Mr. John K. Wood Vice President - Nuclear, Perry FirstEnergy Nuclear Operating Company P.O. Box 97, A200 Perry, OH 44081

SUBJECT: PERRY NUCLEAR POWER PLANT, UNIT 1 - TIA 2000-18 DESIGN-BASIS

ASSUMPTIONS FOR NON-SEISMIC PIPING FAILURES (TAC NO. MB1629)

Dear Mr. Wood:

Region III has recently requested Nuclear Reactor Regulation (NRR) assistance in resolving an issue related to design-basis assumptions for non-seismic piping failures at the Perry facility. The issue involves assumptions being used to determine the differential pressure across a motor-operated valve in the emergency closed cooling water system.

In evaluating a motor-operated valve for Generic Letter 89-10, calculation MOV-OP42-05 assumes a through-wall leakage crack in non-seismic piping. The calculation justifies this assumption by referencing Standard Review Plan, Branch Technical Position MEB 3-1, paragraph 2.c(2) which states, "Through-wall leakage cracks should be postulated in fluid system piping designed to non-seismic standards as necessary to satisfy B.3.d of BTP ASB 3-1." Region III has questioned whether the valve operability determination should assume through-wall leakage cracks or catastrophic failure for non-safety-related [non-seismic] piping, downstream of the valve.

Previously, we discussed BTP MEB 3-1 in a letter to you dated January 27, 1999. However, the letter focused on internal flooding as a result of a potential failure of non-seismic piping at Perry. Thus, the direct applicability of the information in that letter to this issue is not clear. Therefore, through Task Interface Agreement (TIA) 2000-18, Region III has requested that NRR respond to the following concerns:

- When evaluating the differential pressure across a motor-operated valve, does NRR believe that non-seismically supported piping will only leak as specified in MEB 3-1, or should the evaluation assume a complete rupture of downstream non-seismic piping?
- 2) In general, for determining the operability of a structure, system or component, does NRR believe that it is technically adequate to assume that non-seismic piping, for which the licensing/design basis does not take credit in the mitigation of accidents, will only leak instead of completely breaking?

The purpose of this letter is to provide you with the opportunity to respond, in writing, to the issues identified in the TIA. Our internal procedures encourage the input of licensees or other external stakeholders in order to document the basis for responding to a TIA. While you are not required to respond to this letter, your staff has previously indicated a desire to provide input for our evaluation.

Mr. Wood - 2 -

If you decide to respond to this letter, we request that your response be provided within 60 days of receipt of this letter. Please feel free to contact me at 301-415-1364 if you have any questions.

Sincerely,

/RA/

Douglas V. Pickett, Sr. Project Manager, Section 2 Project Directorate III Division of Licensing Project Management Office of Nuclear Reactor Regulation

Docket No. 50-440

cc: See next page

Mr. Wood - 2 -

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Sincerely,

## /RA/

Douglas V. Pickett, Sr. Project Manager, Section 2 Project Directorate III Division of Licensing Project Management Office of Nuclear Reactor Regulation

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cc: See next page

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J. Wood FirstEnergy Nuclear Operating Company

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