



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

October 2, 1992

Docket No. 50-220

Mr. B. Ralph Sylvia  
Executive Vice President, Nuclear  
Niagara Mohawk Power Corporation  
301 Plainfield Road  
Syracuse, New York 13212

Dear Mr. Sylvia:

SUBJECT: ISSUANCE OF AMENDMENT FOR NINE MILE POINT NUCLEAR STATION UNIT NO. 1  
(TAC NO. M82560)

The Commission has issued the enclosed Amendment No. 132 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station Unit No. 1 (NMP-1). The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated January 7, 1992.

The amendment deletes the fire protection technical specifications and their associated Bases and Definitions from the Nine Mile Point Unit 1 Technical Specifications. The deleted requirements have been relocated to the Nine Mile Point Unit 1 Fire Hazards Analysis, which has been incorporated into Appendix 10A of the Nine Mile Point Unit 1 Final Safety Analysis Report (Updated). The amendment augments the Administrative Controls section of the Technical Specifications to require: (1) that written procedures be established, implemented, and maintained for activities involving implementation of the Fire Protection Program, (2) periodic review of the Fire Protection Program and implementing procedures by a qualified individual/organization, and (3) submittal of recommended changes to the Fire Protection Program and implementing procedures to the Safety Review and Audit Board. Conforming changes are also being made to the Index for the technical specifications. License Condition 2.D.(7) is being revised to: (1) require Niagara Mohawk Power Corporation (NMPC) to implement and maintain in effect all provisions of the approved Fire Protection Program as described in the Final Safety Analysis Report (Updated) as approved in the Fire Protection Safety Evaluation Report dated July 26, 1979, and in the fire protection Exemption issued March 21, 1983, and (2) to permit NMPC to make changes to the approved Fire Protection Program without prior approval of the NRC only if those changes do not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire. The changes are in accordance with the guidance provided in NRC Generic Letter 88-12, "Removal of Fire Protection Requirements from Technical Specifications," dated August 2, 1988, and NRC Generic Letter 86-10, "Implementation of Fire Protection Requirements," dated April 24, 1986.

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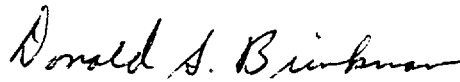
Mr. Ralph B. Sylvia

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October 2, 1992

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,



Donald S. Brinkman, Senior Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 132 to DPR-63
2. Safety Evaluation

cc w/enclosures:

See next page

Mr. B. Ralph Sylvia  
Niagara Mohawk Power Corporation

Nine Mile Point Nuclear Station  
Unit No. 1

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DATED: October 2, 1992

AMENDMENT NO. 132 TO FACILITY OPERATING LICENSE NO. DPR-63-NINE MILE POINT  
UNIT 1

Docket File

NRC & Local PDRs

PDI-1 Reading

S. Varga, 14/E/4

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C. Vogan

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G. Hill (4), P-137

Wanda Jones, P-130A

C. Grimes, 11/F/23

T. Dunning, 11/E/22

C. McCracken, 8/D/1

J. Stang, 13/D/18

ACRS(10)

OPA

OC/LFMB

PD Plant-specific file

C. Cowgill, RGN-I

cc: Plant Service list



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 132  
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated January 7, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 132, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. Accordingly, page 5 of Facility Operating License No. DPR-63 is hereby amended to read as follows:\*

2.D.(7) Fire Protection

Niagara Mohawk Power Corporation shall implement and maintain in effect all provisions of the approved Fire Protection Program as described in the Final Safety Analysis Report (Updated) for the facility and as approved in the Fire Protection Safety Evaluation Report dated July 26, 1979, and in the fire protection Exemption issued March 21, 1983, subject to the following provision:

Niagara Mohawk Power Corporation may make changes to the approved Fire Protection Program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

4. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachments:

1. Page 5 of license
2. Changes to the Technical Specifications

Date of Issuance:

\*Page 5 is attached, for convenience, for the composite license to reflect this change.

2.D.(7) Fire Protection

Niagara Mohawk Power Corporation shall implement and maintain in effect all provisions of the approved Fire Protection Program as described in the Final Safety Analysis Report (Updated) for the facility and as approved in the Fire Protection Safety Evaluation Report dated July 26, 1979, and in the fire protection Exemption issued March 21, 1983, subject to the following provision:

Niagara Mohawk Power Corporation may make changes to the approved Fire Protection Program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

2.D.(8) Hot Process Pipe Penetrations

Hot Process Pipe Penetrations in the Emergency Condenser Steam Supply (2 each), Main Steam (2 each), Feedwater (2 each), Cleanup Suction (1 each), and Cleanup Return (1 each) piping systems have been identified as not fully in conformance with FSAR design criteria. This anomaly in design condition from the original design is approved for the duration of Cycle 8 or until March 31, 1986, whichever occurs first, subject to the following conditions:

- (a) An unidentified leakage limit of a change of 1 gallon per minute in 24 hours to permit operation will be imposed by administrative control (Standing Order) at the facility for the interim period.
- (b) The licensee shall restore the facility to a condition consistent with the FSAR or provide a change to the FSAR criteria for staff review and approval prior to restart from the forthcoming Cycle 8 outage.

E. This license is effective as of the date of issuance and shall expire on August 22, 2009.

FOR THE ATOMIC ENERGY COMMISSION

Original Signed by

Attachment:  
Appendices A & B -  
Technical Specifications  
Date of Issuance: December 26, 1974

A. Giambusso, Deputy Director  
for Reactor Projects  
Directorate of Licensing





SECTION	DESCRIPTION	PAGE
3.6.5	Radioactive Material Sources	241k
3.6.6	(Deleted)	
3.6.7	(Deleted)	
3.6.8	(Deleted)	
3.6.9	(Deleted)	
3.6.10	(Deleted)	
3.6.11	Accident Monitoring Instrumentation	241ee
3.6.12	Reactor Protection System Motor Generator Set Monitoring	241iil
3.6.13	Remote Shutdown Panels	241ii4
3.6.14	Radioactive Effluent Instrumentation	241jj
3.6.15	Radioactive Effluents	241ww
3.6.16	Radioactive Effluent Treatment Systems	241qqq
3.6.17	Explosive Gas Mixture	241ttt
3.6.18	Mark I Containment	241vvv
3.6.19	Liquid Waste Holdup Tanks	241xxx
3.6.20	Radiological Environmental Monitoring Program	241zzz
3.6.21	Interlaboratory Comparison Program	241lll
3.6.22	Land Use Census	241nnnn

1.16 (Deleted)

1.17 (Deleted)

1.18 Gaseous Radwaste Treatment System

A gaseous radwaste treatment system is any system designed and installed to reduce radioactive gaseous effluents by collecting main condenser offgas and providing for delay or holdup for the purpose of reducing the total radioactivity prior to release to the environment.

1.19 Member(s) of the Public

Member(s) of the public shall include persons who are not occupationally associated with the Nine Mile Point Nuclear Station. This category does not include employees of Niagara Mohawk Power Corporation, the New York State Power Authority, its contractors or vendors who are occupationally associated with Nine Mile Point Unit 1. Also excluded from this category are persons who enter the site to service equipment or to make deliveries. This category does include persons who use portions of the site for recreational, occupational, or other purposes not associated with Nine Mile Point Unit 1.

1.20 Milk Sampling Location

A milk sampling location is that location where 10 or more head of milk animals are available for the collection of milk samples.

1.21 Offsite Dose Calculation Manual (ODCM)

The Offsite Dose Calculational Manual shall contain the current methodology and parameters used in the calculation of offsite doses due to radioactive gaseous and liquid effluents, in the calculation of gaseous and liquid effluent monitoring alarm/trip setpoints, and in the conduct of the environmental radiological monitoring program.

Pages 241m through 241dd4 deleted.

Onsite and Offsite Organization (Cont'd)

- d. The persons responsible for the training, health physics and quality assurance functions may report to an appropriate manager onsite, but shall have direct access to responsible corporate management at a level where action appropriate to the mitigation of training, health physics and quality assurance concerns can be accomplished.

Facility Staff

6.2.2 The unit organization shall be subject to the following:

- a. Each on-duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor. During reactor operation, this licensed operator shall be present at the controls of the facility.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection\* procedures shall be on site when fuel is in the reactor.

\* The requirement for a Radiation Protection qualified individual may be less than the minimum requirement for a period of time not to exceed 2 hours in order to accommodate unexpected absence, provided immediate action is taken to fill the required position.

Facility Staff (Cont'd)

- e. A licensed Senior Reactor Operator shall be required in the Control Room during power operations, hot shutdown, and when the emergency plan is activated. This may be the Station Shift Supervisor-Nuclear or the Assistant Station Shift Supervisor-Nuclear or another Senior Reactor Operator during power operations or hot shutdown. When the emergency plan is activated during normal operations or hot shutdown, the Assistant Station Shift Supervisor-Nuclear becomes the Shift Technical Advisor and the Station Shift Supervisor-Nuclear is restricted to the control room until an additional licensed Senior Reactor Operator arrives.
- f. A licensed Senior Reactor Operator shall be responsible for all movement of new and irradiated fuel within the site boundary. All core alterations shall be directly supervised by a licensed senior reactor operator who has no other concurrent responsibilities during this operation. A Licensed Operator will be required to manipulate the controls of all fuel handling equipment except movement of new fuel from receipt through dry storage. All fuel moves within the core shall be directly monitored by a member of the reactor analyst group.
- g. DELETED
- h. Administrative procedures shall be developed and implemented to limit the working hours of facility staff who perform safety-related functions; e.g., licensed Senior Operators, licensed Operators, health physicists, auxiliary operators and key maintenance personnel.

Adequate shift coverage shall be maintained without routine heavy use of overtime. The objective shall be to have operating personnel work a normal 8-hour day, 40-hour week while the facility is operating. However, in the event that unforeseen problems require substantial amounts of overtime to be used, or during extended periods of shutdown for refueling, major maintenance or major plant modifications on a temporary basis, the following guidelines shall be followed:

Activities (Cont'd)

- 6.5.2.9 The Plant Manager shall assure the performance of a review by a qualified individual/organization of changes to the Radiological Waste Treatment systems.
- 6.5.2.10 Review of any accidental, unplanned, or uncontrolled radioactive release including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President - Nuclear Generation and to the Safety Review and Audit Board.
- 6.5.2.11 Review of changes to the Process Control Program and the Offsite Dose Calculation Manual. Approval of any changes shall be made by the Plant Manager or his designee before implementation of such changes.
- 6.5.2.12 Reports documenting each of the activities performed under Specifications 6.5.2.1 through 6.5.2.9 shall be maintained. Copies shall be provided to the Vice President-Nuclear Generation and the Safety Review and Audit Board.
- 6.5.2.13 The Plant Manager shall assure the performance of a review by a qualified individual/organization of the Fire Protection Program and implementing procedures at least every 12 months and submittal of recommended changes to the Safety Review and Audit Board.

6.5.3 Safety Review and Audit Board (SRAB)

Function

- 6.5.3.1 The Safety Review and Audit Board shall function to provide independent review and audit of designated activities in the areas of:
  - a. nuclear power plant operations
  - b. nuclear engineering
  - c. chemistry and radiochemistry
  - d. metallurgy
  - e. instrumentation and control
  - f. radiological safety
  - g. mechanical and electrical engineering
  - h. quality assurance practices
  - i. (other appropriate fields associated with the unique characteristics of the nuclear power plant)

6.6 Reportable Occurrence Action

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Sections 50.72 and 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the SORC and the results of this review submitted to the SRAB and the Vice President - Nuclear Generation.

6.7 Safety Limit Violation

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR 50.36(c)(1)(i) shall be complied with immediately.
- b. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Vice President - Nuclear Generation and the SRAB shall be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, within 30 days of the violation, and to the SRAB, and the Vice President - Nuclear Generation within 14 days.

6.8 Procedures

6.8.1 Written procedures and administrative policies shall be established, implemented and maintained that meet or exceed the requirements and recommendations of Sections 5.1 and 5.3 of ANSI N18.7-1972 and Appendix "A" of USAEC Regulatory Guide 1.33 except as provided in ( 6.8.2 and 6.8.3 below.

- a. Written procedures shall be established, implemented, and maintained for activities involving the Fire Protection Program implementation.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed and approved by the Plant Manager or designee prior to implementation and periodically as set forth in administrative procedures.

- 3) NEDO-20556-P-A "GENERAL ELECTRIC COMPANY ANALYTICAL MODEL FOR LOSS-OF-COOLANT ACCIDENT ANALYSIS IN ACCORDANCE WITH 10CFR50 APPENDIX K". (Latest approved revision)
- .3 The core operating limits shall be determined such that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, ECCS limits, nuclear limits such as shutdown margin, transient analysis limits, and accident analysis limits) of the safety analysis are met.
- .4 The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements shall be provided, upon issuance for each reload cycle, to the NRC Document Control Desk with copies to the Regional Administrator and Resident Inspector.

6.9.2 Fire Protection Program Reports

Noncompliances with the Fire Protection Program (as described in the Final Safety Analysis Report) that adversely affect the ability to achieve and maintain safe shutdown in the event of a fire shall be reported in accordance with the requirements of 10CFR50.72 and 10CFR50.73.





UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 132 TO FACILITY OPERATING LICENSE NO. DPR-63  
NIAGARA MOHAWK POWER CORPORATION  
NINE MILE POINT NUCLEAR STATION UNIT NO. 1  
DOCKET NO. 50-220

1.0 INTRODUCTION

By letter dated January 7, 1992, Niagara Mohawk Power Corporation (the licensee) submitted a request for changes to the Nine Mile Point Nuclear Station Unit No. 1, Technical Specifications (TS). The requested changes would delete the fire protection technical specifications and their associated Bases and Definitions from the Nine Mile Point Unit 1 Technical Specifications. The deleted requirements have been relocated to the Nine Mile Point Unit 1 Fire Hazards Analysis, which is incorporated into Appendix 10A of the Nine Mile Point Unit 1 Final Safety Analysis Report (Updated). The proposed amendment would augment the Administrative Controls section of the Technical Specifications to require: (1) that written procedures be established, implemented, and maintained for activities involving implementation of the Fire Protection Program, and (2) periodic review of the Fire Protection Program and implementing procedures by a qualified individual/organization, and (3) submittal of recommended changes to the Fire Protection Program and implementing procedures to the Safety Review and Audit Board. Conforming changes would also be made to the Index for the technical specifications. License Condition 2.D.(7) would be revised to: (1) require the licensee to implement and maintain in effect all provisions of the approved Fire Protection Program as described in the Final Safety Analysis Report (Updated) as approved in the Fire Protection Safety Evaluation Report dated July 26, 1979, and in the fire protection Exemption issued March 21, 1983, and (2) to permit the licensee to make changes to the approved Fire Protection Program without prior approval of the NRC only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire. The proposed changes are in accordance with the guidance provided in NRC Generic Letter 88-12, "Removal of Fire Protection Requirements from Technical Specifications," dated August 2, 1988, and NRC Generic Letter 86-10, "Implementation of Fire Protection Requirements," dated April 24, 1986.

## 2.0 BACKGROUND

Following the fire at the Browns Ferry Nuclear Power Plant on March 22, 1975, the Commission undertook a number of actions to ensure that improvements were implemented in the Fire Protection Programs for all power reactor facilities. Because of the extensive modification of Fire Protection Programs and the number of open issues resulting from staff evaluations, a number of revisions and alterations occurred in these programs over the years. Consequently, licensees were requested by Generic Letter 86-10 to incorporate the final NRC-approved Fire Protection Program in their Final Safety Analysis Reports (FSARs). In this manner, the Fire Protection Program including the systems, the administrative and technical controls, the organization, and other plant features associated with fire protection would have a status consistent with that of other plant features described in the FSAR. In addition, the Commission concluded that a standard license condition, requiring compliance with the provisions of the Fire Protection Program as described in the FSAR, should be used to ensure uniform enforcement of fire protection requirements. Finally, the Commission stated that with the requested actions, licensees may request an amendment to delete the fire protection TS that would now be unnecessary.

The licensees for the Callaway and Wolf Creek plants submitted lead-plant proposals to remove fire protection requirements from their TS. This action was an industry effort to obtain NRC guidance on an acceptable format for license amendment requests to remove fire protection requirements from TS. Additionally, in the licensing review of new plants, the staff has approved applicant requests to remove fire protection requirements from TS issued with the operating license. Thus, on the basis of the lead-plant proposals and the staff's experience with TS for new licenses, Generic Letter 88-12 was issued to provide guidance on removing fire protection requirements from TS.

## 3.0 EVALUATION

Generic Letter 86-10 recommended the removal of fire protection requirements from the TS. Although a comprehensive Fire Protection Program is essential to plant safety, the basis for this recommendation is that many details of this program that are currently addressed in TS can be modified without affecting nuclear safety. Such modifications can be made provided that there are suitable administrative controls over these changes. These details, that are presently included in TS and which are removed by this amendment, do not constitute performance requirements necessary to ensure safe operation of the facility and, therefore, do not warrant being included in TS. At the same time, suitable administrative controls ensure that there will be careful review and analysis by competent individuals of any changes in the Fire Protection Program including those technical and administrative requirements removed from the TS to ensure that nuclear safety is not adversely affected. These controls include: (1) the TS administrative controls that are applicable

to the Fire Protection Program; (2) the license condition on implementation of, and subsequent changes to, the Fire Protection Program; and (3) the 10 CFR 50.59 criteria for evaluating changes to the Fire Protection Program as described in the FSAR.

The specific details relating to fire protection requirements removed from TS by this amendment include those specifications for fire detection systems, fire suppression systems, fire barriers, and fire brigade staffing requirements. The administrative control requirements have been modified to include Fire Protection Program implementation as an element for which written procedures must be established, implemented, and maintained. In addition, the technical review responsibilities of Station Operations Review Committee will be expanded to include the review of the Fire Protection Program and implementing procedures and submittal of recommended changes to the Safety Review and Audit Board.

The TS changes proposed by the licensee are in accordance with the guidance provided by Generic Letter 88-12, as addressed in the items below.

- (1) Specification 6.5.2, Technical Review and Control, was revised to add the review of the fire protection program implementation and the submittal of recommended changes to the Safety Review and Audit Board.
- (2) Specification 6.8, Procedures, was revised to add Fire Protection Program implementation to those programs for which written procedures shall be established, implemented, and maintained.
- (3) Specification 3.6.6, Fire Detection instrumentation, its associated Surveillance Requirements, and Bases were removed.
- (4) Specifications 3.6.7, 3.6.8, 3.6.9, 3.6.10.2, and 3.6.10.3, Fire Suppression systems, their associated Surveillance Requirements, and Bases were removed.
- (5) Specification 3.6.10.1, Fire Barrier Penetrations, its associated Surveillance Requirements, and Bases were removed.
- (6) Specification 6.2.2.g on fire brigade staffing requirements was deleted.

The licensee proposed to change License Condition 2.D.(7) to read as follows:

Niagara Mohawk Power Corporation shall implement and maintain in effect all provisions of the approved Fire Protection Program as referenced in the Final Safety Analysis Report (Updated) for the facility and as described in the Nine Mile Point Nuclear Station Unit 1 Fire Hazards Analysis as referenced in the Final Safety Analysis Report (Updated), subject to the following provision:

Niagara Mohawk Power Corporation may make changes to the approved Fire Protection Program without prior approval of the Commission only if those

changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

However, since the standard fire protection license condition provided in Generic Letter 86-10 requires this proposed license condition to include the approval date of the NRC staff's Fire Protection Safety Evaluation Report, License Condition 2.D.(7) is being revised to read as follows:

Niagara Mohawk Power Corporation shall implement and maintain in effect all provisions of the approved Fire Protection Program as described in the Final Safety Analysis Report (Updated) for the facility and as approved in the Fire Protection Safety Evaluation Report dated July 26, 1979, and in the fire protection Exemption issued March 21, 1983, subject to the following provision:

Niagara Mohawk Power Corporation may make changes to the approved Fire Protection Program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

The NRC staff discussed this proposed change for License Condition 2.D.(7) with licensee representatives who agreed with the proposed changes. The proposed change does not affect the proposed no significant hazards consideration and is consistent with the guidance provided in Generic Letter 86-10 and is, therefore, acceptable.

As required by Generic Letter 86-10, the licensee confirmed that the NRC-approved Fire Protection Program has been incorporated into the FSAR.

The licensee confirmed that the operational conditions, remedial actions, and test requirements associated with the removed fire protection TS have been included in the Fire Protection Program incorporated into the FSAR. This is in accordance with the guidance of Generic Letter 88-12.

License Amendment No. 71 (issued April 1, 1985) was prepared and issued in response to the licensee's response to Generic Letter 81-12, "Fire Protection Rule (45 FR 76602, November 19, 1980)." License Amendment No. 71 added technical specifications (Limiting Conditions for Operation, Surveillance Requirements, and Bases) requiring the operability of the Remote Shutdown Panels. The Remote Shutdown Panels provide the capability for achieving and maintaining hot shutdown conditions in the event of a fire in the main/auxiliary control room. The technical specifications added by License Amendment No. 71 remain in effect and are unchanged by this amendment.

On the basis of its review of the above items, the staff concludes that the licensee has met the guidance of Generic Letter 88-12. Therefore, the staff finds the proposed changes acceptable.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

#### 5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (56 FR 4489). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

#### 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors:

T. Dunning  
D. Brinkman

Date: October 2, 1992

Mr. Ralph B. Sylvia

- 2 -

October 2, 1992

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original Signed By:

Donald S. Brinkman, Senior Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 132 to DPR-63
- 2. Safety Evaluation

cc w/enclosures:  
See next page

OFFICE	LA:PDI-1 <i>CV</i>	PM:PDI-1 <i>RY</i>	SPLB <i>CR</i>	OGC	D:PDI-1 <i>CV</i>
NAME	CVogan <i>CV</i>	<i>FOR</i> DBrinkman:av1	CMcCracken	<i>S. Horn</i>	RACapra <i>CV</i>
DATE	2/24/92	<i>9/14/92</i> 2/25/92 <i>9/14/92</i>	9/25/92	9/30/92	10/02/92

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