

July 7, 1998

Mr. John H. Mueller
Chief Nuclear Officer
Niagara Mohawk Power Corporation
Nine Mile Point Nuclear Station
Operations Building, Second Floor
Lycoming, NY 13093

SUBJECT: ISSUANCE OF AMENDMENTS FOR NINE MILE POINT NUCLEAR STATION,
UNIT NOS. 1 AND 2 (TAC NOS. MA1942 AND MA1943)

Dear Mr. Mueller:

The Commission has issued the enclosed Amendment No. 162 to Facility Operating License No. DPR-63 and Amendment No. 83 to Facility Operating License No. NPF-69 for the Nine Mile Point Nuclear Station, Unit Nos. 1 and 2. The amendments consist of changes to the Technical Specifications (TSs) in response to your applications transmitted by letters dated May 15, 1998 (two letters, one for each unit).

The amendments change administrative sections of the TSs to reflect a restructuring of your Nuclear Division upper management organization.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

original signed by:

Darl S. Hood, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-220
and 50-410

- Enclosures: 1. Amendment No. 162 to DPR-63
- 2. Amendment No. 83 to NPF-69
- 3. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION: See next page

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NIAGARA MOHAWK POWER CORPORATION
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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

July 7, 1998

Mr. John H. Mueller
Chief Nuclear Officer
Niagara Mohawk Power Corporation
Nine Mile Point Nuclear Station
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The amendments change administrative sections of the TSs to reflect a restructuring of your Nuclear Division upper management organization.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Darl S. Hood".

Darl S. Hood, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-220
and 50-410

Enclosures: 1. Amendment No. 162 to DPR-63
2. Amendment No. 83 to NPF-69
3. Safety Evaluation

cc w/encls: See next page

John H. Mueller
Niagara Mohawk Power Corporation

Nine Mile Point Nuclear Station
Unit Nos. 1 and 2

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DATED: July 7, 1998

AMENDMENT NO.162 TO FACILITY OPERATING LICENSE NO. DPR-63-NINE MILE POINT
UNIT 1

AMENDMENT NO. 83 TO FACILITY OPERATING LICENSE NO. NPF-69-NINE MILE POINT
UNIT 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 162
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated May 15, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 162 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



S. Singh Bajwa, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 7, 1998

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 162 TO FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise Appendix A as follows:

Remove Pages

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Insert Pages

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6.0 ADMINISTRATIVE CONTROLS

6.1 Responsibility

- 6.1.1 The Plant Manager shall be responsible for overall unit operation and shall delegate in writing the succession to this responsibility during his absence.
- 6.1.2 The Station Shift Supervisor - Nuclear (or during his absence from the control room, a designated individual) shall be responsible for the control room command function. A management directive to this effect, signed by the Chief Nuclear Officer shall be re-issued to station personnel on an annual basis.

6.2 Organization

Onsite and Offsite Organization

- 6.2.1 An onsite and an offsite organization shall be established for unit operation and corporate management. The onsite and offsite organization shall include the position for activities affecting the safety of the nuclear power plant.
- a. Lines of authority, responsibility and communication shall be established and defined from the highest management levels through intermediate levels to and including all operating organization positions. Those relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions or in equivalent forms of documentation. The organization charts shall be documented in the Final Safety Analysis Report, and the functional descriptions of departmental responsibilities and relationships and job descriptions for key personnel positions are documented in procedures.
 - b. The Chief Nuclear Officer shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to assure acceptable performance of the staff in operating, maintaining, and providing technical support in the plant so that continued nuclear safety is assured.
 - c. The Plant Manager shall have responsibility for overall unit operation and shall have control over those resources necessary for safe operation and maintenance of the plant.

- g. DELETED
- h. Administrative procedures shall be developed and implemented to limit the working hours of facility staff who perform safety-related functions; e.g., licensed Senior Operators, licensed Operators, health physicists, auxiliary operators and key maintenance personnel.

Adequate shift coverage shall be maintained without routine heavy use of overtime. The objective shall be to have operating personnel work an 8 to 12 hour day, nominal 40-hour week while the facility is operating. However, in the event that unforeseen problems require substantial amounts of overtime to be used, or during extended periods of shutdown for refueling, major maintenance or major plant modifications on a temporary basis, the following guidelines shall be followed:

- 1) An individual should not be permitted to work more than 16 hours straight (excluding shift turnover time).
- 2) An individual should not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any 7 day period (all excluding shift turnover time).
- 3) A break of at least 8-hours should be allowed between work periods (including shift turnover time).
- 4) Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the Plant Manager, or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Controls shall be included in the procedures such that individual overtime shall be reviewed monthly by the Vice President - Nuclear Generation or designee to assure that excessive hours have not been assigned. Routine deviation from the above guidelines is not authorized.

- i. As a minimum, either the Manager Operations or the General Supervisor Operations shall hold a senior reactor operator license. The Station Shift Supervisor Nuclear and Assistant Station Shift Supervisor Nuclear shall hold senior reactor operator licenses. Only licensed individuals may direct licensed activities.

Alternates

- 6.5.1.3 All alternate members shall be appointed in writing by the SORC Chairman or Vice-Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in SORC activities at any one time.

Meeting Frequency

- 6.5.1.4 The SORC shall meet at least once per calendar month and as convened by the SORC Chairman, Vice-Chairman, (a designated alternate.

Quorum

- 6.5.1.5 The quorum of the SORC necessary for the performance of the SORC responsibility and authority provisions of these Technical Specifications shall consist of the Chairman, or a Vice-Chairman, and four members, including alternates.

Responsibilities

- 6.5.1.6 The SORC shall be responsible for:
- a. Review of all REPORTABLE EVENTS.
 - b. Review of unit operations to detect potential safety hazards.
 - c. Performance of special reviews, investigations or analyses and reports thereon as requested by the Plant Manager or the Safety Review and Audit Board.
 - d. Investigation of violations of the Technical Specifications and shall prepare and forward a report covering evaluation and recommendations to prevent recurrence to the Vice President - Nuclear Generation and to the Safety Review and Audit Board.

Authority

- 6.5.1.7 The SORC shall:
- a. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6 (a) through (d) above constitutes an unreviewed safety question.
 - b. Provide written notification within 24 hours to the Vice President - Nuclear Generation and the Safety Review and Audit Board of disagreement between the SORC and the Plant Manager; however, the Plant Manager shall have the responsibility for resolution of such disagreements pursuant to 6.1.1 above.

Records

- 6.5.1.8 The SORC shall maintain written minutes of each meeting and copies shall be provided to the Vice President - Nuclear Generation and the Safety Review and Audit Board.

6.5.2 Technical Review and Control

Activities

- 6.5.2.1 Each procedure and program required by Specification 6.8 and other procedures which affect nuclear safety, and changes thereto, shall be prepared by a qualified individual/organization. Each such procedure, and changes thereto, shall be reviewed by an individual/group other than the individual/group which prepared the procedure, or changes thereto, but who may be from the same organization as the individual/group which prepared the procedure, or changes thereto. Approval of procedures and programs and changes thereto and their safety evaluations, shall be controlled by administrative procedures.
- 6.5.2.2 Proposed changes to the Technical Specifications shall be prepared by a qualified individual/organization. The preparation of each proposed Technical Specifications change shall be reviewed by an individual/group other than the individual/group which prepared the proposed change, but who may be from the same organization as the individual/group which prepared the proposed change. Proposed changes to the Technical Specifications shall be approved by the Plant Manager.

- 6.5.2.3 Proposed modifications to unit structures, systems and components that affect nuclear safety shall be designed by a qualified individual/organization. Each such modification shall be reviewed by an individual/group other than the individual/group which designed the modification, but who may be from the same organization as the individual/group which designed the modification. Proposed modifications to structures, systems and components and the safety evaluations shall be approved prior to implementation by the Plant Manager; or the Manager Technical Support as previously designated by the Plant Manager.
- 6.5.2.4 Individuals responsible for reviews performed in accordance with Specifications 6.5.2.1, 6.5.2.2 and 6.5.2.3 shall be members of the station supervisory staff, previously designated by the Plant Manager to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary such review shall be performed by the appropriate designated station review personnel.
- 6.5.2.5 Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications and their safety evaluations shall be reviewed by the Plant Manager, or the Manager Technical Support as previously designated by the Plant Manager.
- 6.5.2.6 The Plant Manager shall assure the performance of special reviews and investigations, and the preparation and submittal of reports thereon, as requested by the Vice President - Nuclear Generation.
- 6.5.2.7 The facility security program, and implementing procedures, shall be reviewed at least every 12 months. Recommended changes shall be approved by the Plant Manager and transmitted to the Vice President - Nuclear Generation and to the Chairman of the Safety Review and Audit Board.
- 6.5.2.8 The facility emergency plan, and implementing procedures shall be reviewed at least every 12 months. Recommended changes shall be approved by the Plant Manager and transmitted to the Vice President - Nuclear Generation and to the Chairman of the Safety Review and Audit Board.

- 6.5.2.9 The Plant Manager shall assure the performance of a review by a qualified individual/organization of changes to the Radiological Waste Treatment systems.
- 6.5.2.10 Review of any accidental, unplanned, or uncontrolled radioactive release including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President - Nuclear Generation and to the Safety Review and Audit Board.
- 6.5.2.11 Review of changes to the Process Control Program and the Offsite Dose Calculation Manual. Approval of any changes shall be made by the Plant Manager or his designee before implementation of such changes.
- 6.5.2.12 Reports documenting each of the activities performed under Specifications 6.5.2.1 through 6.5.2.9 shall be maintained. Copies shall be provided to the Vice President - Nuclear Generation and the Safety Review and Audit Board.
- 6.5.2.13 The Plant Manager shall assure the performance of a review by a qualified individual/organization of the Fire Protection Program and implementing procedures at least every 12 months and submittal of recommended changes to the Safety Review and Audit Board.

6.5.3 Safety Review and Audit Board (SRAB)

Function

- 6.5.3.1 The Safety Review and Audit Board shall function to provide independent review and audit of designated activities in the areas of:
 - a. nuclear power plant operations
 - b. nuclear engineering
 - c. chemistry and radiochemistry
 - d. metallurgy
 - e. instrumentation and control
 - f. radiological safety
 - g. mechanical and electrical engineering
 - h. quality assurance practices
 - i. (other appropriate fields associated with the unique characteristics of the nuclear power plant)

Audits

- 6.5.3.8 Audits of facility activities shall be performed under the cognizance of the SRAB. These audits shall encompass:
- a. The conformance of facility operation to all provisions contained within the Technical Specifications and applicable license conditions at least once per year.
 - b. The performance, training and qualifications of the entire facility staff at least once per year.
 - c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per six months.
 - d. The performance of all activities required by the Quality Assurance Program to meet the criteria of Appendix "B", 10CFR50, at least once per two years.
 - e. The Facility Emergency Plan and implementing procedures at least once every 12 months.
 - f. The Facility Security Plan and implementing procedures at least once every 12 months.
 - g. The Facility Fire Protection Program and implementing procedures at least once per two years.
 - h. Any other area of facility operation considered appropriate by the SRAB or the Vice President - Nuclear Generation.
 - i. The radiological environmental monitoring program and the results thereof at least once per 12 months.
 - j. The Offsite Dose Calculation Manual and implementing procedures at least once per 24 months.
 - k. The Process Control Program and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months.

6.6 Reportable Occurrence Action

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Sections 50.72 and 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the SORC and the results of this review submitted to the SRAB and the Vice President - Nuclear Generation.

6.7 Safety Limit Violation

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR 50.36(c)(1)(i) shall be complied with immediately.
- b. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Vice President - Nuclear Generation and the SRAB shall be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, within 30 days of the violation, and to the SRAB, and the Vice President - Nuclear Generation within 14 days.

6.8 Procedures

6.8.1 Written procedures and administrative policies shall be established, implemented and maintained that meet or exceed the requirements and recommendations of Sections 5.1 and 5.3 of ANSI N18.7-1972 and Appendix "A" of USAEC Regulatory Guide 1.33 except as provided in 6.8.2 and 6.8.3 below.

- a. Written procedures shall be established, implemented, and maintained for activities involving the Fire Protection Program implementation.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed and approved prior to implementation by the branch manager for the functional area of the procedure or higher levels of management as governed by administrative procedures. Each procedure and administrative policy of 6.8.1 above shall be reviewed periodically as set forth in administrative procedures.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-410

NINE MILE POINT NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 83
License No. NPF-69

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated May 15, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-69 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 83 are hereby incorporated into this license. Niagara Mohawk Power Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



S. Singh Bajwa, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 7, 1998

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO.83 TO FACILITY OPERATING LICENSE NO. NPF-69

DOCKET NO. 50-410

Revise Appendix A as follows:

Remove Pages

6-1
6-3
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6-11
6-14
6-15

Insert Pages

6-1
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6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Plant Manager shall be responsible for overall unit operation and shall delegate in writing the succession to this responsibility during the Plant Manager's absence.

6.1.2 The Station Shift Supervisor - Nuclear (or during the Supervisor's absence from the control room, a designated individual) shall be responsible for the control room command function. A management directive to this effect, signed by the Chief Nuclear Officer shall be reissued to all station personnel annually.

6.2 ORGANIZATION

6.2.1 Onsite and Offsite Organization

An onsite and an offsite organization shall be established for unit operation and corporate management. The onsite and offsite organization shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility and communication shall be established and defined from the highest management levels through intermediate levels to and including all operating organization positions. Those relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions or in equivalent forms of documentation. The organization charts shall be documented in the Final Safety Analysis Report, and the functional descriptions of departmental responsibilities and relationships and job descriptions for key personnel positions are documented in procedures.
- b. The Chief Nuclear Officer shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to assure acceptable performance of the staff in operating, maintaining, and providing technical support in the plant so that continued nuclear safety is assured.
- c. The Plant Manager shall have responsibility for overall unit operation and shall have control over those resources necessary for safe operation and maintenance of the plant.
- d. The persons responsible for the training, health physics and quality assurance functions may report to an appropriate manager onsite, but shall have direct access to responsible corporate management at a level where action appropriate to the mitigation of training, health physics and quality assurance concerns can be accomplished.

UNIT STAFF

6.2.2 The unit organization shall be subject to the following:

- a. Each on-duty shift shall be composed of at least the minimum shift crew shown in Table 6.2.2-1;

6.0 ADMINISTRATIVE CONTROLS

ORGANIZATION

UNIT STAFF

6.2.2.i (Continued)

4. Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the Plant Manager, or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Controls shall be included in the procedures so that individual overtime shall be reviewed monthly by the Vice President - Nuclear Generation or a designee to assure that excessive hours have not been assigned. Routine deviation from the above guidelines is not authorized.

- j. The General Supervisor Operations, Supervisor Operations, Station Shift Supervisor Nuclear and Assistant Station Shift Supervisor Nuclear shall hold senior reactor operator licenses.

6.2.3 INDEPENDENT SAFETY ENGINEERING GROUP

FUNCTION

6.2.3.1 The Independent Safety Engineering Group (ISEG) shall function to examine unit operating characteristics, NRC issuances, industry advisories, Licensee Event Reports, and other sources of unit design and operating experience information, including units of similar design, which may indicate areas for improving unit safety. The ISEG shall make detailed recommendations for revised procedures, equipment modifications, maintenance activities, operations activities, or other means of improving unit safety to the Vice President - Nuclear Safety Assessment and Support.

COMPOSITION

6.2.3.2 The ISEG shall be composed of at least five, dedicated, full-time engineers located on site. Each shall have a bachelor's degree in engineering or related science and at least 2 years of professional level experience in his/her field, at least 1 year of which experience shall be in the nuclear field.

RESPONSIBILITIES

6.2.3.3 The principal function of the ISEG is to examine plant operating characteristics and the various NRC and industry licensing and service advisories, and to recommend areas for improving plant operations or safety. The ISEG will perform independent review of plant activities, including maintenance, modifications, operational concerns, and analysis and make recommendations to the Vice President - Nuclear Safety Assessment and Support.

ADMINISTRATIVE CONTROLS

6.5 REVIEW AND AUDIT

6.5.1 STATION OPERATIONS REVIEW COMMITTEE

FUNCTION

6.5.1.1 The Station Operations Review Committee (SORC) shall function to advise the Plant Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The SORC shall be composed of the

Chairman:	Plant Manager
Vice-Chairman/Member:	Manager Operations
Vice-Chairman/Member:	Manager Technical Support
Member:	Manager QA Operations
Member:	Manager Maintenance
Member:	Manager Chemistry
Member:	Manager Radiation Protection

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SORC Chairman or Vice-Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in SORC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The SORC shall meet at least once every calendar month and as convened by the SORC Chairman, Vice-Chairman, or a designated alternate.

QUORUM

6.5.1.5 The quorum of the SORC necessary for the performance of the SORC responsibility and authority provisions of these Technical Specifications shall consist of the Chairman, or a Vice-Chairman and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The SORC shall be responsible for:

- a. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence, to the Vice President - Nuclear Generation and to the Safety Review and Audit Board;

ADMINISTRATIVE CONTROLS

6.5 REVIEW AND AUDIT

6.5.1 STATION OPERATIONS REVIEW COMMITTEE

6.5.1.6 (Continued)

- b. Review of all REPORTABLE EVENTS;
- c. Review of unit operations to detect potential hazards to nuclear safety;
- d. Performance of special reviews, investigations, or analyses and reports thereon as requested by the Plant Manager or the Safety Review and Audit Board;
- e. Safety evaluations and analyses resulting from technical review and control activities 6.5.2.1, 6.5.2.2, 6.5.2.3, and 6.5.2.5.

DUTIES

6.5.1.7 The SORC shall:

- a. Render determinations in writing with regard to whether or not each item considered under Specification 6.5.1.6.a through e constitutes an unreviewed safety question.
- b. Provide written notification within 24 hours to the Vice President - Nuclear Generation and the Safety Review and Audit Board of disagreement between the SORC and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1.

RECORDS

6.5.1.8 The SORC shall maintain written minutes of each SORC meeting that at a minimum, document the result of all SORC activities performed under the responsibilities and authority provisions of these Technical Specifications. Copies shall be provided to the Vice President - Nuclear Generation and the Safety Review and Audit Board.

6.5.2 TECHNICAL REVIEW AND CONTROL ACTIVITIES

6.5.2.1 Each procedure and program required by Specification 6.8 and other procedures that affect nuclear safety, and changes thereto, shall be prepared by a qualified individual/organization. Each such procedure, and changes thereto, shall be reviewed by an individual/group other than the individual/group that prepared the procedure, or changes thereto, but who may be from the same organization as the individual/group that prepared the procedure, or changes thereto. Approval of procedures and programs and changes thereto and their safety evaluations, shall be controlled by administrative procedures.

6.5.2.2 Proposed changes to the Technical Specifications shall be prepared by a qualified individual/organization. The preparation of each proposed Technical Specifications change shall be reviewed by an individual/group other than the individual/group that prepared the proposed change, but who may be from the same organization as the individual/group that prepared the proposed change.

ADMINISTRATIVE CONTROLS

REVIEW AND AUDIT

TECHNICAL REVIEW AND CONTROL ACTIVITIES

6.5.2.2 (Continued)

Proposed changes to the Technical Specifications shall be approved by the Plant Manager.

6.5.2.3 Proposed modifications to unit structures, systems, and components that affect nuclear safety shall be designed by a qualified individual/organization. Each such modification shall be reviewed by an individual/group other than the individual/group that designed the modification, but who may be from the same organization as the individual/group that designed the modification. Proposed modifications to structures, systems, and components and the safety evaluations shall be approved before implementation by the Plant Manager, or the Manager Technical Support, as previously designated by the Plant Manager.

6.5.2.4 Individuals responsible for reviews performed in accordance with Specifications 6.5.2.1, 6.5.2.2, and 6.5.2.3 shall be members of the station supervisory staff, previously designated by the Plant Manager to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary such review shall be performed by the appropriate designated station review personnel.

6.5.2.5 Proposed tests and experiments that affect station nuclear safety and are not addressed in the FSAR or Technical Specifications and their safety evaluations shall be reviewed by the Plant Manager, or the Manager Technical Support, as previously designated by the Plant Manager.

6.5.2.6 The Plant Manager shall assure the performance of special reviews and investigations, and the preparation and submittal of reports thereon, as requested by the Vice President - Nuclear Generation.

6.5.2.7 The facility security program and implementing procedures shall be reviewed at least every 12 months. Recommended changes shall be approved by the Plant Manager and transmitted to the Vice President - Nuclear Generation, and to the Chairman of the Safety Review and Audit Board.

6.5.2.8 The facility emergency plan and implementing procedures shall be reviewed at least every 12 months. Recommended changes shall be approved by the Plant Manager and transmitted to the Vice President - Nuclear Generation and to the Chairman of the Safety Review and Audit Board.

ADMINISTRATIVE CONTROLS

REVIEW AND AUDIT

TECHNICAL REVIEW AND CONTROL ACTIVITIES

6.5.2.9 The Plant Manager shall assure the performance of a review by a qualified individual/ organization of changes to the Radiological Waste Treatment systems.

6.5.2.10 Review of any accidental, unplanned, or uncontrolled radioactive release, including the preparation of reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President - Nuclear Generation and to the Safety Review and Audit Board.

6.5.2.11 Review of changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL. Approval of any changes shall be made by the Plant Manager or a designee before implementation of such changes.

6.5.2.12 Reports documenting each of the activities performed under Specifications 6.5.2.1 through 6.5.2.9 shall be maintained. Copies shall be provided to the Vice President - Nuclear Generation and the Safety Review and Audit Board.

6.5.3 SAFETY REVIEW AND AUDIT BOARD

FUNCTION

6.5.3.1 The Safety Review and Audit Board (SRAB) shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations
- b. Nuclear engineering
- c. Chemistry and radiochemistry
- d. Metallurgy
- e. Instrumentation and control
- f. Radiological safety
- g. Mechanical and electrical engineering
- h. Quality assurance practices and
- i. Other appropriate fields associated with the unique characteristics of the nuclear power plant.

The SRAB shall report to and advise the Chief Nuclear Officer on those areas of responsibility in Specifications 6.5.3.7 and 6.5.3.8.

COMPOSITION

6.5.3.2 The SRAB shall be composed of the:

Chairman:	Vice President, Manager or Staff Engineer
Member:	Plant Manager or Designee
Member:	Staff Engineer - Nuclear
Member:	Staff Engineer - Mechanical or Electrical
Member:	Consultant (Specification 6.5.3.4)

ADMINISTRATIVE CONTROLS

REVIEW AND AUDIT

SAFETY REVIEW AND AUDIT BOARD

AUDITS

6.5.3.8 (Continued)

- j. Any other area of unit operation considered appropriate by the SRAB or the Vice President - Nuclear Generation.
- k. The Fire Protection Program and implementing procedures at least once per 24 months.
- l. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- m. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 36 months.

AUTHORITY

6.5.3.9 The SRAB shall report to and advise the Chief Nuclear Officer on those areas of responsibility specified in Sections 6.5.3.7 and 6.5.3.8.

RECORDS

6.5.3.10 Records of SRAB activities shall be prepared, approved, and distributed as indicated below:

- a. Minutes of each SRAB meeting shall be prepared, approved, and forwarded to the Chief Nuclear Officer within 14 days following each meeting.
- b. Reports of reviews encompassed by Specification 6.5.3.7b, e, g, h shall be prepared, approved, and forwarded to the Chief Nuclear Officer within 14 days following completion of the review.
- c. Audit reports encompassed by Specification 6.5.3.8 shall be forwarded to the Chief Nuclear Officer and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

ADMINISTRATIVE CONTROLS

6.6 REPORTABLE EVENT ACTION

The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of 10 CFR 50.72 and 10 CFR 50.73, and
- b. Each REPORTABLE EVENT shall be reviewed by the SORC, and the results of this review shall be submitted to the SRAB and the Vice President - Nuclear Generation.

6.7 SAFETY LIMIT VIOLATION

The following actions shall be taken in the event a Safety Limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Vice President - Nuclear Generation and the SRAB shall be notified within 24 hours.
- b. A Safety Limit Violation Report shall be prepared in accordance with 10 CFR 50.73. The report shall be reviewed by the SORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon unit components, systems, or structures, and (3) corrective action taken to prevent recurrence.
- c. The Safety Limit Violation Report shall be submitted to the Commission within 30 days of the violation, and to the SRAB, and the Vice President - Nuclear Generation within 14 days.
- d. Critical operation of the unit shall not be resumed until authorized by the Commission.

6.8 PROCEDURES AND PROGRAMS

PROCEDURES

6.8.1 Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory guide 1.33, Revision 2, February 1978
- b. The applicable procedures required to implement the requirements of NUREG-0737
- c. Refueling operations



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENTS NOS. 162 AND 83 TO FACILITY OPERATING

LICENSE NOS. DPR-63 AND NPF-69

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-220 AND 50-410

1.0 INTRODUCTION

By two letters dated May 15, 1998, Niagara Mohawk Power Corporation (NMPC or the licensee) submitted applications for license amendments to change the Technical Specifications (TSs) for Nine Mile Point Nuclear Station, Unit Nos. 1 and 2 (NMP1 and NMP2). The proposed changes would revise portions of TS Chapter 6, "Administrative Controls," to reflect the restructuring of NMPC's Nuclear Division upper management organization. The proposed changes involve reassignment of responsibility at the upper management level, reporting changes, and title changes.

2.0 BACKGROUND

NMPC proposes to restructure the Nuclear Division upper management organization for NMP1 and NMP2. The proposed amendment reflects these reassignments of Nuclear Division upper management which the licensee proposes in order to provide more direct lines of authority, to improve management efficiency, and to provide for better oversight of plant operation and engineering.

The general features of the Nuclear Division organizational restructuring include the elimination of the Vice President and General Manager - Nuclear position and the establishment of the Vice President - Nuclear Generation position. The Chief Nuclear Officer will assume corporate and TS responsibility for overall plant nuclear safety, which has previously been assigned to the Vice President and General Manager - Nuclear. The TS responsibility for plant operation, which had also been previously assigned to the Vice President and General Manager - Nuclear, will be assumed by the Vice President - Nuclear Generation. The new Vice President - Nuclear Generation position will report directly to the Chief Nuclear Officer.

Under the proposed organizational structure, the Chief Nuclear Officer will have overall responsibility for oversight of the Nuclear Division, which will include: Nuclear Communications and Public Affairs; Nuclear Generation; Nuclear Engineering; Nuclear Safety Assessment and Support; Business Management; and Human Resources and Development. The Vice President and General Manager - Nuclear position is being eliminated to provide the Chief Nuclear Officer with direct reporting from the Vice Presidents responsible for Nuclear Generation and Engineering. Upon approval and implementation of the proposed organizational structure, the Chief Nuclear Officer will maintain the TS responsibilities currently assigned to the position as

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defined in TS 6.5.3.9 and TS 6.5.3.10. In addition, the Chief Nuclear Officer will assume corporate and TS responsibility for overall plant nuclear safety, as proposed in TS 6.2.1.b, which had previously been assigned to the Vice President and General Manager - Nuclear. The Chief Nuclear Officer will also assume the authority to take such measures as may be needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to assure nuclear safety as proposed in TS 6.2.1.b. This will restore the Chief Nuclear Officer's responsibilities and authority to the levels that existed prior to February 20, 1996, when they were altered by implementation of NMP1 Amendment No. 157 and NMP2 Amendment No. 71. Furthermore, in addition to his previous responsibilities, NMPC proposes that the Chief Nuclear Officer be assigned the responsibility for periodically issuing management direction emphasizing the primary responsibilities of the Shift Supervisor as defined in TS 6.1.2. Prior to February 20, 1996, this was the responsibility of the Executive Vice President - Nuclear and was subsequently reassigned to the Vice President and General Manager - Nuclear by Amendment Nos. 157 and 71.

As previously described, the position of Vice President and General Manager - Nuclear was established to reflect the assignment of additional responsibilities beyond those previously associated with the Vice President - Nuclear Generation position. The proposed organizational structure reestablishes the former position and responsibilities of Vice President - Nuclear Generation and eliminates the position of Vice President and General Manager - Nuclear. The Vice President - Nuclear Generation will assume TS responsibility for plant operation, but will be under the direction of the Chief Nuclear Officer to assure required oversight concerning nuclear safety matters. Specifically, the Vice President Nuclear Generation will have oversight responsibility for the NMP1 and NMP2 Operations, Radiation Protection, Maintenance, Chemistry, Technical Support, and Outage Management functions to assure safe, orderly, and efficient plant operation through direct reporting of both Unit Plant Managers.

The changes for NMP1 would also correct a clerical error in which a previous Amendment No. (No. 144) was omitted when designating superseded amendments during preparation of prior Amendment No. 157.

3.0 EVALUATION

The proposed changes to TS Chapter 6 reflect organizational title changes, reporting changes, and revised responsibilities. Proposed changes to the TS which involve a change in management and reporting responsibilities are TSs 6.1.2 and 6.2.1.b (both units), 6.2.2.h (Unit 1 only), and 6.2.2.i (Unit 2 only). The NRC staff uses Standard Review Plan (SRP) Sections 13.1.1, 13.1.2, and 13.4 to review such changes.

In TS 6.1.2 and 6.2.1.b, the licensee is transferring the corporate responsibility for overall plant nuclear safety from the current Vice President and General Manager - Nuclear (a position which is being eliminated) to the Chief Nuclear Officer. SRP Section 13.1.1 states that a corporate officer should clearly be responsible for nuclear activities without having ancillary responsibilities that might detract from his attention to nuclear safety matters. The overall responsibility for plant nuclear safety will remain with the highest ranking executive at the site who is responsible for operations, maintenance, and engineering and whose full-time responsibility is the Nine Mile Point Nuclear Station. These changes proposed to TSs 6.1.2 and 6.2.1.b for each unit are in accordance with SRP 13.1.1 and are acceptable. Also, in TS 6.2.1.b, NMPC proposes a change to eliminate the third sentence which was added to TS 6.2.1.b as part of Amendment Nos. 157 and 71 to clarify the shared responsibilities for Nuclear Division oversight involving the Vice

President and General Manager - Nuclear and the Chief Nuclear Officer. As previously discussed, the position of Vice President and General Manager - Nuclear is being eliminated and the Chief Nuclear Officer position and responsibilities are being restored. Thus, there will no longer be a separation of Nuclear Division oversight responsibilities and the associated clarification provided in TS 6.2.1.b is no longer necessary. The Nuclear Safety Assessment and Support (NSAS) organization currently reports to the Chief Nuclear Officer. The proposed modifications will not affect this reporting structure. SRP Section 13.4, which references ANSI N18.7/ANS 3.2, includes a criterion for independent review organizations, stating that the organization should be independent of line responsibility for operating activities. Since NSAS will report to the Chief Nuclear Officer directly and independently of the operating organizations, the proposed change is in accordance with SRP 13.4 and is acceptable.

The proposed changes to NMP1 TS 6.2.2.h and NMP2 TS 6.2.2.i replace the position title of Vice President and General Manager - Nuclear with the position title of Vice President Nuclear Generation. This change reflects the reassignment of responsibility for oversight to assure safe, orderly, and efficient plant operation from the Vice President and General Manager - Nuclear position to the Vice President - Nuclear Generation position. As defined in TSs 6.2.2.h and 6.2.2.i, the Vice President - Nuclear Generation will assume the responsibility for reviewing individual staff overtime on a monthly basis to assure that adequate shift staffing is maintained without the excessive use of overtime. These changes proposed to NMP1 TS 6.2.2.h and NMP2 TS 6.2.2.i are in accordance with SRP Sections 13.1.1 and 13.1.2, and are acceptable.

Although there are no TS changes proposed to the quality assurance (QA) organization, the NRC staff reviewed whether adverse changes in QA reporting might result from the upper level management changes. SRP Section 13.4, by reference to Section 4.3 of ANSI N18.7/ANS3.2, states that an organizational unit functioning as an independent review body, should report to a manager who is not immediately responsible for the performance being reviewed. QA will continue to report through the division of safety assessment, independent of the line organization for operations, maintenance, and engineering. This independent reporting arrangement is in accordance with SRP 13.4 and is acceptable.

All of the remaining proposed TS changes to the NMPC organization are simple administrative changes to reflect the new title of the Vice President - Nuclear Generation position, and are therefore, acceptable. These changes for NMP1 are made in TSs 6.5.1.6.d, 6.5.1.7.b, 6.5.1.8, 6.5.2.6, 6.5.2.7, 6.5.2.8, 6.5.2.10, 6.5.2.12, 6.5.3.8.h, 6.6.1.b, 6.7.1.b, and 6.7.1.d. Corresponding changes for NMP2 are made in TSs 6.2.2.i, 6.5.1.6.a, 6.5.1.7.b, 6.5.1.8, 6.5.2.6, 6.5.2.7, 6.5.2.8, 6.5.2.10, 6.5.2.12, 6.5.3.8.j, 6.6 item b, 6.7 item a, and 6.7 item c.

The proposed change to NMP1 TS page 360 corrects a clerical error in which a previous Amendment No. (No. 144) was omitted when designating superseded amendments during preparation of prior Amendment No. 157. This correction is purely administrative and is acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: G. Galletti
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Date: July 7, 1998