

April 1, 1985

Docket No. 50-220

Mr. B. G. Hooten  
Executive Director, Nuclear Operations  
Niagara Mohawk Power Corporation  
300 Erie Boulevard West  
Syracuse, New York 13202

Dear Mr. Hooten:

The Commission has issued the enclosed Amendment No. 71 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your request dated May 1, 1984 supplemented and clarified by your letter dated October 22, 1984.

The revision to the Technical Specifications adds limiting conditions for operation, surveillance requirements and bases for the Remote Shutdown Panels.

A copy of the Safety Evaluation is also enclosed.

Sincerely,



Robert A. Hermann, Project Manager  
Operating Reactors Branch #2  
Division of Licensing

Enclosures:

1. Amendment No. 71 to License No. DPR-63
2. Safety Evaluation

cc w/enclosures:  
See next page

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OPA, CMiles  
RDiggs  
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Extra - 5  
JPartlow

DL:ORB#2  
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Mr. B. G. Hooten  
Niagara Mohawk Power Corporation  
Nine Mile Point Nuclear Station, Unit No. 1

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 71  
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated May 1, 1984 as supplemented and clarified October 22, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

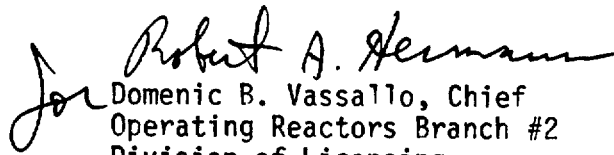
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(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 71, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 1, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 71

FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise the Appendix A Technical Specifications by removing and inserting the following pages:

<u>Existing Page</u>	<u>Revised Page</u>
iiia	iiia
--	241ii4
--	241ii5
--	241ii6
--	241ii7
--	241ii8

The revised areas are indicated by marginal lines.

SECTION	DESCRIPTION	PAGE
3.6.5	Radioactive Material Sources	241k
3.6.6	Fire Detection	241m
3.6.7	Fire Suppression	241q
3.6.8	Carbon Dioxide Suppression System	241u
3.6.9	Fire Hose Stations	241y
3.6.10	Fire Barrier Penetration Fire Seals	241cc
3.6.11	Accident Monitoring Instrumentation	241ee
3.6.12	Reactor Protection System Motor Generator Set Monitoring	241iil
3.6.13	Remote Shutdown Panels	241ii4
3.6.14	Radioactive Effluent Instrumenation	241jj
3.6.15	Radioactive Effluents	241ww
3.6.16	Radioactive Effluent Treatment Systems	241qqq
3.6.17	Explosive Gas Mixture	241ttt
3.6.18	Mark I Containment	241vvv
3.6.19	Liquid Waste Holdup Tanks	241xxx
3.6.20	Radiological Environmental Monitoring Program	241zzz
3.6.21	Interlaboratory Comparison Program	241111
3.6.22	Land Use Census	241nnn

\* to be submitted at a future date

## LIMITING CONDITION FOR OPERATION

### 3.6.13 REMOTE SHUTDOWN PANELS

#### Applicability:

Applies to the operating status of the remote shutdown panels.

#### Objective:

To assure the capability of the remote shutdown panels to provide 1) initiation of the emergency condensers independent of the main/auxiliary control room 2) control of the motor-operated steam supply valves independent of the main/auxiliary control room and 3) parameter monitoring outside the control room.

#### Specification:

- a. During power operation and whenever the reactor coolant temperature is greater than 212°F, at least one remote shutdown panel shall be operable.

## SURVEILLANCE REQUIREMENT

### 4.6.13 REMOTE SHUTDOWN PANELS

#### Applicability:

Applies to the periodic testing requirements for the remote shutdown panels.

#### Objective:

To assure the capability of the remote shutdown panels to provide 1) initiation of the emergency condensers independent of the main/auxiliary control room 2) control of the motor-operated steam supply valves independent of the main/auxiliary control room and 3) parameter monitoring outside the control room.

#### Specification:

The remote shutdown panels surveillance shall be performed as indicated below:

- a. Each remote shutdown panel monitoring instrumentation channel shall be demonstrated operable by performance of the operations and frequencies shown in Table 4.6.13-1.
- b. During each major refueling outage
  1. Each remote shutdown panel shall be demonstrated to initiate the emergency condensers independent of the main/auxiliary control room.

## LIMITING CONDITION FOR OPERATION

### 3.6.13 REMOTE SHUTDOWN PANELS (Continued)

- b. A remote shutdown panel shall be considered inoperable if either the emergency condenser condensate return valve control switch is inoperable, either motor-operated steam supply valve control switch is inoperable, or the number of operable instrumentation channels is less than that required by Table 3.6.13-1.
- c. If Specification 3.6.13.a cannot be met, commence an orderly shutdown within 24 hours and be in cold shutdown within 36 hours.

## SURVEILLANCE REQUIREMENT

### 4.6.13 REMOTE SHUTDOWN PANELS (Continued)

- 2. Each remote shutdown panel shall be demonstrated to open both the motor-operated steam valves.



TABLE 3.6-13-1

REMOTE SHUTDOWN PANEL MONITORINGLimiting Condition for Operation

<u>INSTRUMENT</u>	<u>MINIMUM NUMBER OF OPERABLE CHANNELS</u>
Reactor Pressure	1
Reactor Water Level	1
Reactor Water Temperature	1
Torus Water Temperature	1
Drywell Pressure	1
Emergency Condenser Water Level	1
Drywell Temperature	1
"All Rods In" Light	1

TABLE 4.6.13-1  
REMOTE SHUTDOWN PANEL MONITORING  
Surveillance Requirement

<u>Parameter</u>	<u>Sensor Check</u>	<u>Instrument Channel Calibration</u>
Reactor Pressure	Once per day	Once per 3 months (a)
Reactor Water Level	Once per day	Once per 3 months (a)
Reactor Water Temperature	Once per day	Once per refueling cycle
Torus Water Temperature	Once per day	Once per refueling cycle
Drywell Pressure	Once per day	Once per 3 months (a)
Emergency Condenser Water Level	Once per day	Once per refueling cycle
Drywell Temperature	Once per day	Once per refueling cycle
"All Rods In" Light	Once per refueling cycle	N/A

- (a) The indicator located at the remote shutdown panel will be calibrated at the frequency listed in Table 4.6.13-1. Calibration of the remaining channel instrumentation is provided by Specification 4.6.2.

#### BASES FOR 3.6.13 AND 4.6.13 REMOTE SHUTDOWN PANELS

The remote shutdown panels provide 1) manual initiation of the emergency condensers 2) manual control of the steam supply valves and 3) parameters monitoring independent of the main/auxiliary control room. Two panels are provided, each located in a separate fire area, for added redundancy. Both panels are also in separate fire areas from the main/auxiliary control room. One remote shutdown panel provides the necessary capabilities consistent with 10CFR50 Appendix R. Therefore, only one remote shutdown panel is required to be operable. The electrical design of the panels is such that no single fire can cause loss of both emergency condensers.

Each remote shutdown panel is provided with controls for one emergency condenser loop. The emergency condensers are designed such that automatic initiation is independently assured in the event of a fire 1) in the Reactor Building (principle relay logic located in the auxiliary control room or 2) in the main/auxiliary control room or Turbine Building (redundant relay logic located in the Reactor Building). Each remote shutdown panel also has controls to operate the two motor-operated steam supply valves on its respective emergency condenser loop. A key operated bypass switch is provided to override the automatic isolation signal to these valves. Once the bypass switch is activated, the steam supply valves can be manually controlled from the remote shutdown panels. Since automatic initiation of the emergency condenser is assured, the remote shutdown panels serve as additional manual controlling stations for the emergency condensers. In addition, certain parameters are monitored at each remote shutdown panel.

The remote shutdown panels are normally de-energized, except for the monitoring instrumentation, which is normally energized. To energize the remaining functions on a remote shutdown panel, a power switch located on each panel must be activated. Once the panels are completely energized, the emergency condenser condensate return valve and steam supply valve controls can be utilized.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 71 TO FACILITY OPERATING LICENSE NO. DPR-63

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-220

1.0 Introduction

By application dated May 1, 1984 supplemented and clarified by letter dated October 22, 1984, Niagara Mohawk Power Corporation (the licensee) requested an amendment to Appendix A of Operating License No. DPR-63 for Nine Mile Point Nuclear Station, Unit No. 1. The amendment request involves revising the Technical Specifications to add limiting conditions for operation, surveillance requirements and bases for the Remote Shutdown Panels (RSP).

2.0 Evaluation

In its May 1, 1984 submittal, the licensee proposed limiting conditions for operation (LCOs) and surveillance requirements for the RSPs. Also, the licensee identified those parameters which are monitored on the RSPs and their associated LCOs and surveillance requirements (Tables 3.6-13-1 and 4.6-13-1). The LCO for the RSPs require that at least one shutdown panel be operable at reactor coolant temperature greater than 212°F and during power operation. In addition, the RSP LCO defined the criteria for declaring a panel inoperable and identified the actions to be taken when both RSPs were inoperable. In the case where both panels were inoperable, the licensee was required to submit a special report if at least one panel could not be returned to an operable status within 15 days.

For the monitoring instrumentation, the licensee included all of those parameters identified in its safe shutdown analysis as being needed to achieve hot shutdown, the minimum number of channels required to be operable, the frequency of sensor checks, and channel calibrations. There was no specification for isolation or transfer switches. The reason for this is that the licensee has demonstrated in its safe shutdown analysis that no isolation or transfer switches are required.

The staff reviewed the proposed surveillance requirements for RSP instrumentation at NMP-1. These requirements, sensor checks and instrument channel calibrations, are listed in Table 4.6-13-1 (Remote Shutdown Panel Monitoring Surveillance Requirements). Sensor checks are required once per day (except for the "All Rods In" light which will be checked once per refueling cycle), and instrument channel calibrations will be performed once per 3 months (except for reactor water, torus water, and drywell

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temperatures, and emergency condenser water level, which will be calibrated once per refueling cycle). These surveillance frequencies are at least as conservative as those presented in the Standard Technical Specifications for RSP instrumentation at other BWRs.

The licensee's October 22, 1984 letter revised the actions required when both remote shutdown panels were inoperable. In particular, the new LCO requires that at least one shutdown panel be operable during power operation and whenever reactor coolant temperature is greater than 212°F. If this cannot be met, shutdown will commence within 24 hours and the plant must be in cold shutdown within 36 hours.

Based on its review, the staff has determined that (1) all of the parameters identified in the licensee's safe shutdown analyses have been included in the Technical Specifications, (2) the LCOs and surveillance requirements for the instrument channels are consistent with other Technical Specifications or are within the frequencies recommended by the staff, and (3) the LCOs for the remote shutdown panels ensure operability of the RSPs or require corrective actions in a time frame which is conservative with respect to staff guidance. Therefore, the staff has concluded that Tables 3.6-13-1 and 4.6-13-1 contained in the May 1, 1984 submittal and Specification 3.6-13 and Surveillance Requirement 4.6-13 contained in the October 22, 1984 submittal are acceptable.

### 3.0 Environmental Considerations

This amendment involves a change in the installation or use of a facility component located within the restricted area as well as a change in a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

### 4.0 Conclusion

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: J. Holonich and R. Kendall

Dated: April 1, 1985