



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

June 29, 1992

Docket No. 50-220

Mr. B. Ralph Sylvia
Executive Vice President, Nuclear
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Dear Mr. Sylvia:

SUBJECT: ISSUANCE OF AMENDMENT FOR NINE MILE POINT NUCLEAR STATION UNIT NO. 1
(TAC NO. M83282)

The Commission has issued the enclosed Amendment No. 129 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station Unit No. 1 (NMP-1). The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated April 24, 1992.

The amendment revises Technical Specification 3.4.3/4.4.3 (Access Control) to:
(a) define the operating conditions under which the specification applies,
(b) include an allowable outage time for continued plant operation while restoration of secondary containment integrity is underway, (c) provide action statements associated with the loss of secondary containment due to access control, and (d) provide periodic surveillance requirements for access doors other than the core spray and containment spray pump compartments.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Donald S. Brinkman, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.129 to DPR-63
2. Safety Evaluation

cc w/enclosures:
See next page

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P PDR

Mr. B. Ralph Sylvia
Niagara Mohawk Power Corporation

Nine Mile Point Nuclear Station
Unit No. 1

cc:

Mark J. Wetterhahn, Esquire
Winston & Strawn
1400 L Street, NW
Washington, DC 20005-3502

Mr. Kim Dahlberg
Unit 1 Station Superintendent
Nine Mile Point Nuclear Station
Post Office Box 32
Lycoming, New York 13093

Supervisor
Town of Scriba
Route 8, Box 382
Oswego, New York 13126

Mr. David K. Greene
Manager Licensing
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Mr. Joseph F. Firlit
Vice President - Nuclear Generation
Niagara Mohawk Power Corporation
Nine Mile Point Nuclear Station
Post Office Box 32
Lycoming, New York 13093

Charles Donaldson, Esquire
Assistant Attorney General
New York Department of Law
120 Broadway
New York, New York 10271

Resident Inspector
U.S. Nuclear Regulatory Commission
Post Office Box 126
Lycoming, New York 13093

Mr. Paul D. Eddy
State of New York
Department of Public Service
Power Division, System Operations
3 Empire State Plaza
Albany, New York 12223

Gary D. Wilson, Esquire
Niagara Mohawk Power Corporation
300 Erie Boulevard West
Syracuse, New York 13202

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

Ms. Donna Ross
New York State Energy Office
2 Empire State Plaza
16th Floor
Albany, New York 12223

DATED: June 29, 1992

AMENDMENT NO. 129 TO FACILITY OPERATING LICENSE NO. DPR-63-NINE MILE POINT
UNIT 1

Docket File
NRC & Local PDRs
PDI-1 Reading
S. Varga, 14/E/4
J. Calvo, 14/A/4
R. Capra
C. Vogan
D. Brinkman
OGC-WF
D. Hagan, 3302 MNBB
G. Hill (4), P-137
Wanda Jones, P-130A
C. Grimes, 11/F/23
ACRS (10)
OPA
OC/LFMB
PD plant-specific file
C. Cowgill, Region I
C. McCracken, 8/D/1

cc: Plant Service list



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 129
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated April 24, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 129, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Donald A. Brinkman
for

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 29, 1992

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 129 TO FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise Appendix A as follows:

Remove Pages

171

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-

172

Insert Pages

171

171a (added page)

171b (added page)

172

LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENT
<p>3.4.3 <u>ACCESS CONTROL</u></p> <p><u>Applicability:</u></p> <p>Applies to the access control to the reactor building.</p> <p><u>Objective:</u></p> <p>To specify the requirements necessary to assure the integrity of the secondary containment system.</p> <p><u>Specification:</u></p> <p>a. Whenever the reactor is in the power operating condition, or when irradiated fuel is being handled in the reactor building, or during core alterations, or during irradiated fuel cask handling operations in the reactor building, the following conditions will be met:</p> <ol style="list-style-type: none"> 1. Only one door in each of the double-doored access ways shall be opened at one time. 2. Only one door or closeup of the railroad bay shall be opened at one time. 	<p>4.4.3 <u>ACCESS CONTROL</u></p> <p><u>Applicability:</u></p> <p>Applies to the periodic checking of the condition of portions of the reactor building.</p> <p><u>Objective:</u></p> <p>To assure that pump compartments are properly closed at all times and to assure the integrity of the secondary containment system by verifying that reactor building access doors are closed, as required by Specifications 3.4.3.a.1 and 3.4.3.a.2.</p> <p><u>Specification:</u></p> <ol style="list-style-type: none"> a. The core and containment spray pump compartments shall be checked once per week and after each entry. b. Verify at least once per 31 days that: <ol style="list-style-type: none"> 1. At least one door in each access to the secondary containment is closed. 2. At least one door or closeup of the railroad bay is closed.

LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENT
<p data-bbox="527 253 1052 488">3. The core spray and containment spray pump compartments' doors shall be closed at all times except during passage in order to consider the core spray system and the containment spray system operable.</p> <p data-bbox="436 529 1052 589">b. If these conditions cannot be met, then the actions listed below shall be taken:</p> <p data-bbox="527 634 1052 894">1. If in the power operating condition, restore reactor building integrity within 4 hours or be in at least the hot shutdown condition within the next 12 hours and in the cold shutdown condition within the following 24 hours.</p> <p data-bbox="527 971 1010 1031">2. Suspend any of the following activities:</p> <p data-bbox="621 1073 926 1101">a. core alterations,</p> <p data-bbox="621 1141 999 1239">b. Handling of irradiated fuel in the reactor building,</p>	

LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENT
c. irradiated fuel cask handling operations in the reactor building.	

BASES FOR 3.4.3 AND 4.4.3 ACCESS CONTROL

The reactor building serves as a secondary containment during normal Station operations and as a primary containment during refueling and other periods when the pressure suppression system is open. Maintaining the building integrity and an operative emergency ventilation system for the conditions listed will ensure that any fission products inadvertently released to the reactor building will be routed through the emergency ventilation system to the stack. The worst such incident is due to dropping a fuel assembly on the core during refueling. The consequences of this are discussed in Section XV.C.3 of the FSAR.

As discussed in Section VI-F* all access openings of the reactor building have as a minimum two doors in series. Appropriate local alarms and control room indicators are provided to always insure that reactor building integrity is maintained. Surveillance of the reactor building access doors provides additional assurance that reactor building integrity is maintained.

Maintaining closed doors on the pump compartments ensures that suction to the core and containment spray pumps is not lost in case of a gross leak from the suppression chamber.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 129 TO FACILITY OPERATING LICENSE NO. DPR-63

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION UNIT NO. 1

DOCKET NO. 50-220

1.0 INTRODUCTION

By letter dated April 24, 1992, Niagara Mohawk Power Corporation (the licensee) submitted a request for changes to the Nine Mile Point Nuclear Station Unit No. 1 (NMP-1), Technical Specifications (TS). The requested changes would revise Technical Specification 3.4.3/4.4.3 (Access Control) to (a) define the operating conditions under which the specification applies, (b) include an allowable outage time for continued plant operation while restoration of secondary containment integrity is underway, (c) provide action statements associated with the loss of secondary containment due to access control, and (d) provide periodic surveillance requirements for access doors other than the core spray and containment spray pump compartments.

2.0 EVALUATION

Currently, NMP-1 TS 3.4.3/4.4.3 (Access Control) does not: (1) explicitly specify when the Limiting Condition for Operation (LCO) is applicable, (2) specify an allowable outage time for continued plant operation while restoration of secondary containment integrity is underway, (3) provide action statements (corrective action instructions) to be followed in the event of a loss of secondary containment integrity due to a loss of access control, or (4) provide periodic surveillance requirements for secondary containment access doors other than the core spray and containment spray pump compartments. The proposed change would add these provisions to NMP-1 TS 3.4.3/4.4.3.

The proposed changes would specify that the requirements of TS 3.4.3/4.4.3 would be applicable: (1) whenever the reactor is in the power operating condition, or (2) whenever irradiated fuel is being handled in the reactor building, or (3) during core alterations, or (4) during irradiated fuel cask handling operations in the reactor building. These proposed applicability requirements are consistent with the applicability guidelines of the NRC's General Electric Standard Technical Specifications (GE-STs) for similar applications and with existing NMP-1 TS restrictions associated with reactor building (secondary containment) integrity. The NRC staff finds that the

proposed applicability requirements are appropriate for NMP-1 and are, therefore, acceptable.

The proposed allowable outage time and action statement requirements would require restoration of reactor building (secondary containment) integrity within 4 hours of loss of integrity or the reactor shall be in at least the hot shutdown condition within the next 12 hours and in the cold shutdown condition within the following 24 hours. This allowable outage time and action statement requirements are consistent with similar provisions and requirements in the GE-STS and are considered reasonable and appropriate by the NRC staff. Therefore, we find the proposed allowable outage time and action statement requirements acceptable.

The existing surveillance requirements for TS 3.4.3/4.4.3 would be expanded to include periodic (once per 31 days) verifications that the access doors to the secondary containment are closed. The proposed surveillance requirements are consistent with the guidance provided by the GE-STS and are considered reasonable and appropriate by the NRC staff. Therefore, we find the proposed surveillance requirements acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes to the surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 22264). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor:
Donald S. Brinkman

Date: June 29, 1992



June 29, 1992

Docket No. 50-220

Mr. B. Ralph Sylvia
Executive Vice President, Nuclear
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

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Sincerely,

Original Signed By
Donald S. Brinkman, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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2. Safety Evaluation

cc w/enclosures:

See next page

OFFICE	LA:PDI-1	PM:PDI-1 <i>DB</i>	NRR/SBLB	OGC	D:PDI-1 <i>DB</i>
NAME	CSVogan <i>CV</i>	DBrinkman:pc	CMcCracken	<i>BMB</i>	RACpara <i>for</i>
DATE	6/11/92	6/11/92	6/17/92	6/22/92	6/29/92

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