

June 3, 1986

Docket No. 50-220

Niagara Mohawk Power Corporation
Attn: Mr. C. V. Mangan
Senior Vice President
c/o Miss Catherine R. Seibert
300 Erie Boulevard West
Syracuse, New York 13202

Dear Mr. Mangan:

SUBJECT: REPORTABILITY REQUIREMENTS CONCERNING REACTIVITY ANOMALIES
(TAC 60459)

Re: Nine Mile Point Nuclear Station, Unit No. 1

The Commission has issued the enclosed Amendment No. 84 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station, Unit No. 1. This amendment is in response to your application dated December 31, 1985. The amendment modifies Technical Specification Section 3.1.1(f), by eliminating the reference to the "AEC", and by eliminating the reference to Section 6.9.2 reportability requirements pertaining to reactivity anomalies.

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notices.

Sincerely,

/S/

John A. Zwolinski, Director
BWR Project Directorate #1
Division of BWR Licensing

Enclosures:

1. Amendment No. 84 to License No. DPR-63
2. Safety Evaluation

cc w/enclosures:
See next page

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Mr. C. V. Mangan
Niagara Mohawk Power Corporation

Nine Mile Point Nuclear Station,
Unit No. 1

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 84
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated December 31, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

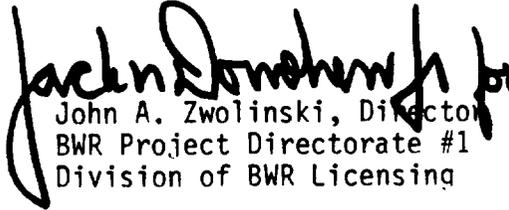
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(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 84, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John A. Zwolinski, Director
BWR Project Directorate #1
Division of BWR Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 3, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 84

FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

32

INSERT

32

LIMITING CONDITION FOR OPERATION

If a control rod with a malfunctioned accumulator is inserted "full-in" and valved out of service, it shall not be considered to have a malfunctioned accumulator.

- e. If specification 3.1.1.a through d, above, are not met, the reactor shall be placed in the hot shutdown condition within ten hours except as noted in 3.1.1.a(2).
- f. **Reactivity Anomalies**

The difference between an observed and predicted control rod inventory shall not exceed the equivalent of one percent in reactivity. If this limit is exceeded, the reactor shall be brought to the cold, shutdown condition by normal orderly shutdown procedure. Operation shall not be permitted until the cause has been evaluated and the appropriate corrective action has been completed.

SURVEILLANCE REQUIREMENT

- f. **Reactivity Anomalies**

The observed control rod inventory shall be compared with a normalized computed prediction of the control rod inventory during startup, following refueling or major core alteration. These comparisons will be used as base data for reactivity monitoring during subsequent power operation throughout the fuel cycle. At specific power operating conditions, the actual control rod configuration will be compared with the expected configuration based upon appropriately corrected past data. This comparison will be made every equivalent full power month.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 84 TO FACILITY OPERATING LICENSE NO. DPR-63
NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT NUCLEAR STATION, UNIT NO. 1
DOCKET NO. 50-220

1.0 INTRODUCTION

By letter dated December 31, 1985, Niagara Mohawk Power Corporation (the licensee) requested an amendment to Appendix A of Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station, Unit No. 1. The proposed amendment would modify Technical Specification (TS) Section 3.1.1(f), by eliminating the reference to the "AEC", and by eliminating the reference to Section 6.9.2 reportability requirements pertaining to reactivity anomalies. The proposed change does not present a new procedure or situation, but merely clarifies an existing one.

2.0 EVALUATION

If the difference between an observed and predicted control rod inventory exceeds the equivalent of 1 percent in reactivity, current TS require notification of the "AEC" within 24 hours in accordance with Specification 6.9.2. Reference to Section 6.9.2 is an inappropriate requirement and unnecessary due to: (1) the AEC's reorganization, (2) the reportability requirements of Sections 50.72 and 50.73 to 10 CFR Part 50, and (3) the intent of Section 6.9.2 being for fire protection related matters only.

Sections 50.72 and 50.73 to 10 CFR Part 50 require the licensee to notify the NRC as soon as practical and in all cases within 1 hour if a TS requiring plant shutdown occurs. This reportability requirement is also found in TS Section 6.6.1. Therefore, if a plant shutdown were to occur due to the conditions covered under Section 3.1.1(f), it would be a plant shutdown required by the TS and thereby reportable in accordance with Section 6.6.1. Additionally, the referenced Section 6.9.2 outlines actions that must be taken with respect to Fire Protection Program Reports. The referenced section is therefore not applicable to Section 3.1.1(f). The proposed change does not present a new procedure or situation, but merely clarifies an existing one, and therefore is acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributor:

J. Kelly

Dated: June 3, 1986