

March 20, 1984

Docket No. 50-220

Mr. G. K. Rhode  
Senior Vice President  
Niagara Mohawk Power Corporation  
300 Erie Boulevard West  
Syracuse, New York 13202

Dear Mr. Rhode:

DISTRIBUTION

Docket File  
NRC PDR  
Local PDR  
ORB#2 Reading  
DEisenhut  
SNorris  
RHermann  
OELD  
SECY  
LJHarmon

ELJordan  
JMTaylor  
TBarnhart (4)  
WJones  
DBrinkman  
FCongel  
ACRS (10)  
OPA, CMiles  
RDiggs  
Gray File  
Extra - 5

The Commission has issued the enclosed Amendment No. 56 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station, Unit No. 1. The amendment changes the Technical Specifications in response to your request dated April 18, 1980 superseded by your request of April 21, 1983.

The amendment revises the Technical Specifications, Administrative Controls (Section 6.0) to: (1) accommodate an increase and improved staff in the on-site organization, correct title changes in the organization and reflect the strengthening of the Quality Assurance (QA) function by elevation of the Manager of QA to a Vice President reporting directly to the President; (2) include changes for the frequency of audits related to the Safeguards Contingency Plan and Emergency Preparedness program from every two years to annually; (3) change the provisions for entering and controlling entry to high radiation areas by imposing additional conditions similar to those included in current Standard Technical Specifications for BWRs; and (4) correct a typographical error with regard to the provisions for the fire brigade staff.

A copy of the Safety Evaluation is also enclosed.

Sincerely,

Original signed by/

Robert A. Hermann, Project Manager  
Operating Reactors Branch #2  
Division of Licensing

Enclosures:

1. Amendment No. 56 to License No. DPR-63
2. Safety Evaluation

cc w/enclosures:  
See next page

DL:ORB#2  
SNorris:ajs  
02/23/84

DL:ORB#2  
RHermann  
02/24/84

DSI:RAB  
FCongel  
02/27/84

DL:ORB#2  
DVassallo  
02/28/84

OELD  
Barnhart  
03/13/84

DL:AD-OR  
GLathas  
03/19/84

8404110062 840320  
PDR ADCK 05000220  
PDR

Mr. G. K. Rhode  
Niagara Mohawk Power Corporation  
Nine Mile Point Nuclear Station, Unit No. 1

cc:

Troy B. Conner, Jr., Esq.  
Conner & Wetterhahn  
Suite 1050  
1747 Pennsylvania Avenue, N. W.  
Washington, D. C. 20006

Mr. Robert P. Jones, Supervisor  
Town of Scriba  
R. D. #4  
Oswego, New York 13126

Niagara Mohawk Power Corporation  
ATTN: Mr. Thomas Perkins  
Plant Superintendent  
Nine Mile Point Nuclear Station  
Post Office Box 32  
Lycoming, New York 13093

U. S. Environmental Protection  
Agency  
Region II Office  
Regional Radiation Representative  
26 Federal Plaza  
New York, New York 10007

Resident Inspector  
U. S. Nuclear Regulatory Commission  
Post Office Box 126  
Lycoming, New York 13093

John W. Keib, Esquire  
Niagara Mohawk Power Corporation  
300 Erie Boulevard West  
Syracuse, New York 13202

Thomas A. Murley  
Regional Administrator  
Region I Office  
U. S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

Mr. Jay Dunkleberger  
Division of Policy Analysis and Planning  
New York State Energy Office  
Agency Building 2, Empire State Plaza  
Albany, New York 12223



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 56  
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated April 18, 1980 superseded by April 21, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

840411006B 840320  
PDR ADDCK 05000220  
PDR

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 56, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 20, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 56

FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

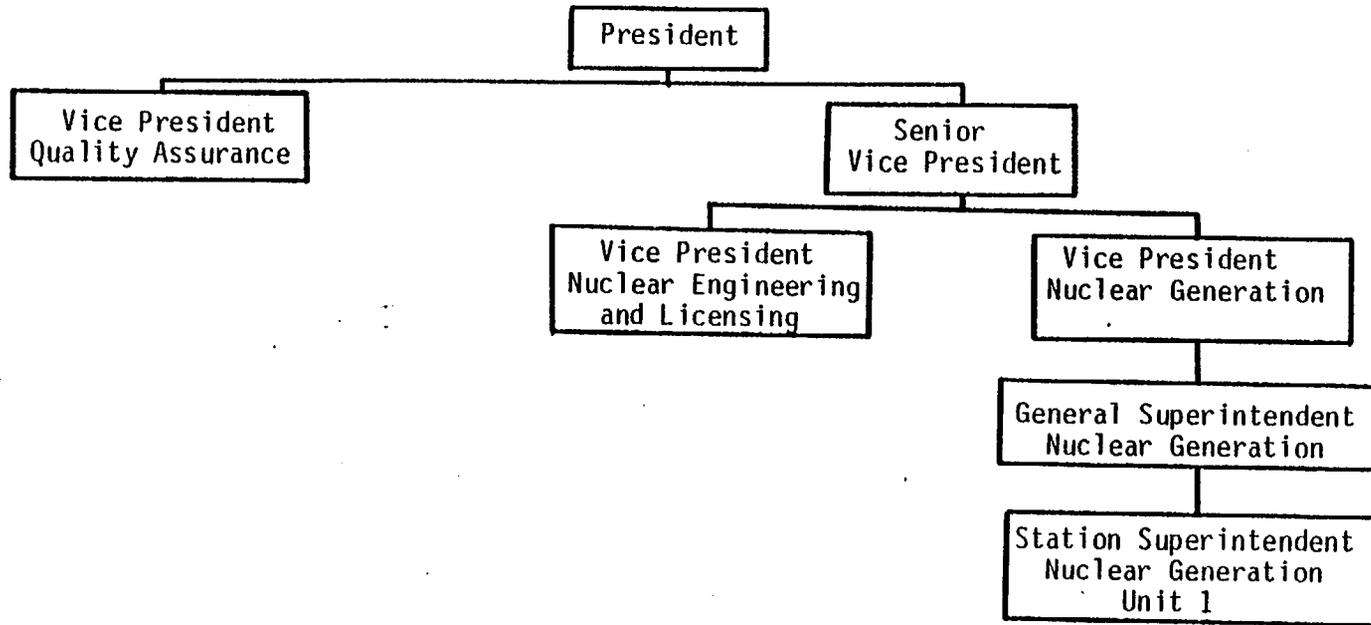
Revise the Appendix A Technical Specifications by removing and inserting the following pages. The revised area is indicated by marginal lines.

<u>Existing Page</u>	<u>Revised Page</u>
245a	245a
246	246
247	247
249	250
251	252
255	255
256	256
263	263

Facility Staff (Continued)

- f. A Fire Brigade of five (5) members shall be maintained on site as defined by 5.1 at all times.

FIGURE 6.2.-1  
NINE MILE POINT NUCLEAR STATION  
MANAGEMENT ORGANIZATION CHART





6.3 Facility Staff Qualifications

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design and response and analysis of the plant for transients and accidents.

6.4 Training

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Superintendent-Training Nuclear and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Superintendent-Training Nuclear and Supervisor-Fire Protection, Nuclear and shall meet or exceed the requirements of Appendix R to 10CFR50.

6.5 Review and Audit

6.5.1 Site Operations Review Committee (SORC)

Function

6.5.1.1 The Site Operations Review Committee shall function to advise the General Superintendent-Nuclear Generation on all matters related to nuclear safety.

Composition

6.5.1.2 The Site Operations Review Committee shall be composed of the:

Chairman:	General Superintendent - Nuclear Generation
Member:	Station Superintendent - Nuclear Generation
Member:	Technical Superintendent - Nuclear Generation
Member:	Superintendent Technical Services - Nuclear
Member:	Site Superintendent Maintenance - Nuclear
Member:	Supervisor Instrument and Control - Nuclear
Member:	Superintendent Chemistry and Radiation Management

### Alternates

- 6.5.1.3 Alternate members shall be appointed in writing by the SORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in SORC activities at any one time.

### Meeting Frequency

- 6.5.1.4 The SORC shall meet at least once per calendar month and as convened by the SORC Chairman.

### Quorum

- 6.5.1.5 A quorum of the SORC shall consist of the Chairman and four members including alternates.

### Responsibilities

- 6.5.1.6 The Site Operations Review Committee shall be responsible for:
- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the General Superintendent-Nuclear Generation to affect nuclear safety.
  - b. Review of all proposed tests and experiments that affect nuclear safety.
  - c. Review of all proposed changes to the Technical Specifications.
  - d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
  - e. Investigation of all violations of the Technical Specifications and shall prepare and forward a report covering evaluation and recommendations to prevent recurrence to the Vice President - Nuclear Generation and to the Chairman of the Safety Review and Audit Board.

## Responsibilities (Continued)

- f. Review of facility operations to detect potential safety hazards.
- g. Performance of special reviews and investigations and reports thereon as requested by the Chairman of the Safety Review and Audit Board.
- h. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Chairman of the Safety Review and Audit Board.
- i. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Chairman of the Safety Review and Audit Board.

## Authority

6.5.1.7 The Site Operations Review Committee shall:

- a. Recommend to the General Superintendent-Nuclear Generation written approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6 (a) through (e) above constitutes an unreviewed safety question.
- c. Provide immediate written notification to the Vice President-Nuclear Generation and Chairman of the Safety Review and Audit Board of disagreement between the SORC and the General Superintendent-Nuclear Generation; however, the General Superintendent-Nuclear Generation shall have the responsibility for resolution of such disagreements pursuant to 6.1.1 above.

## Records

6.5.1.8 The Site Operations Review Committee shall maintain written minutes of each meeting and copies shall be provided to the Vice President - Nuclear Generation and Chairman of the Safety Review and Audit Board.

6.5.2 Safety Review and Audit Board (SRAB)

Function

- 6.5.2.1 The Safety Review and Audit Board shall function to provide independent review and audit of designated activities in the areas of:
- a. nuclear power plant operations
  - b. nuclear engineering
  - c. chemistry and radiochemistry
  - d. metallurgy
  - e. instrumentation and control
  - f. radiological safety
  - g. mechanical and electrical engineering
  - h. quality assurance practices
  - i. (other appropriate fields associated with the unique characteristics of the nuclear power plant)

Composition

6.5.2.2 The Safety Review and Audit Board shall be composed of the:

- |           |   |
|-----------|---|
| Chairman: | Staff Engineer or Manager or Vice President |
| Member:   | General Superintendent - Nuclear Generation |
| Member:   | Staff Engineer - Nuclear                    |
| Member:   | Staff Engineer - Mechanical or Electrical   |
| Member:   | Staff Engineer - Environmental              |
| Member:   | Consultant (See 6.5.2.4)                    |

Audits (Continued)

- d. The performance of all activities required by the Quality Assurance Program to meet the criteria of Appendix "B", 10CFR50, at least once per two years.
- e. The Facility Emergency Plan and implementing procedures at least once every 12 months.
- f. The Facility Security Plan and implementing procedures at least once every 12 months.
- g. The Facility Fire Protection Program and implementing procedures at least once per two years.
- h. Any other area of facility operation considered appropriate by the SRAB, the Vice President-Nuclear Generation or the Vice President-Nuclear Engineering and Licensing.

Authority

6.5.2.9 The SRAB shall report to and advise the Vice President-Nuclear Generation and Vice President-Nuclear Engineering and Licensing on those areas of responsibility specified in Section 6.5.2.7 and 6.5.2.8.

Records

6.5.2.10 Records of SRAB activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each SRAB meeting shall be prepared, approved and forwarded to the Vice President-Nuclear Generation and Vice President-Nuclear Engineering and Licensing within 30 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 e, f, g and h above, shall be prepared, approved and forwarded to the Vice President-Nuclear Generation and Vice President-Nuclear Engineering and Licensing within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above shall be forwarded to the Vice President-Nuclear Generation and Vice President-Nuclear Engineering and Licensing within 14 days following completion of the review.

6.6 Reportable Occurrence Action

6.6.1 The following actions shall be taken in the event of a REPORTABLE OCCURRENCE:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each Reportable Occurrence Report submitted to the Commission shall be reviewed by the SORC and submitted to the SRAB and the Vice President - Nuclear Generation.

6.7 Safety Limit Violation

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10CFR50.36(c)(1)(i) shall be complied with immediately.
- b. The Safety Limit violation shall be reported to the Commission, the Vice President - Nuclear Generation and to the SRAB immediately.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the SRAB and the Vice President - Nuclear Generation within 10 days of the violation.

6.8 Procedures

6.8.1 Written procedures and administrative policies shall be established, implemented and maintained that meet or exceed the requirements and recommendations of Sections 5.1 and 5.3 of ANSI N18.7-1972 and Appendix "A" of USAEC Regulatory Guide 1.33 except as provided in 6.8.2 and 6.8.3 below.

6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20,203(c)(2) of 10 CFR 20, each high radiation area normally accessible\* by personnel in which the intensity of radiation is greater than 100 mrem/hr\*\* but less than 1000 mrem/hr\*\* shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit in accordance with site-approved procedures. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rates in the area have been established and personnel have been made knowledgeable of them.
- c. An individual qualified in radiation protection, with a radiation dose rate monitoring device, who is responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the unit Radiation Protection Supervisor in the Radiation Work Permit.

6.12.2 In addition to the requirements of 6.12.1 areas accessible to personnel with radiation levels such that a major portion of the body could receive in one hour a dose greater than 1000 mrem\*\* shall be provided with locked doors to prevent unauthorized entry, and the hard keys or access provided by magnetic keycard shall be maintained under the administrative control of the Station Shift Supervisor or designate on duty and/or the unit Radiation Protection Supervisor or designate. Doors shall remain locked except during periods of access by personnel under an approved RWP which shall specify the appropriate dose rate levels in the immediate work area and the maximum allowable stay time for individuals in that area. In lieu of the stay time specification of the RWP, continuous surveillance, direct or remote, such as use of closed circuit TV cameras, may be made by personnel qualified in radiation protection procedures to provide positive exposure control over the activities within the area. For individual areas accessible to personnel with radiation levels such that a major portion of the body could receive in one hour a dose in excess of 1000 mrem\*\* that are located within large areas, such as the drywell, where no enclosure exists for purpose of locking, and no enclosure can be reasonably constructed around the individual areas, then that area shall be roped off, conspicuously posted and a flashing light shall be activated as a warning device.

\* by accessible passage and permanently fixed ladders  
\*\* measurement made at 18" from source of radioactivity



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 56 TO FACILITY OPERATING LICENSE NO. DPR-63  
NIAGARA MOHAWK POWER CORPORATION  
NINE MILE POINT NUCLEAR STATION, UNIT NO. 1  
DOCKET NO. 50-220

Introduction

By letter dated April 18, 1980 superseded by letter dated April 21, 1983 Niagara Mohawk Power Corporation (the licensee) proposed changes to the Technical Specifications (TS) of Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station, Unit No. 1. The revision to the Technical Specifications addressed in this Safety Evaluation regards the following changes to the Administrative Controls (Section 6.0): (1) accommodate an increase and improved staff in the on-site organization, correct title changes in the organization and reflect the strengthening of the Quality Assurance (QA) function by elevation of the Manager of QA to a Vice President reporting directly to the President; (2) include changes for the frequency of audits related to the Safeguards Contingency Plan and Emergency Preparedness program from every two years to annually; (3) change the provisions for entering and controlling entry to high radiation areas by imposing additional conditions similar to those included in current Standard Technical Specifications for BWRs; and (4) correct a typographical error with regard to the provisions for the fire brigade staff.

Evaluation

The licensee has proposed changes to Section 6.0, Administrative Controls of the Technical Specifications to reflect the increased on-site staffing, correct organizational titles for consistency with the increased site staffing, and to reflect the strengthening of the Quality Assurance function. The staff has reviewed the changes and finds them to be acceptable. In particular, the changes made to the QA function should tend to provide an administrative framework for a strong organization. Changes made with regard to the site Operations Review Committee and the Safety Review and Audit Board and associated actions and records are strictly administrative in that they only correct changes in titles.

The licensee has proposed changes to the frequency of audits for the Safeguards Contingency Plan and Emergency Preparedness program. The staff issued Generic Letter 82-23 and Generic Letter 82-17 requesting licensees to eliminate inconsistencies in their Technical Specifications with changes in the Code of Federal Regulations for audits of Safeguards and Emergency Preparedness programs. The changes submitted by the licensee are acceptable in that audit frequencies were changed from every two years to annually meeting the current Regulations.

The licensee has proposed changes to the High Radiation Area part of Section 6.0 of the Administrative Controls section to impose additional conditions for entering and controlling entry into high radiation areas. The licensee in a telecon on January 12, 1984 agreed with the staff to some slight changes with the language of the section to achieve closer conformity with the approved Standard Technical Specifications. In particular, the paragraph 6.12 was changed to remove Extended Radiation Work Permits from the proposed Technical Specification but to allow issuance of a Radiation Work Permit in accordance with site approved procedures. With this change to the proposed Technical Specification for the High Radiation Area, the specification is virtually identical to the Standard Technical Specifications. The changes made did not have any substantive effect regarding the description of the action contained in the Federal Register notice of this action. The staff finds the changes submitted to be acceptable since they are essentially similar to those contained in Standard Technical Specifications which meet the staff criteria for controlling entry and entering into high radiation areas.

The last proposed change is the correction of "four" to "five" on page 245a of the Technical Specification. This changes a typographical error to achieve consistency with the number five contained in current specifications. The staff finds this administrative change acceptable.

#### Environmental Considerations

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact, and pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

#### Conclusion

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Robert A. Hermann

Dated: March 20, 1984

March 20, 1984

MEMORANDUM FOR: James R. Miller, Chief  
Operating Reactors Branch #3  
Division of Licensing

FROM: Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

SUBJECT: REQUEST FOR PUBLICATION IN MONTHLY FR NOTICE - NOTICE  
OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

Niagara Mohawk Power Corporation, Docket No. 50-220, Nine Mile Point  
Nuclear Station, Unit No. 1, Oswego County, New York

Date of application for amendment: April 18, 1980, superseded April 21,  
1983

Brief description of amendment: The amendment revises the Technical Specifications, Administrative Controls (Section 6.0) to: (1) accommodate an increase and improved staff in the on-site organization, correct title changes in the organization and reflect the strengthening of the Quality Assurance (QA) function by elevation of the Manager of QA to a Vice President reporting directly to the President; (2) include changes for the frequency of audits related to the Safeguards Contingency Plan and Emergency Preparedness program from every two years to annually; (3) change the provisions for entering and controlling entry to high radiation areas by imposing additional conditions similar to those included in current Standard Technical Specifications for BWRs; and (4) correct a typographical error with regard to the provisions for the fire brigade staff.

Date of issuance: March 20, 1984

Effective date: March 20, 1984

Amendment No. 56

8404050345 840320  
CF ADCK 05000220  
CF

Facility Operating License No. DPR-63.

Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: August 23, 1983 48 FR 38410

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated March 20, 1984.

No significant hazards consideration comments received: No.

Local Public Document Room location: State University College at Oswego, Penfield Library - Documents, Oswego, New York 13126.

Original signed by/

Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

DISTRIBUTION

Docket File

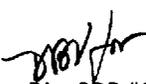
ORB#2 NSHCD File

SNorris

RHermann

JMiller (Orig. + 1)

DL:ORB#2  
SNorris:ajs  
02/23/84

  
DL:ORB#2  
RHermann  
02/28/84

  
DL:ORB#2  
DVassallo  
02/28/84