

May 23, 1988

Docket No. 50-220

Mr. C. V. Mangan
Senior Vice President

Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Dear Mr. Mangan:

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BBenedict		ACRS(10)
MHaughey		GPA/PA
OGC-WF		ARM/LFMB
JJohnson		CCheng

The Commission has issued the enclosed Amendment No. 98 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station Unit 1 (NMP-1). The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated January 29, 1988 (TACS 67170 and 66551).

This amendment revises Technical Specifications 3.2.6 and 4.2.6 of NMP-1 concerning the Inservice Inspection (ISI) and Inservice Testing (IST) Programs. In addition, in a letter dated December 29, 1986, you stated your intention to use ASME Code Case N-409 as an alternate qualification program for inspectors for the augmented inspection program (specifically for intergranular stress corrosion cracking (IGSCC) inspectors). That code case was also referenced in the proposed revised Bases for 3.2.6 and 4.2.6. The staff has determined that the use of that code case as requested for IGSCC inspectors is not acceptable. Consequently, the reference to that code case has been deleted from the revised Bases (enclosed). This change was discussed with Mr. Peter Francisco of your staff on April 12, 1988.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Original signed by

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects, I/II

Enclosures:

1. Amendment No. 98 to DPR-63
2. Safety Evaluation

cc: w/enclosures
See next page

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P PDR

* SEE PREVIOUS CONCURRENCES

PDI-1
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3/28/88 *CV 5/10/88*

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*MHaughey:dlg
4/1/88

PDI-1
*RBenedict
4/4/88

BC:EMEB
*TMarsh
4/11/88

BC:EMTB
*CCheng
4/13/88

OGC/MY
*MYoung
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This amendment revises Technical Specifications 3.2.6 and 4.2.6 of NMP-1 concerning the Inservice Inspection (ISI) and Inservice Testing (IST) Programs. In addition, in a letter dated December 29, 1986, you stated your intention to use ASME Code Case N-409 as an alternate qualification program for inspectors for the augmented inspection program (specifically for intergranular stress corrosion cracking (IGSCC) inspectors). That code case was also referenced in the proposed revised Bases for 3.2.6 and 4.2.6. The staff has determined that the use of that code case as requested for IGSCC inspectors is not acceptable. Consequently, the reference to that code case has been deleted from the revised Bases (enclosed). This change was discussed with Mr. Peter Francisco of your staff on April 12, 1988.

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Robert A. Capra, Director
Project Directorate I-1
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4/1/88

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*MYoung
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The Commission has issued the enclosed Amendment No. to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station Unit 1 (NMP-1). The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated January 29, 1988 (TAC 67170).

This amendment revises Technical Specifications 3.2.6 and 4.2.6 of NMP-1 concerning the Inservice Inspection (ISI) and Inservice Testing (IST) Programs. In addition, in a letter dated December 29, 1986, you stated your intention to use ASME Code Case N-409 as an alternate qualification program for inspectors for the augmented inspection program (specifically for intergranular stress corrosion cracking (IGSCC) inspectors). That code case was referenced in the proposed revised Bases for 3.2.6 and 4.2.6. The staff has determined that the use of that code case as requested for IGSCC inspectors is not acceptable. Consequently, the reference to that code case has been deleted from the revised Bases (enclosed). This change was discussed with Mr. Peter Francisco of your staff on April 12, 1988.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

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Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects, I/II

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Dear Mr. Mangan:

The Commission has issued the enclosed Amendment No. to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station Unit 1 (NMP-1). The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated January 29, 1988 (TAC 67170).

This amendment revises Technical Specifications 3.2.6 and 4.2.6 of NMP-1 concerning the Inservice Inspection (ISI) and Inservice Testing (IST) Programs. In addition, in a letter dated December 29, 1986, you stated your intention to use ASME Code Case N-409 as an alternate qualification program for inspectors for the augmented inspection program. Pursuant to 10CFR 50.55a(a)(3)(i), and on the basis discussed in the enclosed safety evaluation, relief is granted for use of this code case at NMP-1.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects, I/II

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4/4/88

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3/ /88

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TMarsh
3/11/88
4/11

Mr. C. V. Mangan
Niagara Mohawk Power Corporation

Nine Mile Point Nuclear Station,
Unit No. 1

cc:

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Albany, New York 12223



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 98
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated January 29, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 98 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects, I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 23, 1988

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 98 TO FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
92	92
93	93
94	94

LIMITING CONDITION FOR OPERATION

3.2.6 INSERVICE INSPECTION AND TESTING

Applicability:

Applies to components which are part of the reactor coolant pressure boundary and their supports and other safety-related pressure vessels, piping, pumps, and valves.

Objective:

To assure the integrity of the reactor coolant pressure boundary and the operational readiness of safety-related pressure vessels, piping, pumps, and valves.

Specification:

a. Inservice Inspection

1. To be considered operable, Quality Group A, B and C components shall satisfy the requirements contained in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda for continued service of ASME Code Class 1, 2 and 3 components, respectively, except where relief has been granted by the Commission pursuant to 10CFR50, Section 50.55a(g)(6)(i).

SURVEILLANCE REQUIREMENT

4.2.6 INSERVICE INSPECTION AND TESTING

Applicability:

Applies to periodic inspection and testing of components which are part of the reactor coolant pressure boundary and their supports and other safety-related pressure vessels, piping, pumps, and valves.

Objective:

To verify the integrity of the reactor coolant pressure boundary and the operational readiness of safety-related pressure vessels, piping, pumps, and valves.

Specification:

a. Inservice Inspection

1. Inservice inspection of Quality Group A, B and C components shall be performed in accordance with the requirements for ASME Code Class 1, 2 and 3 components, respectively, contained in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10CFR50, Section 50.55a(g), except where relief has been granted by the Commission pursuant to 10CFR part 50 Section 50.55a(g)(6)(i).

LIMITING CONDITION FOR OPERATION

SURVEILLANCE REQUIREMENT

b. Inservice Testing

1. To be considered operable, Quality Group A, B and C pumps and valves shall satisfy the requirements contained in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda for continued service of ASME Code Class 1, 2 and 3 components, respectively, except where relief has been granted by the Commission pursuant to 10CFR50, Section 50.55a(g)(6)(i).

2. An augmented inservice inspection program shall be performed in accordance with the schedules contained in NUREG 0313 Revision 1. The augmented inservice inspection program shall be performed on all nonconforming components. The systems containing nonconforming components shall be identified in the Inservice Inspection Program.

b. Inservice Testing

1. Inservice testing of Quality Group A, B and C pumps and valves shall be performed in accordance with the requirements for ASME Code Class 1, 2 and 3 components contained in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10CFR50, Section 50.55a(g), except where relief has been granted by the Commission pursuant to 10CFR Part 50 Section 50.55a(g)(6)(i).

BASES FOR 3.2.6 AND 4.2.6 INSERVICE INSPECTION AND TESTING

The inservice inspection and testing programs⁽¹⁾⁽²⁾ for the Nine Mile Point Unit 1 plant conform to the requirements of 10CFR50, Section 50.55a(g). Where practical, the inspection of components, pumps and valves classified into NRC Quality Groups A, B and C conforms to the requirements of ASME Code Class 1, 2 and 3 components, pumps and valves, respectively, contained in Section XI of the ASME Boiler and Pressure Vessel Code. If a Code required inspection is impractical for the Nine Mile Point Unit 1 facility, a request for relief from that requirement is submitted to the Commission in accordance with 10CFR50, Section 50.55a(g)(6)(i).

Request for relief from the requirements of Section XI of the ASME Code and applicable Addenda will be submitted to the Commission prior to the beginning of each 10-year inspection interval if they are known to be required at the time. Requests for relief which are identified during the course of inspection will be submitted quarterly throughout the inspection interval.

The augmented inservice inspection program for the Nine Mile Point Unit 1 plant conforms to the schedules contained in NUREG 0313 Revision 1. It is performed in order to detect and survey intergranular stress corrosion cracking of ASME Code Class 1, 2 and 3 pressure boundary piping. Inspections shall be performed by individuals qualified to: (A) the ASME Boiler and Pressure Vessel Code, Section XI, and (B) Ultrasonic Testing Operator Training for the Detection of Intergranular Stress Corrosion Cracking developed by the EPRI Non-Destructive Examination Center. As an alternate, Niagara Mohawk may use other qualification programs approved by the NRC.

References

- (1) Letter from the Nuclear Regulatory Commission (D. B. Vassallo) to Niagara Mohawk Power Corporation (G. K. Rhode), dated September 19, 1983.
- (2) Letter from Niagara Mohawk Power Corporation (D. P. Dise) to the Nuclear Regulatory Commission (T. A. Ippolito), dated August 7, 1981.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 98 TO FACILITY OPERATING LICENSE NO. DPR-63
NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT NUCLEAR POWER STATION UNIT NO. 1
DOCKET NO. 50-220

INTRODUCTION

In a letter dated January 29, 1988, the licensee requested the Technical Specification for Nine Mile Point Nuclear Station Unit 1 (NMP-1) be amended to move the footnotes referencing the specific Inservice Inspection (ISI) and Inservice Testing (IST) Programs in effect from Technical Specifications sections 3.2.6 and 4.2.6 to the Bases. Inasmuch as the Bases are not part of the Technical Specifications, moving the reference to the Bases would allow the licensee to update the ISI and IST programs, as required every ten years, without amending the Technical Specifications.

The licensee's letter of January 29, 1988 also requested Technical Specification 4.2.6.a.2 be revised to delete the list of systems containing non-conforming components.

In addition to the above listed changes to the Technical Specifications, the licensee proposed changes to the Bases for sections 3.2.6 and 4.2.6. The Bases will be revised to allow inspectors to be qualified to alternate qualification programs approved by the NRC, as well as to the ASME Boiler and Pressure Vessel Code (hereinafter referred to as the ASME Code) Section XI and the Ultrasonic Testing Operator Training for the Detection of Intergranular Stress Corrosion Cracking (IGSCC) developed by the EPRI Non-Destructive Examination Center. The specific alternate program referenced is ASME Code Case N-409 which previously was proposed for use by IGSCC inspectors in a letter dated December 29, 1986. On April 12, 1988 the staff informed the licensee that due to the unacceptability of the Code Case, it had been deleted from the proposed TS. This change did not alter or affect the action noticed or the staff initial determination published in the Federal Register on March 9, 1988. Additional, editorial changes to the Bases were also proposed and references to the ISI and IST programs in effect were added.

EVALUATION

Moving the footnotes referencing the specific ISI and IST programs in effect to the Bases would permit the references to be revised without submitting a Technical Specification amendment. This would be an administrative convenience and would not alter substance program requirements. 10 CFR 50.55a requires the

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ISI and IST programs to be updated every ten years to the latest ASME Code approved by the Nuclear Regulatory Commission. This regulation further requires that relief from, or alternative requirements to, the Code in effect must be authorized by the NRC. The revised Technical Specification will not alter the requirement that licensee meet code requirements. This proposed change to the Technical Specifications, therefore, would not degrade the requirements associated with the ISI and IST programs, but would allow an administrative convenience in the process of periodically updating these programs as required by 10 CFR 50.55a. This change is, therefore, acceptable to the staff.

The additional change to Technical Specification 4.2.6.a.2 concerned deleting the list of systems containing non-conforming components from the Technical Specification. The revised Technical Specification will state that, "The systems containing nonconforming components shall be identified in the Inservice Inspection Program." As these systems will be identified in the ISI Program, this change to the Technical Specification is acceptable.

The licensee proposed additional changes to the Bases. Although the Bases are not considered part of the Technical Specifications and, therefore, do not require a Technical Specification amendment to modify them, an evaluation of these changes is included in this safety evaluation for convenience.

The most significant revision to the Bases for 3.2.6 and 4.2.6 concerns the use of alternate qualification programs approved by the NRC for inspectors for the augmented inspection program. The referenced alternate program is that defined by ASME Code Case N-409. In a letter dated December 29, 1986 the licensee stated its intention to use ASME Code Case N-409, "Procedure and Personnel Qualification for Ultrasonic Detection and Sizing of Intergranular Stress Corrosion Cracking in Austenitic Piping Welds, Section XI, Division 1." The licensee's December 29, 1986 letter stated that Code Case N-409 would be used as an approved alternate program to the EPRI program for qualifying IGSCC inspectors. The staff has determined that the use of that code case as requested for IGSCC inspectors will not provide a level of quality and safety equivalent to the EPRI program and, therefore, is not acceptable. Consequently, the reference to that code case has been deleted from the revised Bases. This change was discussed with the licensee on April 12, 1988. If the licensee intends to use other alternatives for qualification of inspectors, then the licensee should apply for relief in accordance with 10 CFR 50.55a(a)(3).

Other changes to the Bases include incorporating footnotes specifying the applicable ISI and IST programs and administrative changes. These changes are also acceptable. When the ISI and IST programs are updated in accordance with the requirements of 10 CFR 50.55a, the references to the applicable programs should also be updated.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of the facility components located within the restricted areas as defined in 10 CFR 20 and changes surveillance requirements. The staff has determined that this amendment

involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: May 23, 1988

PRINCIPAL CONTRIBUTOR:

M. Haughey