4.2.16. _____

REQUIRED WITHIN 15 MINUTES

4.2.5 NOTIFY SWPT within 15 minutes of declaration per Enclosure 2. (Also refer to Section 4.2.10.) _____

RECOMMENDED WITHIN 15 MINUTES

4.2.6 IF a release is occurring as a result of this event, THEN COMPLETE EM-204A, as time permits. _____

RECOMMENDED WITHIN 30 MINUTES

4.2.7 NOTIFY DNPO or EC on-call. REQUEST he notify the EOF Director. _____

4.2.8 NOTIFY CR-3 NRC Resident Inspector (EM-206, Enclosure 3). _____

4.2.9 NOTIFY Units 1/2 & 4/5 Control Rooms per Enclosure 5. _____, _____

4.2.10 NOTIFY NRC via ENS immediately after the State per Enclosure 4. **REQUIRED WITHIN 60 MINUTES.** _____

4.2.11 ACTIVATE ERDS per Enclosure 6. **REQUIRED WITHIN 60 MINUTES.** _____

ONCE TSC OPERATIONAL

4.2.12 NOTIFY Corporate Security (EM-206, Enclosure 3). _____

4.2.13 NOTIFY FPC Risk Management (EM-206, Enclosure 3). _____

ALERT UPDATES/TERMINATION

		TIME
4.2.14	PROVIDE periodic plant status updates to:	
	- SWPT (every 60 minutes or as agreed upon) per Enclosure 2 including the Supplemental Data Sheet	
	- 1,2,4 & 5 Control Rooms per Enclosure 5	
	- CR-3 Plant Personnel via announcements	
4.2.15	If terminating, NOTIFY: DATE _____/_____	
	- Company Senior Officer, if requested	_____
	- DNPO and EOF Director	_____
	- SWPT and document on State Form per Enclosure 2	_____
	- NRC within one hour of termination with verbal summary	_____
	- Unit 1,2,4 & 5 Control Rooms per Enclosure 5	_____
	- CR-3 Plant personnel via PA announcement	_____
	- Corporate Security Specialist (EM-206, Enclosure 3)	_____
	- FPC Risk Management (EM-206, Enclosure 3)	_____
	- REQUEST Licensing to prepare a written summary within twenty-four hours (or next working day) of termination to SWPT and NRC.	_____

4.2.16 **PA ANNOUNCEMENT FOR AN ALERT**

CONSIDER safety of plant personnel and then ANNOUNCE or perform the following:

Time: _____

1) SOUND the appropriate local evacuation alarm if required.

2) "ATTENTION ALL PERSONNEL, CRYSTAL RIVER 3 IS IN AN ALERT BASED ON _____"

3) "THERE (IS OR IS NOT) A RADIOLOGICAL RELEASE TO THE ENVIRONMENT IN PROGRESS."

4) "ACTIVATE THE TSC/OSC. REPORT TO YOUR SHOP OR LOCAL ASSEMBLY AREA FOR ACCOUNTABILITY."

5) STATE any appropriate special instructions (areas to be avoided or evacuated, remaining at critical jobs, etc.)

_____

_____

6) REPEAT the announcement.

7) ESTABLISH continuous monitoring on PL-1.

4.3 EMERGENCY COORDINATOR'S GUIDE FOR SITE AREA EMERGENCY
[NOCS 1129, 96042]

TIME

SITE AREA EMERGENCY DECLARED DATE _____/_____

RECOMMENDED WITHIN 5 MINUTES

4.3.1 NOTIFY Control Room staff. _____

4.3.2 IF the emergency is due to a Security Event, THEN REFER TO Enclosure 3 before proceeding with the following steps. _____

4.3.3 IF safe conditions exist, THEN NOTIFY Security to activate the EOF and TSC. _____

4.3.4 NOTIFY Plant personnel using form from Section 4.3.19 and SOUND Site Evacuation Alarm. _____

REQUIRED WITHIN 15 MINUTES

4.3.5 NOTIFY SWPT within 15 minutes of declaration per Enclosure 2. (Also refer to Section 4.3.11.) _____

RECOMMENDED WITHIN 15 MINUTES [NOCS 9090,9130]

4.3.6 DETERMINE protective actions for Energy Complex using Enclosure 7. NOTIFY Nuclear Security to coordinate with Corporate Security to ENSURE protective action instructions are provided for all areas of the Energy Complex. _____

4.3.7 NOTIFY Units 1/2 & 4/5 Control Rooms per Enclosure 5. _____,_____

4.3.8 IF a release is occurring as a result of this event, THEN COMPLETE EM-204A, as time permits. _____

RECOMMENDED WITHIN 30 MINUTES

4.3.9 NOTIFY DNPO or EC on-call. REQUEST he notify the EOF Director. _____

4.3.10 NOTIFY CR-3 NRC Resident Inspector (EM-206, Enclosure 3). _____

4.3.11 NOTIFY NRC via ENS immediately after the State per Enclosure 4. **REQUIRED WITHIN 60 MINUTES.** (Once operational, this responsibility stays at TSC.) _____

4.3.12 ENSURE ERDS has been activated per Enclosure 6. **REQUIRED WITHIN 60 MINUTES** _____

ONCE TSC OPERATIONAL **TIME**

4.3.13 VERIFY Protected Area accountability is completed by Security within 30 minutes of an evacuation of the Protected Area. _____

4.3.14 NOTIFY FPC Risk Management (EM-206, Enclosure 3). _____

SITE AREA UPDATES/TERMINATION

4.3.15 PROVIDE periodic plant status updates to:

- SWPT (every 60 minutes or as agreed upon) per Enclosure 2 including the Supplemental Data Sheet
- 1,2,4 & 5 Control Rooms per Enclosure 5
- CR-3 Plant Personnel via announcements

4.3.16 IF recommending termination, entering the recovery phase, or de-escalating the event,
THEN COORDINATE the decision with the State, counties, the EOF Director and if requested, the Company Senior Officer, before completing the Florida Nuclear Plant Emergency Notification Form. CONSIDER the following when making this determination:

- Is a release continuing,
- Are plant conditions stable and expected to remain stable,
- Is the full emergency response organization needed to support safe and stable operation, or mitigation activities,
- Do radiological and other plant conditions permit resumption of normal personnel exposure limits to continue mitigation repair activities.
- Do radiological and other plant conditions permit resumption of normal access to plant and surrounding areas.

DATE _____/_____

4.3.17 NOTIFY:

- NRC within one hour of termination with verbal summary. _____
- Unit 1,2,4 & 5 Control Rooms per Enclosure 5. _____
- CR-3 Plant personnel via PA announcement. _____
- Corporate Security Specialist (EM-206, Enclosure 3). _____
- FPC Risk Management (EM-206, Enclosure 3). _____

4.3.18 REQUEST Licensing to prepare a written summary within twenty-four hours (or next working day) of termination to SWPT and NRC. _____

4.3.19 **PA ANNOUNCEMENT FOR A SITE AREA EMERGENCY**
[NOCS 7455]

CONSIDER safety of plant personnel and then ANNOUNCE or perform the following:

Time: _____

1) SOUND the Site Evacuation alarm.

2) "ATTENTION ALL PERSONNEL, CRYSTAL RIVER 3 IS IN A SITE AREA EMERGENCY BASED ON _____"

3) "THERE (IS **OR** IS NOT) A RADIOLOGICAL RELEASE TO THE ENVIRONMENT IN PROGRESS."

4) **IF the TSC/OSC is not activated,**
THEN ANNOUNCE: "ACTIVATE THE TSC/OSC."

5) "PERSONNEL ARE TO IMMEDIATELY EVACUATE THE PROTECTED AREA AND REPORT TO THE SITE ADMINISTRATION AUDITORIUM."

6) "ALL EOF PERSONNEL, REPORT TO THE EOF."

7) STATE any appropriate special instructions (areas to be avoided or evacuated, etc.)

_____

_____

8) REPEAT the announcement.

9) ESTABLISH continuous monitoring on PL-1.

4.4 EMERGENCY COORDINATOR'S GUIDE FOR GENERAL EMERGENCY
[NOCS 1129, 96042]

GENERAL EMERGENCY DECLARED. DATE _____ TIME _____

RECOMMENDED WITHIN 5 MINUTES **TIME**

4.4.1 NOTIFY Control Room staff. _____

4.4.2 IF the emergency is due to a Security Event, THEN REFER TO Enclosure 3 before proceeding with the following steps. _____

4.4.3 IF safe conditions exist, THEN NOTIFY Security to activate the TSC and EOF (if not activated). _____

4.4.4 NOTIFY Plant personnel using form from Section 4.4.20 and SOUND Site Evacuation Alarm if PROTECTED AREA not already evacuated. _____

REQUIRED WITHIN 15 MINUTES

4.4.5 DETERMINE Protective Action Recommendations per Enclosure 8. (Minimum Protective Action Recommendations will be to evacuate Zone 1) _____

4.4.6 NOTIFY SWPT within 15 minutes of declaration per Enclosure 2. (Also refer to Section 4.4.12.) _____

RECOMMENDED WITHIN 15 MINUTES

4.4.7 DETERMINE Energy Complex protective actions per Enclosure 7 and NOTIFY Nuclear Security to coordinate with Corporate Security to ENSURE evacuation instructions are provided for all areas of the Energy Complex. _____

4.4.8 NOTIFY Units 1/2 & 4/5 Control Rooms per Enclosure 5. _____,_____

4.4.9 IF a release is occurring as a result of this event, THEN COMPLETE EM-204A, as time permits. _____

RECOMMENDED WITHIN 30 MINUTES (Not necessary if TSC and EOF Operational)

4.4.10 NOTIFY DNPO or EC on-call. REQUEST he notify the EOF Director. _____

4.4.11 NOTIFY CR-3 NRC Resident Inspector (EM-206, Enclosure 3). _____

4.4.12 NOTIFY NRC via ENS immediately after the State per Enclosure 4. **REQUIRED WITHIN 60 MINUTES.** (Once operational this responsibility stays at TSC.) _____

4.4.13 ENSURE ERDS has been activated per Enclosure 6. **REQUIRED WITHIN 60 MINUTES.** _____

ONCE TSC IS OPERATIONAL **TIME**

4.4.14 VERIFY Protected Area accountability is Completed by Security within 30 minutes of an evacuation of the Protected Area.

4.4.15 NOTIFY FPC Risk Management (EM-206, Enclosure 3). _____

GENERAL EMERGENCY UPDATES/TERMINATION

4.4.16 PROVIDE periodic plant status updates to:

- SWPT (every 60 minutes or as agreed upon) per Enclosure 2 including the Supplemental Data Sheet

- 1,2,4 & 5 Control Rooms per Enclosure 5

- CR-3 Plant Personnel via announcements

4.4.17 IF recommending termination, entering the recovery phase, or de-escalating the event,
THEN COORDINATE the decision with the State, counties, the EOF Director and if requested, the Company Senior Officer, before completing the Florida Nuclear Plant Emergency Notification Form.
Consider the following when making this determination:

- Is a release continuing,
- Are plant conditions stable and expected to remain stable,
- Is the full emergency response organization needed to support safe and stable operation, or mitigation activities,
- Do radiological and other plant conditions permit resumption of normal personnel exposure limits to continue mitigation repair activities.
- Do radiological and other plant conditions permit resumption of normal access to plant and surrounding areas.

DATE _____/_____

4.4.18 NOTIFY:

- NOTIFY NRC within one hour of termination with verbal summary. _____

- Unit 1,2,4 & 5 Control Rooms per Enclosure 5. _____

- CR-3 Plant personnel via PA announcement. _____

- Corporate Security Specialist (EM-206, Enclosure 3). _____

- FPC Risk Management (EM-206, Enclosure 3). _____

4.4.19 REQUEST Licensing to prepare a written summary within twenty-four hours (or next working day) of termination to SWPT and NRC. _____

4.4.20 **PA ANNOUNCEMENT FOR A GENERAL EMERGENCY** [NOCS 7455]

CONSIDER safety of plant personnel and then **ANNOUNCE** or perform the following:

Time: _____

1) **IF the Protected Area has not been evacuated,**
THEN SOUND the Site Evacuation alarm.

2) "ATTENTION ALL PERSONNEL, CRYSTAL RIVER 3 IS IN A GENERAL EMERGENCY BASED ON _____"

3) "THERE (IS **OR** IS NOT) A RADIOLOGICAL RELEASE TO THE ENVIRONMENT IN PROGRESS."

4) **IF the TSC/OSC is not activated,**
THEN ANNOUNCE: "ACTIVATE THE TSC/OSC."

5) **IF the Protected Area has not been evacuated,**
THEN ANNOUNCE: "ALL NON-ESSENTIAL PERSONNEL, IMMEDIATELY EVACUATE THE PROTECTED AREA AND FOLLOW INSTRUCTIONS FROM SECURITY."

6) **IF the EOF is not activated,**
THEN ANNOUNCE: "ALL EOF PERSONNEL, REPORT TO THE EOF."

7) STATE any appropriate special instructions (areas to be avoided or evacuated, etc.)

_____

_____

8) REPEAT the announcement.

9) ESTABLISH continuous monitoring on PL-1.

EMERGENCY CLASSIFICATION TABLE
EMERGENCY ACTION LEVEL INDEX

ABNORMAL RADLEVELS/ RADIOLOGICAL EFFLUENT				
CATEGORY	UE	ALERT	SAE	GE
Gaseous Effluents	1.1	1.2	1.3	1.4
Liquid Effluents	1.5	1.6		
Unexpected Radiation Levels	1.7	1.8		
Fuel Handling Spent Fuel Pool or Transfer Canal Water Level	1.9	1.10		

NATURAL/MAN-MADE HAZARDS AND EC JUDGMENT				
CATEGORY	UE	ALERT	SAE	GE
Earthquake Experienced	2.1	2.2		
External Flooding	2.3	2.4		
Hurricane	2.5			
Tornado/High Winds	2.6	2.7		
Aircraft/Vehicle Crash	2.8	2.9		
Toxic or Flammable Gases	2.10	2.11		
Explosions/Catastrophic Pressurized Equipment Failure	2.12	2.13		
Fire	2.14	2.15		
Control Room Evacuation		2.16	2.17	
Security Event	2.18	2.19	2.20	2.21
Internal Flooding	2.22	2.23		
Emergency Coordinator Judgment	2.24	2.25	2.26	2.27

SYSTEM MALFUNCTION				
CATEGORY	UE	ALERT	SAE	GE
Loss of Communications	3.1			
Failure of Reactor Protection		3.2	3.3	3.4
Inability to Reach ITS Time Limits	3.5			
Loss of Alarms/Indications	3.6	3.7	3.8	
Fuel Clad Degradation	3.9			
Turbine Failure	3.10	3.11		
RCS Leakage	3.12			
Inability to Maintain Hot Shutdown			3.13	
Inadvertent Criticality	3.14			
Inability to Maintain Plant in Cold Shutdown		3.15		
Loss of Water Level in Reactor Vessel that has Uncovered or Will Uncover Fuel			3.16	
LOSS OF POWER				
CATEGORY	UE	ALERT	SAE	GE
Loss of AC Power	4.1	4.2	4.3	4.4
Loss of AC Power (Shutdown)		4.5		
Loss of Vital DC Power			4.6	
Loss of Vital DC Power (Shutdown)	4.7			

EMERGENCY CLASSIFICATION TABLE
FISSION PRODUCT BARRIER MATRIX
APPLICABLE MODES: 1-4 COMPLETE FOR ALL BARRIERS

FUEL CLAD LOSS FACTOR (+4)		RCS LOSS FACTOR (+4)		CONTAINMENT LOSS FACTOR (+2)	
1. CORE CONDITIONS IN REGION 3 OR SEVERE ACCIDENT REGION OF ICC CURVES		1. RCS LEAK OR OTSG TUBE LEAK RESULTING IN LOSS OF ADEQUATE SUBCOOLING MARGIN		1. RAPID UNEXPLAINED RB PRESSURE DECREASE FOLLOWING INITIAL INCREASE	
2. RCS ACTIVITY >300 µCi/gm I-131		2. RM-G29 OR 30 > 10 R/hr FOR 15 MINUTES OR LONGER		2. CONTAINMENT PRESSURE OR SUMP LEVEL RESPONSE NOT CONSISTENT WITH LOCA CONDITIONS	
3. RM-G29 OR 30 >100 R/hr FOR 15 MINUTES OR LONGER		3. EC DEEMS RCS BARRIER IS LOST		3. AN OTSG HAS > 10 GPM TUBE RUPTURE WITH PROLONGED STEAMING TO THE ATMOSPHERE FROM THE AFFECTED OTSG OR AN UNISOLABLE STEAM LEAK OUTSIDE RB FROM THE AFFECTED OTSG	
4. EC DEEMS FUEL CLAD BARRIER IS LOST				4. CONTAINMENT ISOLATION IS INCOMPLETE AND RELEASE PATH TO THE ENVIRONMENT EXISTS	
				5. EC DEEMS CONTAINMENT BARRIER IS LOST	
IF ANY ITEM IS CHECKED, BARRIER IS LOST, ENTER 4 FOR FUEL CLAD FACTOR IN CLASSIFICATION TABLE BELOW		IF ANY ITEM IS CHECKED, BARRIER IS LOST ENTER 4 FOR RCS FACTOR IN CLASSIFICATION TABLE BELOW		IF ANY ITEM IS CHECKED, BARRIER IS LOST ENTER 2 FOR CONTAINMENT FACTOR IN CLASSIFICATION TABLE BELOW	
FUEL CLAD POTENTIAL LOSS FACTOR (+3)		**RCS POTENTIAL LOSS FACTOR (+3)**		**CONTAINMENT POTENTIAL LOSS FACTOR (+1.5)**	
1. RCS CONDITIONS WARRANT ENTRY INTO EOP-07		1. RCS LEAK OR OTSG TUBE LEAK REQUIRING ONE OR MORE INJECTION VALVES		1. RB PRESSURE >54 psig	
2. CORE EXIT THERMOCOUPLES >700°F		2. RCS LEAK OR OTSG TUBE LEAK RESULTS IN ES ACTUATION ON LOW RCS PRESSURE		2. RB HYDROGEN CONCENTRATION >4%	
3. EC DEEMS FUEL CLAD BARRIER IN JEOPARDY		3. RCS PRESSURE/TEMPERATURE RELATIONSHIP VIOLATES NDT LIMITS		3. RB PRESSURE >30 psig WITH NO BUILDING SPRAY AVAILABLE	
		4. HPI/PORV OR HPI/SAFETY VALVE COOLING IS IN PROGRESS		4. RMG-29 OR 30 READINGS >25,000 R/hr	
		5. EC DEEMS RCS BARRIER IN JEOPARDY		5. CORE CONDITIONS IN SEVERE ACCIDENT REGION OF ICC CURVES FOR >15 MINUTES	
				6. EC DEEMS CONTAINMENT BARRIER IN JEOPARDY	
IF ANY ITEM IS CHECKED, BARRIER IS POTENTIALLY LOST ENTER 3 FOR FUEL CLAD FACTOR IN CLASSIFICATION TABLE BELOW		IF ANY ITEM IS CHECKED, BARRIER IS POTENTIALLY LOST, ENTER 3 FOR RCS FACTOR IN CLASSIFICATION TABLE BELOW		IF ANY ITEM IS CHECKED, BARRIER IS POTENTIALLY LOST, ENTER 1.5 FOR CONTAINMENT FACTOR IN CLASSIFICATION TABLE BELOW	

CLASSIFICATION TABLE

ENTER LOSS FACTOR OR POTENTIAL LOSS FACTOR OR ZERO FOR EACH BARRIER THEN TOTAL AND DETERMINE CLASS BELOW

FUEL CLAD FACTOR _____ + RCS FACTOR _____ + CONTAINMENT FACTOR _____ = _____

IF TOTAL IS:	RECOMMENDED EVENT CLASSIFICATION IS:
> 0 BUT ≤ 2	UNUSUAL EVENT
> 2 BUT ≤ 4	ALERT
> 4 BUT ≤ 8.5	SITE AREA EMERGENCY
> 8.5	GENERAL EMERGENCY

EMERGENCY CLASSIFICATION TABLE
ACCIDENT CONDITION:

ABNORMAL RAD LEVELS/RADIOLOGICAL EFFLUENT

CATEGORY	UNUSUAL EVENT	ALERT	SITE AREA EMERGENCY	GENERAL EMERGENCY
Gaseous Effluents MODES: ALL	1.1 MODES: ALL (1 or 2) 1. A VALID reading on RM-A1 or RM-A2 exceeds the high alarm setpoint for 60 minutes or longer OR 2. Sample analysis confirms gaseous effluent being released exceeds 2 times the ODCM noble gas release setpoint for 60 minutes or longer	1.2 MODES: ALL (1 or 2) 1. A VALID reading on RM-A1 or RM-A2 Mid-Range monitor exceeds 6mR/hr. for 15 minutes or longer OR 2. Sample analysis confirms gaseous effluent being released exceeds 200 times the ODCM noble gas release setpoint for 15 minutes or longer	1.3 MODES: ALL (1 or 2 or 3) 1. VALID RM-A1 or RM-A2 Mid-Range monitor reading exceeds the values on the following Table for the current Stability Class for 15 minutes or longer: Stab. Class / Reading (mR/hr) A, B or C / 100 D or E / 20 F or G / 12 OR 2. Dose Assessment results indicate SITE BOUNDARY dose >100 mR TEDE or >500 mR thyroid CDE for the actual or projected duration of the release OR 3. Field survey results indicate closed windows dose rates >100mR/hr expected to continue for more than one hour; or analyses of field survey samples indicate thyroid CDE of 500mR for one hour of inhalation, at or beyond SITE BOUNDARY	1.4 MODES: ALL (1 or 2 or 3) 1. VALID RM-A1 or RM-A2 Mid-Range monitor reading exceeds the values on the Table below for the current Stability Class for 15 minutes or longer: Stab. Class / Reading (mR/hr) A, B or C / 1000 D or E / 200 F or G / 120 OR 2. Dose Assessment results indicate SITE BOUNDARY dose >1000 mR TEDE or >5000 mR thyroid CDE for the actual or projected duration of the release AND core damage is suspected or has occurred OR 3. Field survey results indicate closed windows dose rates >1000mR/hr expected to continue for more than one hour; or analyses of field survey samples indicate thyroid CDE of 5000mR for one hour of inhalation, at or beyond SITE BOUNDARY
Liquid Effluents MODES: ALL	1.5 MODES: ALL (1 or 2) 1. A VALID reading on RM-L2, RM-L7, or sample analysis confirms the release exceeds 2 times the ODCM release setpoint for 60 minutes or longer OR 2. Release continued for 60 minutes or longer with no dilution flow	1.6 MODES: ALL A VALID reading on RM-L2, RM-L7, or sample analysis confirms the release exceeds 200 times the ODCM release setpoint for 15 minutes or longer	*Not Applicable*	*Not Applicable*

EMERGENCY CLASSIFICATION TABLE
ACCIDENT CONDITION:

ABNORMAL RAD LEVELS/RADIOLOGICAL EFFLUENT

CATEGORY	UNUSUAL EVENT	ALERT	SITE AREA EMERGENCY	GENERAL EMERGENCY
Unexpected Radiation Levels MODES: ALL	1.7 MODES: ALL One or more VALID radiation monitor readings unexpectedly exceed the values below for 15 minutes or longer: RM-G3 = 400 mR/hr RM-G4 = 600 mR/hr RM-G5 = 3000 mR/hr RM-G9 = 100 mR/hr RM-G10 = 800 mR/hr RM-G14 = 1000 mR/hr RM-G17 = 800 mR/hr	1.8 MODES: ALL (1 or 2) 1. VALID radiation reading greater than 15 mR/hr for 15 minutes or longer in the Control Room (RM-G1) or the Central Alarm Station (CAS) OR 2. One or more VALID radiation monitor readings unexpectedly exceed the values below for 15 minutes or longer: RM-G3 = 5,000 mR/hr RM-G4 = 5,000 mR/hr RM-G9 = 5,000 mR/hr RM-G10 = 5,000 mR/hr RM-G17 = 5,000 mR/hr	*Refer to Fission Product Barrier Matrix, Gaseous Effluents, or Emergency Coordinator Judgment*	*Refer to Fission Product Barrier Matrix, Gaseous Effluents, or Emergency Coordinator Judgment*
Fuel Handling Spent Fuel Pool or Transfer Canal Water Level MODES: ALL	1.9 MODES: ALL (1 and 2) 1. (a or b) a. Uncontrolled level decrease resulting in indications of -2.5 feet in spent fuel pool OR b. Confirmed plant personnel report of uncontrolled significant water level drop in spent fuel pool or transfer canal when Spent Fuel transfer tubes are open AND 2. Fuel remains covered with water	1.10 MODES: ALL (1 or 2) 1. (a and b) a. Plant personnel report damage of irradiated fuel AND b. VALID high alarm as indicated on RM-G15 or RM-G16 OR 2. Plant personnel report spent fuel pool or transfer canal water level drop has or will exceed makeup capacity such that irradiated fuel will be uncovered	*Refer to Gaseous Effluents or Emergency Coordinator Judgment*	*Refer to Gaseous Effluents or Emergency Coordinator Judgment*

EMERGENCY CLASSIFICATION TABLE
ACCIDENT CONDITION

NATURAL/MAN-MADE HAZARDS AND EC JUDGMENT

CATEGORY	UNUSUAL EVENT	ALERT	SITE AREA EMERGENCY	GENERAL EMERGENCY
Earthquake Experienced MODES: ALL	2.1 MODES: ALL (1 and 2) 1. Ground motion sensed by plant personnel AND 2. Confirmed earthquake causing Annunciator C-3-14 "Seismic System Trouble" alarm	2.2 MODES: ALL (1 and 2) 1. Ground motion sensed by plant personnel or confirmed Annunciator C-3-14 "Seismic System Trouble" alarm AND 2. (a or b) a. Analysis confirms the earthquake at >0.05g OR b. Indications show degraded SAFE SHUTDOWN EQUIPMENT performance due to the earthquake	*Refer to Fission Product Barrier Matrix or Emergency Coordinator Judgment*	*Refer to Fission Product Barrier Matrix or Emergency Coordinator Judgment*
External Flooding MODES: ALL	2.3 MODES: ALL Intake canal level or visual observation indicates flood water level ≥ 98 feet	2.4 MODES: ALL (1 and 2) 1. Intake canal level or visual observation indicates flood water level ≥ 98 feet AND 2. Indications show degraded SAFE SHUTDOWN EQUIPMENT performance due to the flooding	*Refer to Fission Product Barrier Matrix or Emergency Coordinator Judgment*	*Refer to Fission Product Barrier Matrix or Emergency Coordinator Judgment*
Hurricane MODES: ALL	2.5 MODES: ALL The plant is within a Hurricane Warning area	*Refer to Fission Product Barrier Matrix, Tornado/High Winds, or Emergency Coordinator Judgment*	*Refer to Fission Product Barrier Matrix or Emergency Coordinator Judgment*	*Refer to Fission Product Barrier Matrix or Emergency Coordinator Judgment*

EMERGENCY CLASSIFICATION TABLE
ACCIDENT CONDITION:
NATURAL/MAN-MADE HAZARDS AND EC JUDGMENT

CATEGORY	UNUSUAL EVENT	ALERT	SITE AREA EMERGENCY	GENERAL EMERGENCY
Tornado/High Winds MODES: ALL	2.6 MODES: ALL Report by plant personnel of a Tornado striking within the PROTECTED AREA	2.7 MODES: ALL (1 and 2) 1. Tornado <u>or</u> High Winds <u>or</u> windborne object strike one of the following structures: - Auxiliary Building, - BWST, - Control Complex, - Diesel Generator Building, - EFT-2 Building, - Intermediate Building, - Reactor Building - EFP-3 Building **<u>AND</u>** 2. (a or b) a. Confirmed report of significant VISIBLE DAMAGE to buildings listed above OR b. Indications show degraded SAFE SHUTDOWN EQUIPMENT performance due to the tornado or high winds or windborne objects	*Refer to Fission Product Barrier Matrix or Emergency Coordinator Judgment*	*Refer to Fission Product Barrier Matrix or Emergency Coordinator Judgment*
Aircraft/Vehicle Crash MODES: ALL	2.8 MODES: ALL Report by plant personnel of Aircraft <u>or</u> Vehicle Crash involving the following permanent structures: - Auxiliary Building, - BWST - Control Complex - Diesel Generator Building - EFT-2 Building - Intermediate Building - Reactor Building - EFP-3 Building	2.9 MODES: ALL (1 or 2) 1. Confirmed report of significant VISIBLE DAMAGE to buildings listed below: - Auxiliary Building - BWST - Control Complex - Diesel Generator Building - EFT-2 Building - Intermediate Building - Reactor Building - EFP-3 Building **<u>OR</u>** 2. Indications show degraded SAFE SHUTDOWN EQUIPMENT performance due to the Aircraft <u>or</u> Vehicle Crash	*Refer to Fission Product Barrier Matrix or Emergency Coordinator Judgment*	*Refer to Fission Product Barrier Matrix or Emergency Coordinator Judgment*

EMERGENCY CLASSIFICATION TABLE

ACCIDENT CONDITION:

NATURAL/MAN-MADE HAZARDS AND EC JUDGMENT

CATEGORY	UNUSUAL EVENT	ALERT	SITE AREA EMERGENCY	GENERAL EMERGENCY
Toxic or Flammable Gases **MODES: ALL**	**2.10 MODES: ALL** **(1 or 2)** 1. Report or detection of Toxic or Flammable Gas within the SITE BOUNDARY that could enter the Protected Area at levels >IDLH or >25% Lower Explosive Limits affecting normal operation of the plant **OR** 2. Confirmed notification by FPC, County, or State personnel to evacuate or shelter site personnel based on an offsite event	**2.11 MODES: ALL** (1 or 2 or 3) 1. Flammable Gas levels > 25% Lower Explosive Limit in areas required to maintain safe operations or establish and maintain cold shutdown **OR** 2. Toxic Gas levels ≥ IDLH levels in areas that require continuous occupancy to maintain safe operation or establish or maintain cold shutdown **OR** 3. Toxic Gas levels ≥ IDLH levels within the PROTECTED AREA such that plant personnel are unable to perform actions necessary to maintain safe operations or establish and maintain cold shutdown using protective equipment	*Refer to Fission Product Barrier Matrix, System Malfunction, or Emergency Coordinator Judgment*	*Refer to Fission Product Barrier Matrix, System Malfunction, or Emergency Coordinator Judgment*

EMERGENCY CLASSIFICATION TABLE
ACCIDENT CONDITION:
NATURAL/MAN-MADE HAZARDS AND EC JUDGMENT

CATEGORY	UNUSUAL EVENT	ALERT	SITE AREA EMERGENCY	GENERAL EMERGENCY
Explosions/ Catastrophic Pressurized Equipment Failure MODES: ALL	2.12 MODES: ALL Report by plant personnel of VISIBLE DAMAGE to permanent structures or equipment within the PROTECTED AREA due to an EXPLOSION or catastrophic failure of pressurized equipment *Refer to Security Event*	2.13 MODES: ALL (1 and 2) 1. EXPLOSION or catastrophic failure of pressurized equipment in any of the following structures: - Auxiliary Building - BWST - Control Complex - Diesel Generator Building - EFT-2 Building, - Intermediate Building - Reactor Building - EFP-3 Building AND 2. (a or b) a. Report by plant personnel of EXPLOSION or catastrophic failure of pressurized equipment causing VISIBLE DAMAGE to SAFE SHUTDOWN EQUIPMENT OR b. Indications show degraded SAFE SHUTDOWN EQUIPMENT performance due to the EXPLOSION or pressurized equipment failure	*Refer to Fission Product Barrier Matrix, System Malfunction, or Emergency Coordinator Judgment*	*Refer to Fission Product Barrier Matrix, System Malfunction, or Emergency Coordinator Judgment*
Fire MODES: ALL	2.14 MODES: ALL (1 and 2) 1. FIRE in or threatening one of the following structures: - Auxiliary Building - BWST - Control Complex, - Diesel Generator Building - EFT-2 Building - Intermediate Building - Reactor Building - EFP-3 Building AND 2. FIRE not extinguished within 15 minutes from either Control Room notification or receipt of a VALID fire alarm in the Control Room	2.15 MODES: ALL (1 or 2) 1. Report by plant personnel of VISIBLE DAMAGE to SAFE SHUTDOWN EQUIPMENT due to the FIRE OR 2. Indications show degraded SAFE SHUTDOWN EQUIPMENT performance due to the FIRE	*Refer to Fission Product Barrier Matrix, Control Room Evacuation, System Malfunctions, or Emergency Coordinator Judgment*	*Refer to Fission Product Barrier Matrix, Control Room Evacuation, System Malfunctions, or Emergency Coordinator Judgment*

EMERGENCY CLASSIFICATION TABLE

ACCIDENT CONDITION:

NATURAL/MAN-MADE HAZARDS AND EC JUDGMENT

CATEGORY	UNUSUAL EVENT	ALERT	SITE AREA EMERGENCY	GENERAL EMERGENCY
Control Room Evacuation MODES: ALL	*Not Applicable*	2.16 MODES: ALL Control Room evacuation is required per AP-990, "Shutdown Outside of the Control Room"	2.17 MODES: ALL (1 and 2) 1. Control Room evacuation is required per AP-990, "Shutdown Outside of the Control Room" AND 2. Control of the necessary equipment not established per AP-990 within 15 minutes	*Refer to Fission Product Barrier Matrix, System Malfunction, or Emergency Coordinator Judgment*
Security Event MODES: ALL	2.18 MODES: ALL (1 or 2 or 3) Report by Security Shift Supervisor of one or more of the following events: 1. Occurrence of SABOTAGE OR 2. HOSTAGE/EXTORTION situation or hostile STRIKE ACTION threatening to interrupt plant operations OR 3. A violent CIVIL DISTURBANCE ongoing outside of the PROTECTED AREA but within the SITE BOUNDARY	2.19 MODES: ALL (1 or 2) 1. Discovery of BOMB within the PROTECTED AREA OR 2. INTRUDER(S) penetrates the PROTECTED AREA	2.20 MODES: ALL INTRUDER(S) penetrates or a BOMB is discovered in any of the areas listed below: - Auxiliary Building - BWST - Control Complex - EFT-2 Building - Diesel Generator Building - Intermediate Building - Reactor Building - EFP-3 Building	2.21 MODES: ALL INTRUDER(S) has taken control of the Control Room, or Remote Shutdown Room or plant equipment such that plant personnel are unable to operate equipment required to establish and maintain safe shutdown conditions

EMERGENCY CLASSIFICATION TABLE
ACCIDENT CONDITION:

NATURAL/MAN-MADE HAZARDS AND EC JUDGMENT

CATEGORY	UNUSUAL EVENT	ALERT	SITE AREA EMERGENCY	GENERAL EMERGENCY
Internal Flooding **Modes: ALL**	**2.22 MODES: ALL** **(1 and 2)** 1. Indication of uncontrolled flooding in the Auxiliary Building or Intermediate Building **AND** 2. Water level/flooding has the potential to affect or immerse SAFE SHUTDOWN EQUIPMENT	**2.23 MODES: ALL** **(1 and 2)** 1. Water level exceeds 1.5 feet in the Auxiliary Building or Intermediate Building **AND** 2. **(a or b)** a. Indications show degraded SAFE SHUTDOWN EQUIPMENT due to the flooding **OR** b. Electrical hazards prevent plant personnel normal access to areas of plant containing SAFE SHUTDOWN EQUIPMENT	*Refer to Fission Product Barrier Matrix or Emergency Coordinator Judgment*	*Refer to Fission Product Barrier Matrix or Emergency Coordinator Judgment*
Emergency Coordinator Judgment **MODES: ALL**	**2.24 MODES: ALL** Other conditions exist which indicate a potential degradation of the level of safety of the plant	**2.25 MODES: ALL** Other conditions exist which indicate that events are in process or have occurred which involve potential or actual substantial degradation of the level of safety of the plant	**2.26 MODES: ALL** Other conditions exist which indicate actual or likely major failures of plant functions needed for the protection of the public	**2.27 MODES: ALL** **(1 or 2)** Other conditions exist which indicate: 1. Actual or imminent substantial core degradation with potential loss of containment integrity **OR** 2. The potential for uncontrolled radionuclide releases that can be expected to exceed EPA Protective Action Guidelines Plume Exposure Levels beyond the SITE BOUNDARY (see EAL 1.4)

EMERGENCY CLASSIFICATION TABLE
ACCIDENT CONDITION:

SYSTEM MALFUNCTION

CATEGORY	UNUSUAL EVENT	ALERT	SITE AREA EMERGENCY	GENERAL EMERGENCY
Loss of Communication **MODES: ALL**	**3.1 MODES: ALL** **(1 or 2)** 1. Loss of all the following in-plant communications capability: a. FPC Internal Telephone System b. PAX c. Portable UHF Radios **OR** 2. Loss of all of the following Offsite Communication capability: a. FPC Telephone System b. State Hot Ringdown (SHRD) c. All FTS 2000 NRC phones (ENS, HPN, etc.) d. State-Wide Emergency Satellite Communication (ESATCOM) System e. Cellular Phones	*Not Applicable*	*Not Applicable*	*Not Applicable*
Failure of Reactor Protection **MODES: 1,2,3 for ALERT** **MODES: 1,2 for SITE AREA and GENERAL Emergencies**	*Not Applicable*	**3.2 MODES: 1,2,3** **(1 and 2)** 1. RPS Trip setpoint exceeded and no Reactor trip occurred **AND** 2. Manual Reactor trip from Control Room was successful and reactor is shutdown	**3.3 MODES: 1,2** **(1 and 2)** 1. RPS Trip setpoint exceeded and no Reactor trip occurred **AND** 2. Manual Reactor trip from Control Room was not successful in shutting down the reactor	**3.4 MODES: 1,2** **(1 and 2 and 3)** 1. RPS Trip setpoint exceeded and no Reactor trip occurred **AND** 2. Manual Reactor trip from Control Room was not successful in shutting down the reactor **AND** 3. **(a or b)** a. Core exit thermocouple temperatures > 700°F, as indicated on SPDS. **OR** b. Adequate Secondary Cooling not available

EMERGENCY CLASSIFICATION TABLE

ACCIDENT CONDITION:

SYSTEM MALFUNCTION

CATEGORY	UNUSUAL EVENT	ALERT	SITE AREA EMERGENCY	GENERAL EMERGENCY
Inability to reach required mode within Improved Technical Specification time limits MODES: 1,2,3,4	3.5 MODES: 1,2,3,4 (1 and 2) 1. Entry into an Improved Technical Specification LCO statement requiring a mode reduction AND 2. The plant is not in the required operating mode within the time prescribed by the LCO required action	*Not Applicable*	*Not Applicable*	*Not Applicable*
Loss of Alarms/Indications MODES: 1,2,3,4	3.6 MODES: 1,2,3,4 (1 or 2) 1. UNPLANNED loss of Annunciator panels A-L and Annunciator printer for 15 minutes or longer OR 2. UNPLANNED loss of NNI-X and NNI-Y for 15 minutes or longer	3.7 MODES: 1,2,3,4 (1 and 2) 1. (a or b) a. UNPLANNED loss of Annunciator panels A-L and Annunciator printer for 15 minutes or longer OR b. UNPLANNED loss of NNI-X and NNI-Y for 15 minutes or longer AND 2. (a or b) a. SIGNIFICANT TRANSIENT in progress OR b. Loss of Plant Computer and SPDS	3.8 MODES: 1,2,3,4 (1 and 2 and 3 and 4) 1. (a or b) a. Loss of Annunciator panels A-L and Annunciator printer for 15 minutes or longer OR b. Loss of NNI-X and NNI-Y for 15 minutes or longer AND 2. SIGNIFICANT TRANSIENT in progress AND 3. Loss of Plant Computer and SPDS AND 4. Inability to directly monitor any one of the following: Subcriticality Core Cooling Containment RCS Inventory	*Refer to Fission Product Barrier Matrix or Emergency Coordinator Judgment*

EMERGENCY CLASSIFICATION TABLE

ACCIDENT CONDITION:

SYSTEM MALFUNCTION

CATEGORY	UNUSUAL EVENT	ALERT	SITE AREA EMERGENCY	GENERAL EMERGENCY
Fuel Clad Degradation MODES: 1,2,3,4,5	3.9 MODES: 1,2,3,4,5 (a or b) Radiochemistry analysis indicates: a. Dose Equivalent Iodine (I-131) >1.0 µCi/gm for 48 hours or longer OR b. Specific activity >100/E-bar for 48 hours or longer	*Refer to Fission Product Barrier Matrix*	*Refer to Fission Product Barrier Matrix*	*Refer to Fission Product Barrier Matrix*
Turbine Failure MODES: 1,2,3	3.10 MODES: 1,2,3 Report by plant personnel of main turbine failure causing penetration of the turbine casing or damage to main generator seals	3.11 MODES: 1,2,3 (1 or 2) 1. Report by plant personnel of projectiles generated by a main turbine failure causing significant VISIBLE DAMAGE any of the following structures: - Auxiliary Building - BWST - Control Complex - Diesel Generator Building - EFT-2 Building - Intermediate Building - Reactor Building - EFP-3 Building OR 2. Indications show degraded SAFE SHUTDOWN EQUIPMENT performance due to turbine generated projectiles	*Refer to Fission Product Barrier Matrix*	*Refer to Fission Product Barrier Matrix*

EMERGENCY CLASSIFICATION TABLE
ACCIDENT CONDITION:
SYSTEM MALFUNCTION

CATEGORY	UNUSUAL EVENT	ALERT	SITE AREA EMERGENCY	GENERAL EMERGENCY
RCS Leakage MODES: 1,2,3,4	3.12 MODES: 1,2,3,4 (1 or 2) 1. Unidentified Leakage ≥ 10 gpm or Pressure Boundary Leakage ≥10 gpm OR 2. Identified leakage ≥ 25 gpm	*Refer to Fission Product Barrier Matrix or Emergency Coordinator Judgment*	*Refer to Fission Product Barrier Matrix or Emergency Coordinator Judgment*	*Refer to Fission Product Barrier Matrix or Emergency Coordinator Judgment*
Inability to Maintain Hot Shutdown MODES: 1,2,3,4	*Not Applicable*	*Not Applicable*	3.13 MODES: 1,2,3,4 (1 and 2) 1. Complete loss of Main, Emergency, and Auxiliary Feedwater and unable to establish HPI cooling AND 2. Loss of subcooling margin	*Refer to Fission Product Barrier Matrix or Emergency Coordinator Judgment*
Inadvertent Criticality MODES: 2,3,4,5,6	3.14 MODES: 2,3,4,5,6 An extended and unplanned sustained positive startup rate monitored by nuclear instrumentation	*Not Applicable*	*Not Applicable*	*Not Applicable*
Inability to Maintain Plant in Cold Shutdown MODES: 5,6	*Not Applicable*	3.15 MODES: 5,6 (1 or 2) 1. Inability to maintain reactor coolant temperature below 200°F OR 2. Uncontrolled reactor coolant temperature approaching 200°F	*Refer to Loss of Water in Reactor Vessel that has uncovered or will uncover fuel*	*Not Applicable*

EMERGENCY CLASSIFICATION TABLE

ACCIDENT CONDITION:

SYSTEM MALFUNCTION

CATEGORY	UNUSUAL EVENT	ALERT	SITE AREA EMERGENCY	GENERAL EMERGENCY
Loss of Water Level in Reactor Vessel that Has Uncovered or Will Uncover Fuel **MODES: 5,6**	*Not Applicable*	*Not Applicable*	**3.16 MODES 5,6 (1 and 2)** **1.** Loss of decay heat removal per AP-404 **AND** **2. (a or b)** **a.** Incores indicating superheated conditions **OR** **b.** Incores unavailable and time to uncovery exceeded as specified in OP-103H	*Not Applicable*

EMERGENCY CLASSIFICATION TABLE

ACCIDENT CONDITION:

LOSS OF POWER

CATEGORY	UNUSUAL EVENT	ALERT	SITE AREA EMERGENCY	GENERAL EMERGENCY
Loss of AC Power MODES: ALL for UNUSUAL EVENT MODES: 1,2,3,4 for ALERT, SITE AREA and GENERAL Emergencies	**4.1 MODES: ALL** **(1 and 2)** 1. Offsite Power Transformer (OPT) and Backup ES Transformer (BEST) and Auxiliary Transformer not available for 15 minutes or longer **AND** 2. EDGs supplying power to required 4160V ES Busses	**4.2 MODES: 1,2,3,4** AC power capability to the 4160V ES busses reduced to a single power source for 15 minutes or longer such that **only** one of the following is available: - "A" EDG - "B" EDG - Offsite Power Transformer(OPT) - Backup ES Transformer (BEST)	**4.3 MODES: 1,2,3,4** Neither 4160V ES bus is capable of being energized within 15 minutes	**4.4 MODES: 1,2,3,4** **(1 and 2)** 1. Neither 4160V ES bus is capable of being energized **AND** 2. **(a or b)** a. Restoration of 4160V ES Bus A or 4160V ES Bus B is not likely within 4 hours **OR** b. Core exit thermocouples > 700°F as indicated on SPDS
Loss of AC Power (Shutdown) MODES: 5,6, No Mode (defueled)	*Not Applicable*	**4.5 MODES:5,6, No Mode** Neither 4160V ES bus is capable of being energized within 15 minutes	*Not Applicable*	*Not Applicable*
Loss of Vital DC Power MODES: 1,2,3,4	*Not Applicable*	*Not Applicable*	**4.6 MODES: 1,2,3,4** Standby Power Status Lights for BUS A1, A2, and BUS B1, B2 on the Main Control Board (SSF Panel) are out for 15 minutes or longer	*Refer to Fission Product Barrier Matrix*
Loss of Vital DC Power (Shutdown) MODES: 5,6, No Mode (defueled)	**4.7 MODES: 5,6, No Mode** Standby Power Status Lights for BUS A1, A2, and BUS B1, B2 on the Main Control Board (SSF Panel) are out for 15 minutes or longer	*Not Applicable*	*Not Applicable*	*Not Applicable*

FLORIDA NUCLEAR PLANT EMERGENCY NOTIFICATION FORM

1. THIS IS CRYSTAL RIVER UNIT 3. A. ☐ THIS IS A DRILL. B. ☐ THIS IS AN ACTUAL EVENT. I HAVE A MESSAGE.
ENSURE: ☐ STATE ☐ CITRUS ☐ LEVY ☐ RAD. CONTROL-ORLANDO (M-F ONLY) ARE ON LINE.

A. Time/Date contact made _____ B. Reported by: (Name/Title) _____

C. Message Number _____ D. Reported from: ☐ Control Room ☐ TSC ☐ EOF

3. SITE A. ☐ CRYSTAL RIVER UNIT 3 B. ☐ ST LUCIE UNIT 1 D. ☐ TURKEY POINT UNIT 3
C. ☐ ST LUCIE UNIT 2 E. ☐ TURKEY POINT UNIT 4

4. ACCIDENT CLASSIFICATION A. ☐ NOTIFICATION OF UNUSUAL EVENT C. ☐ SITE AREA EMERGENCY
B. ☐ ALERT D. ☐ GENERAL EMERGENCY

5. CURRENT EMERGENCY DECLARATION: TIME _____ DATE _____

6. REASON FOR EMERGENCY DECLARATION: _____

7. ADDITIONAL INFORMATION OR UPDATE: _____

8. INJURIES REQUIRING OFFSITE SUPPORT: A. ☐No ☐Yes ☐Unk B. Contaminated: ☐No ☐Yes ☐Unk

9. WEATHER DATA: A. Wind direction from _____ degrees.
B. Downwind Sectors affected (minimum of 3): ____, ____, ____

10. RELEASE STATUS: A. ☐ No Release (Go to Item 12) C. ☐ A Release occurred, but stopped
B. ☐ A Release is occurring

11. OFFSITE RELEASE SIGNIFICANCE CATEGORY (at the Site Boundary)

A. ☐ Information not available at this time.

B. ☐ Release within normal operating limits (Tech Specs/ODCM)

C. ☐ Non-Significant (Fraction of PAG Range, release is > normal limits and <PAG levels)

D. ☐ PAG Range (Protective Actions required)

12. UTILITY RECOMMENDED PROTECTIVE ACTIONS

A. ☐ NONE B. ☐ SHELTER ZONES/AREAS: _____

EVACUATE ZONES/AREAS: _____

OR C. ☐

MILES	NO ACTION	EVACUATE SECTORS	SHELTER SECTORS
0 – 2			
2 – 5			
5 – 10			

13. HAS EVENT BEEN TERMINATED?: A. ☐ NO B. ☐ YES: Time _____ Date _____

14. SUPPLEMENTAL FORM IS ATTACHED?: A. ☐ NO B. ☐ YES

15. MESSAGE RECEIVED BY: Name _____ Time _____ Date _____
THIS IS CRYSTAL RIVER UNIT 3. ☐ THIS IS A DRILL. ☐ THIS IS AN ACTUAL EVENT. END OF MESSAGE.

'EOF DIRECTOR INITIALS: _____

SUPPLEMENTAL DATA SHEET

The following supplemental data is to be completed by the TSC or EOF for an Alert or higher emergency declaration.

Supplement to Message Number _____

PLANT CONDITIONS INFORMATION

RITICAL SAFETY FUNCTIONS:

A. REACTOR SHUTDOWN? ☐ YES ☐ NO

B. CORE ADEQUATELY COOLED? ☐ YES ☐ NO

C. ADEQUATE EMERGENCY POWER AVAILABLE (DIESELS OR OFFSITE SOURCE) ☐ YES ☐ NO

FISSION PRODUCT BARRIER STATUS: (Check one condition for each barrier)

BARRIER	•	INTACT	•	CHALLENGED	•	LOST	•	REGAINED
FUEL CLADDING		No indication of clad damage		Clad is intact but losing subcooling, water level, etc.		Clad has failed, indicated by high temps., high containment rad, etc		Cooling restored, no further degradation expected
PRI. REACTOR COOLANT SYSTEM		Leakage is within normal charging or makeup pump capacity		Leakage is within safety injection capacity		Leakage exceeds safety injection capacity		Leakage reduced to within injection capacity (system repaired)
CONTAINMENT		No evidence of containment leakage or tube rupture release is only through condenser		No leakage but containment pressure is at or above safety system actuation points		Evidence of containment leakage (known release path or rad surveys)		Repair efforts have isolated leak or containment pressure has reduced to stop leakage

COMPLETED BY:_____ **TIME:**_____ **DATE:**_____

RADIOLOGICAL DOSE ASSESSMENT DATA

1. RELEASE STATUS: A. ☐ No Release (no further data required) C. ☐ A Release occurred, but stopped

B. ☐ A Release is occurring

2. RELEASE RATE:

A. ☐ NOBLE GASES: _____ Curies per second ☐ Measured ☐ Default

B. ☐ IODINES: _____ Curies per second ☐ Measured ☐ Default

3. TYPE OF RELEASE:

A. ☐ AIRBORNE Time/Date Started:_____ B. ☐ LIQUID Time/Date Started:_____

Time /Date Stopped:_____ Time/Date Stopped:_____

4. PROJECTED OFFSITE DOSE RATE:

DISTANCE	THYROID DOSE RATE (CDE)	TOTAL DOSE RATE (TEDE)
1 Mile (Site Boundary)	A._____mrem/hr	B._____mrem/hr
2 Miles	C._____mrem/hr	D._____mrem/hr
5 Miles	E._____mrem/hr	F._____mrem/hr
10 Miles	G._____mrem/hr	H._____mrem/hr

5. WEATHER DATA (used for the above data):

A. Wind Direction from _____ degrees.

B. Wind Speed _____ MPH (2.24 X meters/sec.)

C. Stability Class_____ (Sigma Theta or Wind Range; See page 3 of 5)

OMPLETED BY: _____ **TIME:**_____ **DATE:**_____

STATE OF FLORIDA NOTIFICATION PROTOCOL
[NOCS 96024]

WITHIN 15 MINUTES of declaration of emergency classification, NOTIFY STATE WARNING POINT TALLAHASSEE. (This so notifies Citrus and Levy counties and the Department of Health, Bureau of Radiation Control (DHBRC)-lando. If information is not available, do not delay notification to State Warning Point Tallahassee.

Using one of the following communications networks listed by priority:

- STATE Hot Ringdown (SHRD) - Station 120 or 121
- Commercial Telephone System - 1-850-413-9911 or 1-800-320-0519 or 1-850-413-9900
- Florida Emergency Satellite Communication System - (ESATCOM)
- Local Government Radio (LGR) via Citrus County
- Portable Satellite Phone (Located in TSC cabinet)

If the Commercial Telephone is used for notification, a separate notification to Citrus (746-2555) and Levy County (1-352-486-5212 or 1-352-486-5111 after hours) is required.

When making the initial notification of an emergency condition to SWPT, report the current emergency classification declared at the time the notification is made. If prior to initial notification or since the previous notification conditions were met (even briefly) for a higher classification, explain in Incident Description or Update.

INITIAL NOTIFICATION

Once communications are established with the SWPT Duty Officer and the station roll call is complete, READ the message in its entirety and REPEAT information and answer questions as requested. FAX the State Form by using Group 1 from the Fax machine.

SECTORS AFFECTED

DEGREES	SECTORS	DEGREES	SECTORS	DEGREES	SECTORS
349-11 (349-371)	H J K	102-123 (462-483)	N P Q	214-236	B C D
12-33 (372-393)	J K L	124-146 (484-506)	P Q R	237-258	C D E
34-56 (394-416)	K L M	147-168 (507-528)	Q R A	259-281	D E F
57-78 (417-438)	L M N	169-191 (529-540)	R A B	282-303	E F G
79-101 (439-461)	M N P	192-213	A B C	304-326	F G H
				327-348	G H J

STABILITY CLASS

SIGMA THETA (deg)	WIND RANGE (deg)	STABILITY CLASS
≥ 22.5	≥ 135	A (most dispersed plume)
<22.5 to 17.5	134 to 105	B
<17.5 to 12.5	104 to 75	C
<12.5 to 7.5	74 to 45	D
< 7.5 to 3.8	44 to 23	E
<3.8 to 2.1	22 to 12	F
<2.1	<12	G

UPDATE NOTIFICATION

Update SWPT every sixty minutes after initial notification and upgrades of emergency classification.

The use of the FLORIDA NUCLEAR PLANT EMERGENCY NOTIFICATION FORM is required for:

- Initial notification that an emergency condition exists (Item 4)
- Any change in emergency classification (Item 4)
- Any change in Protective Action Recommendations (Item 12)
- Termination of an emergency classification (Item 13)

:her updated information not meeting the above criteria does not require the use of the Form.

The sixty minute update notification is still required with a statement there is no change from last update, unless the SWPT agrees to less frequent updates.

GUIDANCE FOR COMPLETING THE FLORIDA NUCLEAR PLANT EMERGENCY NOTIFICATION FORM

1. Select appropriate box based on a drill or actual event. Ensure offsite agencies are on -line. If not, separate notifications to Citrus and Levy County are required.

2. A. Enter the time contact is made with the State Warning Point or Risk County. This time must be within 15 minutes of the "Current Emergency Declaration" time or within 60 minutes of the previous notification if used for an update (Item 5).

   B. Enter name and title of person making the notification.

   C. Enter message number (beginning with #1 and following through sequentially in all facilities).

   D. Enter location from which the notification is made.

3. Check Crystal River Unit 3.

4. Check the classification corresponding to current plant conditions. If, prior to the initial notification or since the prev ious notification, conditions were met (ev en briefly) for a higher classification, ensure that classification and condition is noted in Item 7, "Additional Information or Update."

5. Enter the emergency declaration time and date for the current accident classification.

6. Enter wording to indicate the Emergency Action Level or Fission Pr oduct Barrier loss or potential loss used to declare the ev ent. This information should remain the same throughout update messages unless there is a classification change.

7. Enter additional significant events here, including if conditions briefly exist ed for a higher emergency classification but no longer exist, or conditions that would have independently warranted declaration of an equal or lower classification (e. g. a fire within the Protected Area during a SITE AREA or GENERAL EMERGENCY).

8. Item "A"; Check "YES" only if there are injuries that require off -site support (EMS, hospital). Check "Unk" if the extent of the injuries is unknown at this time or if it is not yet known if offsite treatment is necessary. Check "Unk" in item "B" if the nature of the injuries has prevented thorough monitoring onsite or if there is any doubt whether contamination is present.

9. Enter the wind direction in degrees in Item "A" and the three downwind sectors in Item "B." The downwind sectors confirm wind direction because of potential confusion with degrees "from" versus degrees "to."

10. **Check Item "A"** if there are no indications of a release, then go to Item 12. **Check Item "B"** if a release is occurring, even though it may be less than normal operating limits. **Check Item "C"** if a release has occurred but stopped. Specific dose information will be supplied on the supplemental data sheet after the TSC is declared operational at an ALERT or higher. **RELEASE:** (Refer to "Release (State Form)" definition)

11. **Check Item "A"** if Release Significance Category (See page 5 of 5) information is not available at the time of notification and follow up as soon as possible with information. **Check Item "B"** if the current release is or the previous release was within normal operating li mits (ITS/ODCM). Releases monitored by RM-A1 or RM -A2 are within normal operating limits if the low -range gas channel is below its high alarm setpoint. **Check Item "C"** if the current release is or the prev ious release was greater than normal operating limits, but less than EPA PAG values. This involves any radiological release that may occur when there is no fuel damage. No PARs are required at this level. C **heck Item "D"** if there is any indication of fuel damage (cladding failure or melt) and there is a ny indication of a release (effluent monitors, surv eys, etc.). PARs would be automatically required. This terminology should be easily understood by decision-makers at all levels within the utility and at the State and local levels.

12. **Check Item "A"** if no Protective Actions are necessary. **Check Item "B"** if PARs are necessary and enter Zone designation. (Item "C" is used by other Florida nuclear sites.)

13. Enter the time the event has been terminated or when the transition from the "Emergency Phase" to th e "Recovery Phase" has taken place.

14. Check "no" unless a Supplemental Form has been completed for this particular message. If a Supplemental Form is attached, the Form is to be read as part of the emergency notification from the TSC or EOF and faxed.

15. Enter the name of the SWPT Duty Officer or the individual that receives the notification. Enter time and date call is completed or when Form is provided to Deputy State Coordinating Officer at the EOF.

plemental Page - Complete at the TSC or EOF at an Al ert Classification or higher and provide to State & ocals with Page 1.

RELEASE SIGNIFICANCE CATEGORIES

CORE CONDITION	RELEASE STATUS	RELEASE SIG CATEGORY
No Core Damage	No release	NR
	Release in progress	<NOL, NS
Clad Failure	No release	NR
	Release in progress	PAG
Core Melt	No release	NR
	Release in progress	EHE (PAG* State Form)

NR: NO RELEASE
This category indicates no release is occurring. This category is appropriate regardless of core status, if there are no indications of a release (e.g., unexplained containment pressure decrease, unexplained abnormal radiation levels in Auxiliary Building or Intermediate Building, on the berm, or in the field). Do not assume Design Basis Leakage is occurring if it has not been detected. If a release occurred but has now stopped, maintain the appropriate category below until EPZ doses have dissipated.

<NOL: RELEASE WITHIN NORMAL OPERATING LIMITS (ITS/ODCM)
This category indicates releases that are monitored by RM-A1 or RM-A2, occurring when the fuel is undamaged. These releases are within normal operating limits if the low-range gas channel is below its high alarm setpoint. Do not make this selection for releases not monitored by RM-A1 or RM-A2 unless they have been evaluated per the ODCM.

NS: NON-SIGNIFICANT (FRACTION OF PROTECTIVE ACTION GUIDELINE VALUES)
This category indicates releases that are occurring when the fuel is undamaged. It includes releases exceeding RM-A1 or RM-A2 high alarm setpoint and releases not monitored by RM-A1 or RM-A2 (e.g., releases due to LOCA, Waste Gas System failures, and steam generator tube ruptures). These releases will not produce site boundary doses that approach the EPA Protective Action Guideline values of 1 REM TEDE and/or 5 REM thyroid. No Protective Action Recommendations are necessary.

PAG: AT OR NEAR PROTECTIVE ACTION GUIDELINE VALUES
This category indicates releases that are occurring after at least some fuel cladding failure has taken place. It includes damage to irradiated fuel stored in the fuel pools. Site Boundary doses greater than the EPA Protective Action Guideline of 1 REM TEDE and/or 5 REM thyroid are possible. The category is appropriate with fuel cladding failure even if only minor offsite doses are detected. Evacuation of at least 5 miles, 360° (Zone 1) should be recommended. Shelter or evacuation beyond 5 miles should be determined based on plant status and dose projections.

EHE: EARLY HEALTH EFFECTS (not on State Notification Form, see NOTE below)
This category indicates releases that are occurring after severe core damage has taken place and where containment has failed early in the event. Doses of 25 REM TEDE and/or 2500 RADS thyroid could cause early health effects and these doses are easily possible within three miles from the plant. Evacuation of the Energy Complex should be performed and evacuation of the 10-mile EPZ (Zones 1,2,3) should be recommended (never sheltering) even if evacuees are exposed to the plume.

* NOTE: This category is not listed on the State Notification Form because the State implements protective actions at the PAG range above. However, it will be posted on status boards in the TSC and EOF.

CONSIDERATIONS FOR A SECURITY EMERGENCY

****

CAUTION: Security may use force (including deadly force) to protect plant personnel and vital plant equipment. Personnel are not to move around inside the protected area unless directed to do so by Operations or Security, or until the Security Emergency is cleared.

****

- Security notifies the Control Room of the Security Emergency.

  OR

- IF Security is not aware of the potential Security Emergency, THEN NOTIFY the Security Shift Supervisor immediately.

- In concurrence with Security, the Control Room announces the Security Emergency directing personnel to secure equipment and take suitable cover immediately, until an "All Clear" announcement is made.

- MAINTAIN contact between the Control Room and the Security Shift Supervisor.

- RETAIN personnel in the Control Room and await instructions from the Security Shift Supervisor.

- Unless concurred with by the Security Shift Supervisor:

  - DO NOT SOUND the evacuation alarm.

  - DO NOT staff the TSC/OSC (unless already activated).

  - DO NOT call in TSC staff by phone or pagers.

  - DO NOT instruct personnel to go to their Local or Main Assembly Areas.

- MAINTAIN EC duties and functions in the Control Room until the EC determines is safe to SOUND the evacuation alarms and/or staff the TSC/OSC.

- RETURN to the appropriate procedure section to continue making appropriate notifications, except as identified above.

1) Use ENS phone sticker # for NRC direct
IF ENS OUT OF SERVICE , use
2) Commercial 1-301-816-5100 or
1-301-951-0550

NRC EVENT NOTIFICATION WORKSHEET

NRC COMMUNICATOR_____ NRC EVENT

NOTIFICATION TIME	FACILITY	UNIT	CALLER'S NAME	CALL BACK ENS # 700-821-0027 Or # 1-352-795-6958
	CRYSTAL RIVER	3		
EVENT TIME	**EVENT DATE**		**POWER/MODE BEFORE**	**POWER/MODE AFTER**

EVENT CLASSIFICATIONS - 50.72 (a)(1)	1-HOUR NON-EMERGENCY 50.72 (b)(1) (Cont'd)	4 HOUR NON-EMERGENCY 50.72 (b)(2) (Cont'd)
☐ GENERAL EMERGENCY	☐ (iii) Tornado	☐ (iii)(B) RHR Capability
☐ SITE AREA EMERGENCY	☐ (iii) Other Natural Phenomena	☐ (iii)(C) Control of Rad Release
☐ ALERT	☐ (iv) ECCS Discharge to RCS	☐ (iii)(D) Accident Mitigation
☐ UNUSUAL EVENT	☐ (v) Lost ENS	☐ (iv)(A) Air Release >20X Appendix B
1-HOUR NON-EMERGENCY 50.72(b)(1)	☐ (v) Lost Emergency Assessment	☐ (iv)(B) Liq Release >20X Appendix B
	☐ (v) Lost Offsite Communications	☐ (v) Offsite Medical
☐ (i)(A) TS Required Shut Down	☐ (v) Emergency Sirens Inoperable	☐ (vi) Offsite Notification
☐ (i)(B) TS Deviation	☐ (vi) Fire	**OTHER EVENTS**
☐ (ii) Degraded Condition	☐ (vi) Toxic Gas	
(ii)(A) Unanalyzed Condition	☐ (vi) Rad Release	☐ PHYSICAL SECURITY (73.71)
☐ (ii)(B) Outside Design Basis	☐ (vi) Other Hampering Safe Operation	☐ TRANSPORTATION
☐ (ii)(C) Not Covered By OPs/EPs	**4 HOUR NON-EMERGENCY 50.72 (b)(2)**	☐ MATERIAL/EXPOSURE (20.2202)
☐ (iii) Earthquake		☐ FITNESS FOR DUTY
☐ (iii) Flood	☐ (i) Degrade While Shut Down	☐ OTHER
☐ (iii) Hurricane	☐ (ii) RPS Actuation (scram)	
☐ (iii) Ice/Hail	☐ (ii) ESF Actuation	
☐ (iii) Lightning	☐ (iii)(A) Safe Shut/Down Capability	

DESCRIPTION

Include: Systems affected, actuations & their initiating signals, causes, effect of event on plant, actions taken or planned, etc.

NOTIFICATIONS	YES	NO	WILL BE					
NRC RESIDENT				ANYTHING UNUSUAL OR NOT UNDERSTOOD?		YES (explain above)		NO
STATE WARNING POINT				DID ALL SYSTEMS FUNCTION AS REQUIRED?		YES		NO (explain above)
ITRUS&LEVY COUNTIES								
STATE BUREAU OF RADIATION CONTROL				MODE OF OPERATION UNTIL CORRECTED:	ESTIMATE FOR RESTART DATE:			ADDITIONAL INFO ON BACK?
MEDIA/PRESS RELEASE								

NOTIFICATION TIME _____

ADDITIONAL INFORMATION

Radiological Release Information	Check or Fill in Applicable Items **(specific details/explanations should be covered in event description)**		
Offsite Release *(State release path in description)*	☐ Offsite Protective Actions Recommended	☐ Areas Evacuated	☐ Personnel Exposed or Contaminated

Release Description	Check or Fill in Applicable Items **(specific details/explanations should be covered in event description)**							
Liquid Release	☐ Monitored	☐ Unmonitored	☐ Planned	☐ Unplanned	☐ Ongoing	☐ Terminated	☐ ODCM Exceeded	☐ RM Alarms
Gaseous Release	☐ Monitored	☐ Unmonitored	☐ Planned	☐ Unplanned	☐ Ongoing	☐ Terminated	☐ ODCM Exceeded	☐ RM Alarms

Release Rates/Limits (From Dose Assessment Team)	**Release Rate** (Ci/sec)	**% ODCM Limit**	**Total Activity** (Ci)	**% ODCM Limit**
Noble Gas				
Iodine				
Particulate				
Liquid *(excluding tritium & dissolved noble gases)*				
Liquid (tritium)				
Total Activity				

Monitor Readings	**Plant Stack** (RMA-2)	**Condenser/Air Ejector** (RMA-12)	**Main Steam Line** (RMG-25,26,27,28)	**Other (List)**
RAD Monitor Readings:				
Alarm Setpoints:				
% ODCM Limit *(if applicable)*				

RCS or SG Tube Leaks	Check or Fill In Applicable Items: ***(specific details/explanations should be covered in event description)***		
Location of the leak (e.g., SG#, valve, pipe, etc.)			
Leak Rate:	Units: gpm/gpd	S.T.S. Limits:	☐ Sudden or ☐ Long Term Development
Leak Start Date:	Time:	Coolant Activity Primary µCi/ml Secondary µCi/ml	
List of Safety Related Equipment Not Operational:			

EVENT DESCRIPTION (Continued from front)

_____

_____

_____

_____

_____

EC INITIALS _____ **DATE:**

EMERGENCY NOTIFICATION UNITS 1, 2, 4 & 5

Use Enclosure 7 to determine protective action recommendations for Energy Complex personnel. (None for Unusual Event or Alert.)

Unit 1 & 2 (ext. 2120 or 563-4454) Contact _____ Time _____

Unit 4 & 5 (ext. 5283 or 563-4460) Contact _____ Time _____

GIVE THE FOLLOWING INFORMATION TO THE FOSSIL UNITS:

1. Your name and position: _____

2. Emergency or drill: _____

3. Current Emergency Classification: _____

4. Briefly explain plant conditions using basic facts: _____

_____

5. STATE (a) or (b):

   (a) "NO RADIOACTIVE MATERIAL WAS RELEASED."

   (b) "RADIOACTIVE MATERIAL IS BEING RELEASED."

6. STATE if conditions are:

   a. "IMPROVING"
   b. "STABLE"
   c. "DEGRADING"

7. STATE (a) or (b) or (c) or (d):

   a. (IF UNUSUAL EVENT OR ALERT) "NO ASSEMBLY OR EVACUATION IS NECESSARY AT THIS TIME."

   b. (SITE AREA EMERGENCY; see Enclosure 7) "BEGIN STANDARD ASSEMBLY AND ACCOUNTABILITY. REFER TO THE CRYSTAL RIVER COAL PLANT SITE ACCOUNTABILITY/EVACUATION MANUAL. ONCE ACCOUNTABILITY IS COMPLETE, NOTIFY CR-3 SECURITY AT EXTENSION 3258, AND STANDBY FOR FURTHER INSTRUCTIONS."

   c. (General Emergency, no release and release not likely within 3 hrs; see Enclosure 7) "BEGIN STANDARD ASSEMBLY AND ACCOUNTABILITY. REFER TO THE CRYSTAL RIVER COAL PLANT SITE ACCOUNTABILITY/EVACUATION MANUAL. ONCE ACCOUNTABILITY IS COMPLETE, NOTIFY CR-3 SECURITY AT EXTENSION 3258, AND EVACUATE NON-ESSENTIAL PERSONNEL. STANDBY FOR FURTHER INSTRUCTION."

   d. (General Emergency, release has occurred or is likely to occur within 3 hours; see Enclosure 7) "SECURE THE PLANT AND EVACUATE. DO NOT PERFORM ASSEMBLY."

8. If time permits and you feel qualified, ask for questions.

9. STATE: "WE WILL KEEP YOU INFORMED."

INITIATION OF THE EMERGENCY RESPONSE DATA SYSTEM (ERDS) [NOCS 40730]

WITHIN THE FIRST HOUR of the declaration of an ALERT, SITE AREA EMERGENCY or GENERAL EMERGENCY classification ACTIVATE ERDS. Once activated, ERDS operates automatically.

ERDS IS LOCATED in the Control Room in the Cabinet labeled:

"Computer Main Frame,
Cab. #5"

ACTIVATION OF ERDS - Open the cabinet and perform the following:

1) Push button "B" on the COMMANDER for ERDS initiation. Make sure the red light comes on.

2) ALT-TAB to ERDS Display.

3) Push ALT-C on the keyboard.

The ERDS window will display a series of messages such as "Waiting for Connect" and "Waiting for Accept." Once the connection with the NRC has been established, the messages will alternate between "Transmitting" and "Idle." If no activation response is indicated on the monitor, contact the Nuclear Computer Controls Specialist for assistance, and NOTIFY the NRC over the ENS link, providing parameters as requested. If the link is inadvertently terminated once communications are established, ERDS automatically continues trying to reestablish communications.

DEACTIVATION OF ERDS

NOTIFY the NRC before disconnecting the ERDS data link. Once concurrence is given by the NRC, ERDS transmission is terminated. If one of the above mentioned message is not on the monitor, it means that ERDS is not activated. Proceed with the following only if ERDS is still activated. The NRC also has the capability of terminating the ERDS transmission if needed.

1) Push button "B" on the COMMANDER for ERDS deactivation.

2) ALT-TAB to ERDS Display

3) Push ALT-C on the keyboard.

A series of messages will appear in the ERDS window. When the shut down is finished, the message "Shutdown Completed" will be displayed.

EVACUATION PLANNING GUIDE

ENERGY COMPLEX PROTECTIVE ACTIONS

1. DETERMINE protective actions for the Energy Complex using B or C or D below. (Use information in the tables and map on the following pages of this enclosure as necessary.)

   A. UNUSUAL EVENT OR ALERT: NO PROTECTIVE ACTIONS

   B. SITE AREA EMERGENCY:

   - Perform assembly and accountability and instruct Fossil Control Rooms to report results to CR-3 Security at extension 3258.
   - Consider sheltering for releases lasting less than two hours.
   - For releases lasting greater than two hours or for planned releases evacuate non-essential personnel.

   C. GENERAL EMERGENCY:
   (Release has not occurred and release not likely within 3 hours.)

   - Perform assembly and accountability and instruct Fossil Control Rooms to report results to CR-3 Security at extension 3258.
   - Evacuate non-essential personnel (including MAA personnel).
   - NOTIFY fossil control rooms to standby for instructions.
   - Consider supplying dosimetry to remaining personnel.

   D. GENERAL EMERGENCY:
   (Release has occurred or is likely to occur within 3 hours.)

   - NOTIFY fossil control rooms to secure their plants.
   - Evacuate the Energy Complex even if a release has already started (including MAA personnel).
   - Evacuate without performing assembly.

2. ENSURE the fossil units are notified using Enclosure 5, per the EC's Guide.

3. ENSURE Nuclear Security coordinates with Corporate Security to ensure these protective action instructions are provided to all areas of the Energy Complex, per the EC Guide.

Evacuation Considerations:
- Approximately 35 minutes for notification, equipment shutdown, assembly and accountability.
- Approximately 125 minutes to evacuate site using Access Road.*
- Approximately 160 minutes to evacuate site during adverse weather.* - Consider a suitable evacuation route from the site.
- Early evacuation may be required under certain meteorological or radiological conditions.

* Based on 1344 vehicles on-site (approximately 700 - 1000 vehicles on the Energy Complex during normal operations).

EVACUATION PLANNING GUIDE

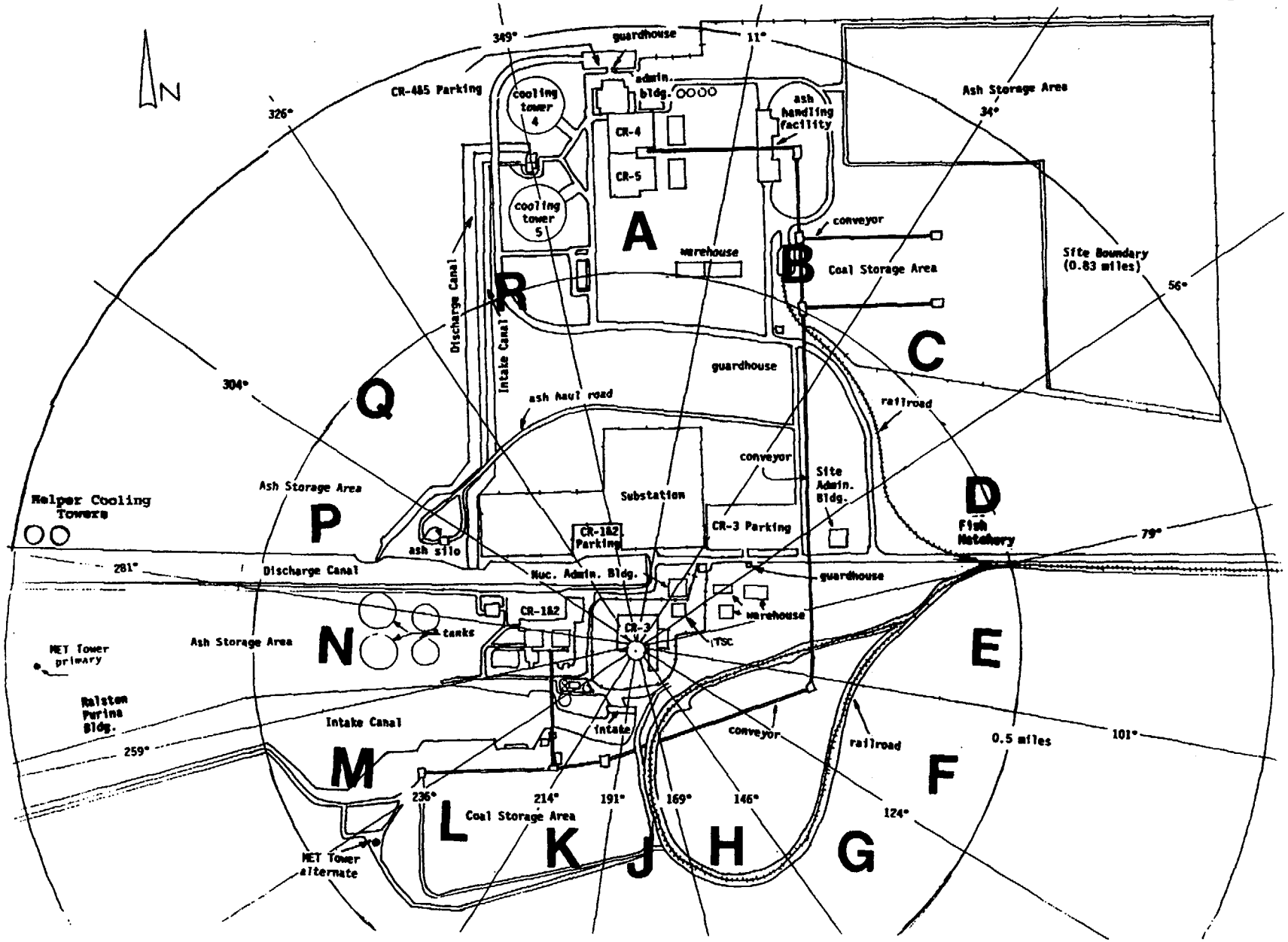
WIND DIRECTION DATA

WIND FROM DIRECTION	WIND FROM DEGREES	SECTORS AFFECTED
N	349-11 (349-371)	H J K
NNE	12-33 (372-393)	J K L
NE	34-56 (394-416)	K L M
ENE	57-78 (417-438)	L M N
E	79-101 (439-461)	M N P
ESE	102-123 (462-483)	N P Q
SE	124-146 (484-506)	P Q R
SSE	147-168 (507-528)	Q R A
S	169-191 (529-540)	R A B
SSW	192-213	A B C
SW	214-236	B C D
WSW	237-258	C D E
W	259-281	D E F
WNW	282-303	E F G
NW	304-326	F G H
NNW	327-348	G H J

EVACUATION PLANNING GUIDE

CONTACTS FOR PERSONNEL ASSEMBLY

SECTOR	AREA	CONTACT
A	Units 4 & 5	Units 4 & 5 Control Room
B / C	Nuclear Administration Bldg.	Public Address System
B / C	North Coal Yard	Units 4 & 5 Control Room
D / E	CR-3 Warehouse Area Site Administration Building	Corporate Security Specialist
D / E	Mariculture Center	Corporate Security Specialist
E / F / G / H	Coal Train Yard	Units 4 & 5 Control Room
J / K / L	South Coal Yard	Units 1 & 2 Control Room
N	Units 1 & 2	Units 1 & 2 Control Room
N	Ralston Purina Building	Corporate Security Specialist


N
349°
guardhouse
11°
CR-485 Parking
admin. bldg.
cooling tower 4
ash handling facility
Ash Storage Area
34°
326°
CR-4
CR-5
cooling tower 5
A
conveyor
warehouse
B
Coal Storage Area
Site Boundary (0.83 miles)
56°
R
Discharge Canal
Intake Canal
C
guardhouse
304°
Q
ash haul road
railroad
conveyor
Site Admin. Bldg.
D
Fish Hatchery
Substation
Ash Storage Area
Helper Cooling Towers
P
ash silo
CR-1&2 Parking
CR-3 Parking
79°
281°
Discharge Canal
Nuc. Admin. Bldg.
guardhouse
CR-1&2
CR-3
warehouse
tanks
TSC
Ash Storage Area
N
E
MET Tower primary
Ralston Purina Bldg.
Intake Canal
intake
conveyor
railroad
0.5 miles
101°
259°
M
F
236°
214°
191°
169°
146°
124°
L
Coal Storage Area
K
J
H
G
MET Tower alternate

GUIDELINES FOR PROTECTIVE ACTION RECOMMENDATIONS FOR NON-ESSENTIAL GENERATING COMPLEX PERSONNEL AND GENERAL POPULATION

[NOCS 1128, 1592]

PLANT CONDITIONS/OFF-SITE DOSE ESTIMATES	RECOMMENDED ACTION 0-5 MILES	RECOMMENDED ACTION 5-10 MILES
1. **CONDITION:** **GENERAL EMERGENCY DECLARED.** **NO APPARENT CORE DAMAGE.** CORE DAMAGE INDICATIONS: a. RCS pressure vs temperature in Region 1 or 2 (Refer to EOP-07); or b. RM-G29/30 reading < 100 R/hr; or c. PASS results.	Evacuate Zone 1 (See Note 2.)	None (See Note 1.)
2. **CONDITION:** **GENERAL EMERGENCY DECLARED.** **CLAD DAMAGE/GAS GAP RELEASE (NO CORE MELT).** CORE DAMAGE INDICATIONS: a. RCS pressure vs temperature in Region 3 (Refer to EOP-07); or b. Core uncovered for 15-30 minutes; or c. RM-G29/30 reading of 100-75,000 R/hr (RB spray off) OR 100-25,000 R/hr (RB spray on); or d. PASS results. **OR:** * Dose at the 0.83 mile Site Boundary is projected to be: a) TEDE: ≥ 1.0 Rem b) Thyroid CDE: ≥ 5.0 Rem	Evacuate Zone 1 (See Note 2.)	Shelter Zones 2 & 3 (See Note 1.)
3. **CONDITION:** **GENERAL EMERGENCY DECLARED.** **CORE MELT OCCURRING OR LIKELY.** CORE DAMAGE INDICATIONS: a. RCS pressure vs temperature in the Severe Accident Region (Refer to EOP-07); or b. Core uncovered for > 30 minutes; or c. RM-G29/30 reading > 75,000 R/hr (RB spray off) or > 25,000 R/hr (RB spray on). **WITH:** NO projected containment failure and NO release underway.	Evacuate Zone 1 (See Note 2.)	Shelter Zones 2 & 3 (See Note 1.)
Projected containment failure and/or release underway.	Evacuate Zone 1 (See Note 2.)	Evacuate Zones 2 & 3 (See Note 2.) (See Note 3.)

*** PARs within the first hour of an event should be based on PLANT CONDITIONS ONLY until the Dose Assessment Team is operational.**

NOTE 1: Relocate/evacuate population affected by ground contamination after plume passage or at any time projected dose from actual release is ≥ 1.0 REM TEDE or ≥ 5.0 REM Thyroid CDE.

NOTE 2: Evacuation time estimates are 2 hours for a Zone 1 evacuation and 4 hours for Zones 2 & 3 evacuation. (These times do not include notification or preparation time for evacuees.)

NOTE 3: IF projected dose from an actual release is >1.0 REM TEDE or 5.0 REM Thyroid beyond 10 miles, THEN RECOMMEND evacuation to State and local government by distance in miles, OR by subdivision and geographic boundaries.

ZONE DESCRIPTIONS

Zone 1: 0-5 miles 360° and out to 10 miles in Gulf.
Zone 2: 5-10 miles in Citrus County.
Zone 3: 5-10 miles in Levy County.

GUIDELINES FOR FPC EMERGENCY WORKER EXPOSURE

CONDITION	DOSE LIMIT (REM TEDE)	GUIDANCE
1. Emergency conditions not requiring actions to prevent serious injury or protect valuable property.	5	Emergency worker exposure should not exceed 5 REM TEDE.
2. Emergency conditions requiring actions to prevent serious injury or protect valuable property.	10	Exposure greater than 5 REM TEDE should receive approval of the Emergency Coordinator. Appropriate controls for emergency workers include time limitations and respirators.
3. Emergency conditions requiring lifesaving actions or actions to protect large populations.	25	Exposure greater than 5 REM TEDE should receive approval of the Emergency Coordinator. Appropriate controls for emergency workers include time limitations, respirators, and thyroid blocking.
4. Emergency conditions requiring lifesaving actions or actions to protect large populations.	> 25	Exposure greater than 5 REM TEDE receive approval of the Emergency Coordinator. Exposure at this level should be to volunteers who are healthy, above the age of 45, have an understanding of the health risks involved, and, preferably, be those whose normal duties have trained them for such missions. Appropriate controls for emergency workers include time limitations, respirators, and thyroid blocking.

NOTE: Reference for this table is Table 2.2 in the Manual of Protective Action Guides and Protective Actions for Nuclear Incidents (EPA 400-R/92-001).

PROCEDURE DEVELOPMENT AND REVISION RECORD

Procedure: EM0202 **New Rev:** 67 **PRR#:** 20290

Title: DUTIES OF THE EMERGENCY COORDINATOR

MINOR CHANGES

If Minor Changes are included, check the applicable box(es) and provide a list of affected steps.
The following corrections are incorporated throughout:

- __ Sentence Structure
- __ Punctuation
- __ Capitalization
- __ Spelling
- __ Organizational Changes: position titles, department names, or telephone numbers
- __ Redundant words or phrases
- __ Abbreviations
- __ Obviously incorrect units of measure
- __ Inadvertently omitted symbols (#, %, etc.)
- __ Obvious step numbering discrepancies
- __ Format

The following corrections are incorporated in the step(s) indicated: "Throughout" is used in lieu of Step# if a specific change affects a large number of steps.

	Changing information that is obviously incorrect and referenced correctly elsewhere
	Misplaced decimals that are neither setpoint values nor tolerances
	Reference to a procedure when an approved procedure has taken the place of another procedure
	Fixing branching points when it is clear the branching steps were originally intended but were overlooked or incorrectly stated due to step number changes
	Adding clarifying information such as NOTES and CAUTIONS
	Adding words to clarify steps, NOTES, or CAUTIONS which clearly do not change the methodology or intent of the steps

Procedure: EM0202 **New Rev:** 67 **PRR#:** 20290

Title: DUTIES OF THE EMERGENCY COORDINATOR

NON-INTENT CHANGES

Changes are incorporated for the reasons provided. "Throughout" is used in lieu of Step # if a specific change affects a large number of steps. For new or cancelled procedures the reason is provided.

EAL 3.8, delete the words "unplanned" from the EAL	To make the EAL wording consistent with the NEI 99-01 guidance. The EAL should not differentiate between a planned or unplanned loss at a Site Area Emergency since neither are tolerable based on plant conditions. This was identified by an NRC Inspector based on a review of the RERP. This item captured by PC 01-0884.