

December 29, 1986

Docket No. 50-220

Niagara Mohawk Power Corporation
Attn: Mr. C. V. Mangan
Senior Vice President
c/o Miss Catherine R. Seibert
300 Erie Boulevard West
Syracuse, New York 13202

Dear Mr. Mangan:

SUBJECT: TECHNICAL SPECIFICATIONS PER GENERIC LETTER 86-04 - POLICY STATEMENT
ON ENGINEERING EXPERTISE ON SHIFT (TAC 62965)

Re: Nine Mile Point Nuclear Station, Unit No. 1

The Commission has issued the enclosed Amendment No. 90 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station, Unit No. 1. This amendment is in response to your application dated September 15, 1986. The amendment modifies Technical Specification (TS) Sections 6.2.2, 6.3 and Table 6.2-1 to reflect changes required to conform to the Nuclear Regulatory Commission's "Policy Statement on Engineering Expertise on Shift," Generic Letter 86-04. Specifically, the Shift Technical Advisor will be a licensed Senior Reactor Operator and will also perform the function of Assistant Station Shift Supervisor. In addition, the "equivalency" option to a bachelor's degree in a scientific or engineering discipline has been removed from the Shift Technical Advisor job description and the alternative for a Professional Engineer's license has been added.

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notices.

Sincerely,

Original signed by
John A. Zwolinski, Director
BWR Project Directorate #1
Division of BWR Licensing

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Enclosures:

1. Amendment No. 90 to License No. DPR-63
2. Safety Evaluation

cc w/enclosures:
See next page

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Mr. C. V. Mangan
Niagara Mohawk Power Corporation

Nine Mile Point Nuclear Station,
Unit No. 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 90
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated September 15, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

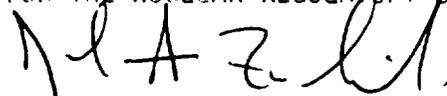
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(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 90, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John A. Zwolinski, Director
BWR Project Directorate #1
Division of BWR Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 29, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 90

FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

246
250
251

INSERT

246
250
251

Facility Staff (Cont'd)

- e. A licensed Senior Reactor Operator shall be required in the Control Room during power operations, hot shutdown, and when the emergency plan is activated. This may be the Station Shift Supervisor-Nuclear or the Assistant Station Shift Supervisor-Nuclear or another Senior Reactor Operator during power operations or hot shutdown. When the emergency plan is activated during normal operations or hot shutdown, the Assistant Station Shift Supervisor-Nuclear becomes the Shift Technical Advisor and the Station Shift Supervisor-Nuclear is restricted to the control room until an additional licensed Senior Reactor Operator arrives.
- f. A licensed Senior Reactor Operator shall be responsible for all movement of new and irradiated fuel within the site boundary. All core alterations shall be directly supervised by a licensed senior reactor operator who has no other concurrent responsibilities during this operation. A Licensed Operator will be required to manipulate the controls of all fuel handling equipment except movement of new fuel from receipt through dry storage. All fuel moves within the core shall be directly monitored by a member of the reactor analyst group.
- g. A Fire Brigade of five (5) members* shall be maintained on site as defined by 5.1 at all times.
- h. Administrative procedures shall be developed and implemented to limit the working hours of facility staff who perform safety-related functions; e.g., licensed Senior Operators, licensed Operators, health physicists, auxiliary operators and key maintenance personnel.

Adequate shift coverage shall be maintained without routine heavy use of overtime. The objective shall be to have operating personnel work a normal 8-hour day, 40-hour week while the facility is operating. However, in the event that unforeseen problems require substantial amounts of overtime to be used, or during extended periods of shutdown for refueling, major maintenance or major plant modifications on a temporary basis, the following guidelines shall be followed:

* The Radiation Protection qualified Individual and Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed two hours in order to accommodate unexpected absence, provided immediate action is taken to fill the required positions.

Table 6.2-1

MINIMUM SHIFT CREW COMPOSITION(1) (6)

<u>License</u>	<u>Normal Operation</u>	<u>Shutdown Condition</u>	<u>Operation⁽³⁾ W/O Process Computer</u>	<u>Reactor Startups</u>
Senior Operator	1	1(5)	1	1
Operator	2	1	2	3
Unlicensed ⁽²⁾	2	1	3	2
Asst. Station Shift Supervisor (Shift Technical Advisor Function) (Senior Operator License) ⁽⁷⁾	1	1(4)	1	1

Notes:

- (1) At any one time, more licensed or unlicensed operating people could be present for maintenance, repairs, refuel outages, etc.
- (2) Those operating personnel not holding an "Operator" or "Senior Operator" License.
- (3) For operation longer than eight hours without process computer.
- (4) Hot shutdown condition only.
- (5) An additional senior reactor operator who has no other concurrent responsibilities shall supervise all core alterations.
- (6) The Shift Crew Composition may be one less than the minimum requirements of Table 6.2-1 for a period of time not to exceed two hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the Shift Crew Composition to within the minimum requirements of Table 6.2-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent.
- (7) The Assistant Station Shift Supervisor performs the Shift Technical Advisor function during normal operations or hot shutdown and shall hold a senior reactor operator license. Normally the Assistant Station Shift Supervisor is a combined Assistant Station Shift Supervisor/Shift Technical Advisor; however, there may be instances when a shift may be staffed by two Senior Reactor Operators plus a dedicated Shift Technical Advisor.

6.3 Facility Staff Qualifications

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Shift Technical Advisor who shall have a bachelor's degree in a physical science or engineering or a professional engineer license issued by examination and shall have received specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 Training

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Superintendent-Training Nuclear and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Superintendent-Training Nuclear and Supervisor-Fire Protection, Nuclear and shall meet or exceed the requirements of Appendix R to 10CFR50.

6.5 Review and Audit

6.5.1 Site Operations Review Committee (SORC)

Function

6.5.1.1 The Site Operations Review Committee shall function to advise the General Superintendent-Nuclear Generation on all matters related to nuclear safety.

Composition

6.5.1.2 The Site Operations Review Committee shall be composed of the:

Chairman: General Superintendent - Nuclear Generation
Member: Station Superintendent - Nuclear Generation
Member: Technical Superintendent - Nuclear Generation
Member: Superintendent Technical Services - Nuclear
Member: Site Superintendent Maintenance - Nuclear
Member: Supervisor Instrument and Control - Nuclear
Member: Superintendent Chemistry and Radiation Management



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 90 TO FACILITY OPERATING LICENSE NO. DPR-63
NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT NUCLEAR STATION, UNIT NO. 1
DOCKET NO. 50-220

1.0 INTRODUCTION

Following the accident at Three Mile Island in March 1979, a number of studies were conducted to determine why the accident occurred, what factors might have contributed to its severity, and what the industry and the NRC could do to prevent the recurrence of the same or a similar accident. These studies concluded, among other things, that a number of actions should be taken to improve the ability of shift operating personnel to recognize, diagnose, and effectively deal with plant transients or other abnormal conditions.

To address these recommended improvements, the NRC initiated both short-term and long-term efforts. The short-term effort required that as of January 1, 1980, each nuclear power plant have on duty a Shift Technical Advisor (STA) whose function was to provide engineering and accident assessment advice to the Shift Supervisor in the event of abnormal or accident conditions. The STA was required to have a bachelor's degree in engineering or the equivalent and specific training in plant response to transients and accidents.

In support of the long-term effort, the staff issued Generic Letter 86-04 (GL 86-04), Policy Statement on Engineering Expertise on Shift, on February 13, 1986. The Policy Statement offers the licensees two options for meeting the current requirements for providing engineering expertise on shift (NUREG-0737, Item I.A.1.1) and meeting licensed operator staffing requirements (10 CFR 50.54(m)(2)). Option 1 provides for elimination of the separate STA position by allowing licensees to combine one of the required Senior Reactor Operator (SRO) positions with the STA position into a dual-role (SRO/STA) position. Option 2 states that a licensee may continue to use an NRC-approved STA program while meeting licensed operator staffing requirements. Licensees may use either option on each shift.

By letter dated May 14, 1986, Niagara Mohawk Power Corporation (NMPC) (the licensee), responded to GL 86-04 regarding which option would be used at the Nine Mile Point Nuclear Station, Unit No. 1 (NMP 1). NMPC stated that NMP 1 will normally use Option 1 as described in GL 86-04, except

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for those shifts which would be staffed by two SRO's plus a dedicated STA. In addition, NMPC proposed to remove the "equivalency" option to a bachelor's degree in a scientific or engineering discipline from the STA job description and introduce the alternative for a Professional Engineer's license.

2.0 EVALUATION

By application dated September 15, 1986, NMPC requested an amendment to Appendix A of Facility Operating License No. DPR-63 for the NMP1. The amendment would modify Technical Specification (TS) Sections 6.2.2, 6.3 and Table 6.2-1 to reflect changes required to conform to the Nuclear Regulatory Commission's "Policy Statement on Engineering Expertise on Shift," Generic Letter 86-04. Specifically, the STA would be a licensed SRO and would also perform the function of Assistant Station Shift Supervisor. In addition, the "equivalency" option to a bachelor's degree in a scientific or engineering discipline would be removed from the STA job description and the alternative for a Professional Engineer's license, obtained by examination, would be added.

The TS changes identified above are a result of the issuance of GL 86-04. NMP 1 has chosen to implement Option 1 as described in GL 86-04, except for those times when one or more shifts would be staffed by two SRO's plus a dedicated STA. The use of either Option 1 or Option 2 on each shift is acceptable, as stated in GL 86-04. NMP 1 also proposes to eliminate the "equivalency" option to a bachelor's degree in a scientific or engineering discipline from the STA job description, and add the alternative for a Professional Engineer's license, obtained by examination. Personnel currently filling the STA position at NMP 1 have bachelor's degrees in Engineering or in Physical Science. Removing the "equivalency" option and adding the Professional Engineer's license has also been done in order to conform to GL 86-04, and is therefore acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment relates to changes in administrative requirements. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributor: J. Kelly

Dated: December 29, 1986