

March 25, 1988

Docket No. 50-220

Mr. C. V. Mangan
Senior Vice President
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Dear Mr. Mangan:

DISTRUBTION

Docket File
NRCPDR
Local PDR
PDI-1 Rdg.
SVarga
BBoger
CVogan
DNeighbors
BBenedict
JJohnson

DHagan
EJordan
JPartlow
TBarnhart(4)
Wanda Jones
EButcher
ACRS(10)
GPA/PA
ARM/LFMB

The Commission has issued the enclosed Amendment No. 96 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station Unit 1 (NMP-1). The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated January 21, 1988 (TAC 67070).

This amendment revises Technical Specification Table 3.2.7 to delete the Main Steam Warmup Valves. These valves are being removed from the plant because they are not required.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Robert A. Benedict, Project Manager
Project Directorate I-1
Division of Reactor Projects, I/II

Enclosures:

1. Amendment No. 96 to DPR-63
2. Safety Evaluation

cc: w/enclosures
See next page

PDI-1
CVogan *CV*
3/2/88
CV
3/23/88

RA B 3/24/88
PDI-1 *RA B*
DNeighbors:dlg
3/2/88

RA B
SPLB
3/4/88

OGC *LKX*
SH Lewis
3/14/88

RA C
PDI-1
RCapra
3/25/88

*Wait for
Nuke
permits
before
repair*

8804050289 880325
PDR ADDCK 05000220
PDR

Mr. C. V. Mangan
Niagara Mohawk Power Corporation

Nine Mile Point Nuclear Station,
Unit No. 1

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 96
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation of New York, Inc. (the licensee) dated January 21, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

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PDR ADOCK 05000220
P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 96, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Capra

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects, I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 25, 1988

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 96 TO FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise Appendix A as follows:

Remove Page

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Insert Page

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LIMITING CONDITIONS FOR OPERATION
Table 3.2.7

REACTOR COOLANT SYSTEM ISOLATION VALVES

Line or System	No. of Valves (Each Line)	Location Relative to Primary Containment	Normal Position	Motive Power	Oper. Time (Sec)	Action on Initiating Signal	Initiating Signal (All Valves Have Remote Manual Backup)
<u>Main Steam</u> (Two Lines)	1 1	Inside Outside	Open Open	A.I.P.O.* A.I.P.O.*	10 10	Close Close	Reactor water level low-low, or main steam line high radia- tion, or main steam line high flow, or low condenser vacuum or high temperature in the pipe tunnel
<u>Main Steam-Emergency Cooling Vents</u> (Two Lines)	2	Outside	Open	A.I.P.O.	5	Close	
<u>Feedwater</u> (Two Lines)	1 1	Outside Outside	Open -	R.M.P.O.* Self Act. Ck.	60 --	- -	
<u>Emergency Cooling</u>							
<u>Steam Leaving Reactor</u> (Two Lines)	1 1	Outside Outside	Open Open	A.I.P.O. A.I.P.O.	38 38	Close Close	High system flow
<u>Condenser Return to Reactor</u> (Two Lines)	1 1	Inside Outside	- Closed	Self Act. Ck. A.I.P.O.	-- 60	- Close	
<u>Reactor Cleanup</u>							
<u>Water Leaving Reactor</u> (One Line)	1 1	Inside Outside	Open Open	A.I.P.O. A.I.P.O.	18 18	Close Close	Reactor water level low-low, or high area temperature, liquid poison initiation or high system pressure, or low system flow, or high system temperature
<u>Water Return to Reactor</u> (One Line)	1 1	Inside Outside	Open -	A.I.P.O. Self Act. Ck.	18 --	Close -	
<u>Shutdown Cooling</u>							
<u>Water Leaving Reactor</u> (One Line)	1 1	Inside Outside	Closed Closed	A.I.P.O. A.I.P.O.	40 40	Close Close	Reactor water level low-low, or high area temperature
<u>Water Return to Reactor</u> (One Line)	1 1	Inside Outside	Closed -	A.I.P.O. Self Act. Ck.	40 --	Close -	



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 96 TO FACILITY OPERATING LICENSE NO. DPR-63
NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT NUCLEAR POWER STATION, UNIT NO. 1
DOCKET NO. 50-220

INTRODUCTION

By letter dated January 21, 1988, Niagara Mohawk Power Corporation (the licensee) requested an amendment to Facility Operating License No. DPR-63 for Nine Mile Point Nuclear Power Station, Unit 1. The amendment would revise Technical Specification Table 3.2.7 to delete the Main Steam Warmup Valves.

DISCUSSION AND EVALUATION

The Main Steam Warmup Valves were originally designed to equalize temperature and pressure around the outboard Main Steam Isolation Valves during startup. After the pressures and temperatures were equal, the outboard Main Steam Isolation Valves could be opened without causing a pressure transient downstream. The licensee performs startups with all four Main Steam Isolation Valves open. This procedure warms up the steam lines as the reactor heats up. Thus, the warmup valves are not required.

The warmup valves were also designed to equalize pressure and temperature when bringing the reactor from the hot standby condition (reactor critical, pressure less than 600 psig and Main Steam Isolation Valves closed) to full power. At Nine Mile Point Unit 1, this operation is performed by fully opening the outboard air-operated isolation valves and partially opening the inboard electrically-operated isolation valves. When the pressures and temperatures are equal, the inboard valves are fully opened. Thus, the warmup valves are no longer required.

Since the warmup valves are no longer required for operational purpose, the licensee has physically removed them from the plant. The removal of the warmup valves does not affect the operation of any safety system nor the performance of the Main Steam Isolation Valves. Also, the warmup valves have no role in accident mitigation.

The removal of these valves eliminates two potential containment leakage paths.

We have concluded that this change is acceptable since the warmup valves are not needed for operation of any safety system and are not required to mitigate any accidents.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of the facility components located within the restricted areas as defined in 10 CFR 20. The staff has determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: March 25, 1988

PRINCIPAL CONTRIBUTOR:

J. Neighbors