



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

December 3, 1992

Docket No. 50-220

Mr. B. Ralph Sylvia
Executive Vice President, Nuclear
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Dear Mr. Sylvia:

SUBJECT: ISSUANCE OF AMENDMENT FOR NINE MILE POINT NUCLEAR STATION UNIT NO. 1
(TAC NO. M79135)

The Commission has issued the enclosed Amendment No. 134 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station Unit No. 1 (NMP-1). The amendment consists of changes to the Technical Specifications in partial response to your application transmitted by letter dated November 20, 1990, as superseded February 7, 1992. Those portions of your February 7, 1992, application not contained in this amendment will be the subject of future correspondence.

The amendment revises Table 3.3.4 of Technical Specification 3.3.4 to require closure of the drywell and suppression chamber vent and purge isolation valves on a high radiation signal from the stack radiation monitor. This change is consistent with the requirements of Item II.E.4.2 of NUREG-0737.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Donald S. Brinkman, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 134 to DPR-63
2. Safety Evaluation

cc w/enclosures:
See next page

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Mr. B. Ralph Sylvia
Niagara Mohawk Power Corporation

Nine Mile Point Nuclear Station
Unit No. 1

cc:

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Albany, New York 12223

DATED: December 3, 1992

AMENDMENT NO. 134 TO FACILITY OPERATING LICENSE NO. DPR-63-NINE MILE POINT
UNIT 1

Docket File

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cc: Plant Service list

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 134
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated November 20, 1990, as superseded February 7, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 134, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Capra

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 3, 1992

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 134 TO FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise Appendix A as follows:

Remove Page
146

Insert Page
146

LIMITING CONDITION FOR OPERATION

Table 3.3.4

PRIMARY CONTAINMENT ISOLATION VALVES
LINES ENTERING FREE SPACE OF THE CONTAINMENT

<u>Line or System</u>	<u>No. of Valves (Each Line)</u>	<u>Location Relative to Primary Containment</u>	<u>Normal Position</u>	<u>Motive Power</u>	<u>Maximum Oper. Time (Sec)</u>	<u>Action on Initiating Signal</u>	<u>Initiating Signal (All Valves Have Remote Manual Backup)</u>
<u>Drywell Vent & Purge</u>							
<u>N₂ Connection (One Line)</u>	1	Outside	Closed (a)	Air/D.C. Sol.	60	Close	Reactor water level low-low or high drywell pressure or high radiation at stack monitoring.
	1	Outside	Closed (a)	A.C. Motor	60	Close	
<u>Air Connection (One Line)</u>	1	Outside	Closed (a)	Air/D.C. Sol.	60	Close	
	1	Outside	Closed (a)	A.C. Motor	60	Close	
<u>Suppression Chamber Vent & Purge</u>							
<u>N₂ Connection (One Line)</u>	1	Outside	Closed (a)	Air/D.C. Sol.	60	Close	Reactor water level low-low or high drywell pressure or high radiation at stack monitoring.
	1	Outside	Closed (a)	A.C. Motor	60	Close	
<u>Air Connection (One Line)</u>	1	Outside	Closed (a)	Air/D.C. Sol.	60	Close	
	1	Outside	Closed (a)	A.C. Motor	60	Close	
<u>Drywell N₂ Makeup (One Line)</u>	2	Outside	Closed (b)	Air/D.C. Sol.	60	Close	Reactor water level low-low or drywell high pressure
<u>Suppression Chamber N₂ Makeup (One Line)</u>	2	Outside	Closed (b)	Air/D.C. Sol.	60	Close	Reactor water level low-low or drywell high pressure
<u>Drywell Equipment Drain Line (One Line)</u>	1	Inside	Open	A.C. Motor	60	Close	Reactor water level low-low or drywell high pressure
	1	Outside	Open	Air/D.C. Sol.	60	Close	
<u>Floor Drain Line (One Line)</u>	1	Inside	Open	A.C. Motor	60	Close	
	1	Outside	Open	Air/D.C. Sol.	60	Close	
<u>Suppression Chamber Water Makeup (One Line)</u>	1	Outside	Closed (b)	A.C. Motor	60	-	Remote manual
	1	Outside	-	Self Act. Ck.	--	-	-
<u>Vacuum Relief Atmosphere to Pressure Suppression System (Three Lines)</u>	1	Outside	Closed	A.C. Motor	5	Open	Negative pressure rel- ative to atmosphere
	1	Outside	-	Self Act. Ck.	--	-	-
<u>Reactor Cleanup System Relief Valve Discharge (One Line to Suppression Chamber)</u>	2	Outside	-	Self Act. Ck.	--	-	-



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20565

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 134 TO FACILITY OPERATING LICENSE NO. DPR-63
NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT NUCLEAR STATION UNIT NO. 1
DOCKET NO. 50-220

1.0 INTRODUCTION

By letter dated November 20, 1992, as superseded February 7, 1992, Niagara Mohawk Power Corporation (the licensee) submitted a request for changes to the Nine Mile Point Nuclear Station Unit No. 1, Technical Specifications (TS). The requested changes would revise TS Tables 3.2.7 and 3.2.7.1; TS 3.3.3, 4.3.3, and 3.3.4; and associated Bases to update these TS to conform them to the requirements of 10 CFR Part 50, Appendix J, and the NRC safety evaluation dated May 6, 1988. The requested changes would also add a requirement to TS Table 3.3.4 to require closure of the drywell and suppression chamber vent and purge isolation valves on a high radiation signal from the stack radiation monitor. Closure of these vent and purge isolation valves on a high radiation signal from the stack radiation monitor is a requirement of Item II.E.4.2 of NUREG-0737 and this is the only portion of the February 7, 1992, request being approved by this safety evaluation and license amendment. The licensee's February 7, 1992, letter superseded a letter from the licensee dated November 28, 1990.

2.0 EVALUATION

The proposed change would add high radiation at the stack radiation monitor as an initiation signal which would cause automatic closure of the drywell and suppression chamber vent and purge isolation valves. The proposed change would add diversity to the parameters sensed for the initiation of containment isolation and would therefore be in accordance with the requirements of Section 6.2.4 of the Standard Review Plan. The additional of high radiation as initiating signal for containment isolation is also in accordance with the requirements of Item II.E.4.2 of NUREG-0737. Therefore, the NRC staff finds this proposed change acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 4868 and 57 FR 11111). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: D. Shum
D. Brinkman

Date: December 3, 1992

December 3, 1992

Docket No. 50-220

Mr. B. Ralph Sylvia
Executive Vice President, Nuclear
Niagara Mohawk Power Corporation
301 Plainfield Road
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Sincerely,

Original signed by:

Donald S. Brinkman, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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2. Safety Evaluation

cc w/enclosures:

See next page

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NAME	CVogan <i>CV</i>	DBrinkman:avl	CMcCracken	<i>E HOLLER</i>	RACapra <i>RU</i>
DATE	<i>11/10/92</i>	<i>11/10/92</i>	<i>11/10/92</i>	<i>11/16/92</i>	<i>12/03/92</i>

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