

April 18, 1984

Mr. G. K. Rhode  
Senior Vice President  
Niagara Mohawk Power Corporation  
300 Erie Boulevard West  
Syracuse, New York 13202

Dear Mr. Rhode:

The Commission has issued the enclosed Amendment No. 57 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your request dated November 4, 1983.

The revision to the Technical Specifications changes the limiting conditions for operation and surveillance requirements for inservice inspection, inservice testing and augmented inservice inspection, as well as the supporting bases. The changes to the Technical Specifications are similar to those contained in Standard Technical Specifications and accommodate the updating provisions of 10 CFR 50.55a(g).

A copy of the Safety Evaluation is also enclosed.

Sincerely,

Original signed by/

Robert A. Hermann, Project Manager  
Operating Reactors Branch #2  
Division of Licensing

Enclosures:

- 1. Amendment No. 57 to License No. DPR-63
- 2. Safety Evaluation

cc w/enclosures:

See next page

DISTRIBUTION

Docket File	RHermann	WJones	Gray File
NRC PDR	OELD	DBrinkman	Extra - 5
Local PDR		ACRS (10)	TWambach
ORB#2 Reading	LJHarmon	OPA, CMiles	SNorris
	ELJordan	RFerguson	TBarnhart (4)
DEisenhut	JMTaylor	RDiggs	NSIC
MEB			
DL:ORB#2	DL:ORB#2	DL:ORB#2	DL:ORB#2
SNorris:pob	RHermann	DVassallo	GLainas
4/3/84	4/4/84	4/12/84	4/17/84
		DL/MEB	OELD
		7	DL:ORB#2
		4/9/84	4/11/84

8405150477 840418  
PDR ADOCK 05000220  
P PDR

Mr. G. K. Rhode  
Niagara Mohawk Power Corporation  
Nine Mile Point Nuclear Station, Unit No. 1

cc:

Troy B. Conner, Jr., Esq.  
Conner & Wetterhahn  
Suite 1050  
1747 Pennsylvania Avenue, N. W.  
Washington, D. C. 20006

Mr. Robert P. Jones, Supervisor  
Town of Scriba  
R. D. #4  
Oswego, New York 13126

Niagara Mohawk Power Corporation  
ATTN: Mr. Thomas Perkins  
Plant Superintendent  
Nine Mile Point Nuclear Station  
Post Office Box 32  
Lycoming, New York 13093

U. S. Environmental Protection  
Agency  
Region II Office  
Regional Radiation Representative  
26 Federal Plaza  
New York, New York 10007

Resident Inspector  
U. S. Nuclear Regulatory Commission  
Post Office Box 126  
Lycoming, New York 13093

John W. Keib, Esquire  
Niagara Mohawk Power Corporation  
300 Erie Boulevard West  
Syracuse, New York 13202

Thomas A. Murley  
Regional Administrator  
Region I Office  
U. S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

Mr. Jay Dunkleberger  
Division of Policy Analysis and Planning  
New York State Energy Office  
Agency Building 2, Empire State Plaza  
Albany, New York 12223



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 57  
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated November 4, 1983 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

8405150479 840418  
PDR ADDCK 05000220  
P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 57, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 18, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 57

FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise the Appendix A Technical Specifications by removing and inserting the following pages:

<u>Existing Page</u>	<u>Revised Page</u>
92-115 258	92-115 258

The revised areas are indicated by marginal lines. Those pages left intentionally blank do not contain marginal markings.

LIMITING CONDITION FOR OPERATION

SURVEILLANCE REQUIREMENT

3.2.6 INSERVICE INSPECTION AND TESTING

Applicability:

Applies to components which are part of the reactor coolant pressure boundary and their supports and other safety-related pressure vessels, piping, pumps, and valves.

Objective:

To assure the integrity of the reactor coolant pressure boundary and the operational readiness of safety-related pressure vessels, piping, pumps, and valves.

Specification:

a. Inservice Inspection

1. To be considered operable, Quality Group A, B and C components shall satisfy the requirements contained in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda for continued service of ASME Code Class 1, 2 and 3 components, respectively, except where relief has been granted by the Commission pursuant to 10CFR50, Section 50.55a(g)(6)(i). (1)

4.2.6 INSERVICE INSPECTION AND TESTING

Applicability:

Applies to the periodic inspection and testing of components which are part of the reactor coolant pressure boundary and their supports and other safety-related pressure vessels, piping, pumps, and valves.

Objective:

To verify the integrity of the reactor coolant pressure boundary and the operational readiness of safety-related pressure vessels, piping, pumps, and valves.

Specification:

a. Inservice Inspection

1. Inservice inspection of Quality Group A, B and C components shall be performed in accordance with the requirements for ASME Code Class 1, 2 and 3 components, respectively, contained in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10CFR50, Section 50.55a(g), except where relief has been granted by the Commission pursuant to 10 CFR Part 50 Section 50.55a(g)(6)(i). (1)

b. Inservice Testing

1. To be considered operable, Quality Group A, B and C pumps and valves shall satisfy the requirements contained in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda for continued service of ASME Code Class 1, 2 and 3 components, respectively, except where relief has been granted by the Commission pursuant to 10CFR50, Section 50.55a(g)(6)(i).(2)

2. An augmented inservice inspection program shall be performed in accordance with the schedules contained in NUREG 0313 Revision 1. The augmented inservice inspection program shall be performed on service sensitive components. The following systems contain service sensitive components: core spray, shutdown cooling, emergency condensers, liquid poison, reactor head spray and control rod drive return.

b. Inservice Testing

1. Inservice testing of Quality Group A, B and C pumps and valves shall be performed in accordance with the requirements for ASME Code Class 1, 2 and 3 components contained in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10CFR50, Section 50.55a(g), except where relief has been granted by the Commission pursuant to 10 CFR Part 50 Section 50.55a(g)(6)(i).(2)

The inservice inspection and testing program for the Nine Mile Point Unit 1 plant conforms to the requirements of 10 CFR 50, Section 50.55a(g). Where practical, the inspection of components, pumps and valves classified into NRC Quality Groups A, B and C conforms to the requirements of ASME Code Class 1, 2 and 3 components, pumps and valves, respectively, contained in Section XI of the ASME Boiler and Pressure Vessel Code. If a Code required inspection is impractical for the Nine Mile Point Unit 1 facility, a request for a deviation from that requirement is submitted to the Commission in accordance with 10 CFR 50, Section 50.55a(g)(6)(i).

Deviations which are needed from the procedures prescribed in Section XI of the ASME Code and applicable Addenda will be reported to the Commission prior to the beginning of each 10-year inspection period if they are known to be required at that time. Deviations which are identified during the course of inspection will be reported quarterly throughout the inspection period.

The augmented inservice inspection program for the Nine Mile Point Unit 1 plant conforms to the schedules contained in NUREG 0313 Revision 1. It is performed in order to detect and survey intergranular stress corrosion cracking of ASME Code Class 1, 2 and 3 pressure boundary piping. Inspections shall be performed by individuals qualified to (1) the ASME Boiler and Pressure Vessel Code, Section XI, as specified to the NRC, and (2) Ultrasonic Testing Operator Training for Intergranular Stress Corrosion Cracking developed by the EPRI Non-Destructive Examination Center, as specified to the NRC.

#### References

- (1) Letter from the Nuclear Regulatory Commission (D. B. Vassallo) to Niagara Mohawk Power Corporation (G. K. Rhode), dated September 19, 1983.
- (2) Letter from Niagara Mohawk Power Corporation (D. P. Dise) to the Nuclear Regulatory Commission (T. A. Ippolito), dated August 7, 1981.

THIS PAGE BLANK

THIS PAGE BLANK

Amendment No. 57

THIS PAGE BLANK

- b. Safety Class 1 Inservice Inspection, Specification 4.2.6 (Three months)
- c. Safety Class 2 Inservice Inspections, Specification 4.2.6 (Three months)
- d. Safety Class 3 Inservice Inspections; Specification 4.2.6 (Three months)
- e. Primary Containment Leakage Testing, Specification 3.3.3 (Three months)
- f. Secondary Containment Leakage Testing, Specification 3.4.1 (Three months)
- g. Sealed Source Leakage In Excess Of Limits, Specification 3.6.5.2 (Three months)

6.10 Record Retention

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. REPORTABLE OCCURRENCE Reports.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 57 TO FACILITY OPERATING LICENSE NO. DPR-63

NIAGARA MOHAWK POWER CORPORATION  
NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-220

1.0 Introduction

By letter dated November 4, 1983 Niagara Mohawk Power Corporation (the licensee) proposed changes to the Technical Specifications (TS) of Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station, Unit No. 1. The revision to the Technical Specifications addressed in this Safety Evaluation regards a change to the limiting conditions for operation, surveillance requirements, and bases pertaining to the inservice inspection, the augmented inservice inspection and inservice testing programs. Requirements for the inservice testing and augmented inspection programs were not formerly included in the Technical Specifications. The proposed Technical Specification changes achieve consistency with 10 CFR 50.55(a)(g) and more closely match the Standard Technical Specifications. The Technical Specification changes are general enough to permit changes in accordance with Section XI of the ASME Boiler and Pressure Vessel Code to meet the updating provisions of 10 CFR 50 without a corresponding Technical Specification change. This submittal does not affect, or change, the present programs being implemented by Niagara Mohawk. Changes to the inservice inspection or inservice testing programs must receive prior approval from the Nuclear Regulatory Commission.

2.0 Evaluation

The licensee has proposed changes to the Technical Specifications for the inservice inspection and testing programs for the Nine Mile Point Nuclear Station, Unit No. 1. The change for the Inservice Inspection Program is necessary to update Technical Specifications with the revised Inservice Inspection Program approved by the staff in a Safety Evaluation report dated September 19, 1983. The change adding the Inservice Testing Program is necessary, since the requirement for inservice testing of components in accordance with the ASME Boiler and Pressure Vessel Code was established by changes in 10 CFR subsequent to the issuance of the Nine Mile Point Nuclear Station, Unit 1 license. The changes to the Technical Specifications for the inservice inspection and testing programs are consistent with NUREG-0123, Standard Technical Specifications for Boiling Water Reactors that is endorsed by Chapter 16 of the Standard Review Plant and are, therefore, acceptable.

8405150480 840418  
PDR ADOCK 05000220  
P PDR

Niagara Mohawk Power Corporation has replaced the recirculation system and portions of other system piping attached to the recirculation system at Nine Mile Point Nuclear Station, Unit No. 1 with "conforming material" as defined in NUREG-0313, Rev. 1. The licensee has proposed an augmented inservice inspection for the balance of systems containing "non-conforming" service sensitive piping identified by NUREG-0313, Rev. 1. The program requires the extent of inspections in accordance with the frequencies contained in the NUREG using procedures that have been shown capable of finding in situ stress corrosion cracking and by individuals qualified to (1) the ASME Boiler and Pressure Code, Section XI and (2) Ultrasonic Testing Operator Training for Intergranular Stress Corrosion Cracking developed by the EPRI Non-destructive Examination Center. We find the proposed augmented inspection program acceptable since the extent of inspection and frequencies specified in NUREG-0313, Rev. 1 are met for the remaining non-conforming service sensitive piping systems.

### 3.0 Environmental Considerations

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact, and pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

### 4.0 Conclusion

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Robert A. Hermann

Dated: April 18, 1984