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Docket No. 50-220

Mr. Donald P. Dise
Vice President - Engineering
Niagara Mohawk Power Corporation
300 Erie Boulevard West
Syracuse, New York 13202

Dear Mr. Dise:

The Commission has issued the enclosed Amendment No. 38 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station, Unit No. 1. The amendment consists of changes to the Technical Specifications as requested by your letter of October 29, 1980.

The amendment revises the Technical Specifications to permit plant operation for four hours with increased secondary containment allowable leakage. This interim approval allows improvement work on safety related systems.

As agreed to by members of your staff the aforementioned approval pertains only to TMI modifications. Therefore, please be advised that this is a one time approval.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original Signed by
T. A. I[illegible]

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Enclosures:
1. Amendment No. 38 to License No. DPR-63
2. Safety Evaluation
3. Notice

cc w/encls:
See next page

Concurrence as to form of Amdt and F.R. Notice only

OFFICE	DL:ORB#2	DL:ORB#2	DL:ORB#2	DL:OR	OELD	
SURNAME	SNorris	PPolk	TAIppolito	TMNovak	BMBordenick	
DATE	12/4/80	12/4/80	12/4/80	12/5/80	12/11/80	

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300 Erie Boulevard West
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Dear Mr. Dise:

The Commission has issued the enclosed Amendment No. to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station, Unit No. 1. The amendment consists of changes to the Technical Specifications as requested by your letter of October 29, 1980.

The amendment revises the Technical Specifications to permit plant operation for four hours without secondary containment. This interim approval allows improvement work on safety related systems.

As agreed to by members of your staff the aforementioned approval pertains only to TMI modifications. Therefore, please be advised that this is a one time approval.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

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OFFICE	DL:ORB#2	DL:ORB#2	DL:ORB#2	DL:OR	OELD	
SURNAME	SNorris	PPolk	TAIppolito	TMNovak		
DATE	11/2/80	~~11~~/ ~~/80~~ 12/2/80	11/ /80	11/ /80	11/ /80	

cc:
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Mr. Robert P. Jones, Supervisor
Town of Scriba
R. D. #4
Oswego, New York 13126

Niagara Mohawk Power Corporation
ATTN: Mr. Thomas Perkins
Plant Superintendent
Nine Mile Point Plant
300 Erie Boulevard West
Syracuse, New York 13202

Director, Criteria and Standards
Division
Office of Radiation Programs (ANR-460)
U. S. Environmental Protection Agency
Washington, D. C. 20460

U. S. Environmental Protection Agency
Region II Office
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26 Federal Plaza
New York, New York 10007

State University at Oswego
Penfield Library - Documents
Oswego, New York 13126

Resident Inspector
c/o U. S. NRC
P. O. Box 126
Lycoming, New York 13093

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 38
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated October 29, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 38 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 12, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 38

FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise Appendix A as follows:

Remove	Insert
v	v
166	166
168	168
--	168a

LIMITING CONDITIONS FOR OPERATION

3.4.1 LEAKAGE RATE

Applicability:

Applies to the leakage rate of the secondary containment.

Objective:

To specify the requirements necessary to limit exfiltration of fission products released to the secondary containment as a result of an accident.

Specification:

Whenever the reactor is in the refueling or power operating condition, the reactor building leakage rate as determined by Specification 4.4.1 shall not exceed 2000 cfm. If this cannot be met after a routine surveillance check, then the actions listed below shall be taken:

a. Suspend immediately irradiated fuel handling, fuel pool and reactor cavity activities, and irradiated fuel cask handling operations in the reactor building.

b. Restore the reactor building leakage rates to within specified limits within 4 hours or initiate normal orderly shutdown and be in a cold shutdown condition within 10 hours.

SURVEILLANCE REQUIREMENTS

4.4.1 LEAKAGE RATE

Applicability:

Applies to the periodic testing requirements of the secondary containment leakage rate.

Objective:

To assure the capability of the secondary containment to maintain leakage within allowable limits.

Specification:

Once during each operating cycle - isolate the reactor building and start emergency ventilation system fan to demonstrate negative pressure in the building relative to external static pressure. The fan flow rate shall be varied so that the building internal differential pressure is at least as negative as that on Figure 3.4.1 for the wind speed at which the test is conducted. The fan flow rate represents the reactor building leakage referenced to zero mph with building internal pressure at least 0.25 inch of water less than atmospheric pressure. The test shall be done at wind speeds less than 20 miles per hour.

In the answers to Questions II-3 and IV-5 of the Second Supplement and also in the Fifth Supplement,* the relationships among wind speed direction, pressure distribution outside the building, building internal pressure, and reactor building leakage are discussed. The curve of pressure in Figure 3.4.1 represents the wind direction which results in the least building leakage. It is assumed that when the test is performed, the wind direction is that which gives the least leakage.

If the wind direction was not from the direction which gave the least reactor building leakage, building internal pressure would not be as negative as Figure 3.4.1 indicates. Therefore, to reduce pressure, the fan flow rate would have to be increased. This erroneously indicates that reactor building leakage is greater than if wind direction were accounted for. If wind direction were accounted for, another pressure curve could be used which was less negative. This would mean that less fan flow (or measured leakage) would be required to establish building pressure. However, for simplicity it is assumed that the test is conducted during conditions leading to the least leakage while the accident is assumed to occur during conditions leading to the greatest reactor building leakage.

As discussed in the Second Supplement and Fifth Supplement, the pressure for Figure 3.4.1 is independent of the reactor building leakage rate referenced to zero mph wind speed at a negative differential pressure of 0.25 inch of water. Regardless of the leakage rate at these design conditions, the pressure versus wind speed relationship remains unchanged for any given wind direction.

By requiring the reactor building pressure to remain within the limits presented in Figure 3.4.1 and a reactor building leakage rate of less than 2000 cfm, exfiltration would be prevented. This would assure that the leakage from the primary containment is directed through the filter system and discharged from the 350-foot stack. Secondary Containment may be broken at one penetration for up to four hours to allow work to proceed on modifications which will enhance the overall safety of the plant. While the secondary containment is open, administrative controls will be in place to assure integrity can be restored immediately, if necessary. Typically, existing penetrations will be opened in the Reactor Building walls to allow for cable installation for various plant modifications.

Preoperational reactor building capability tests shall be conducted after isolating the reactor building and placing either branch of the emergency ventilation system in operation. The tests shall be performed under a number of different environmental wind conditions, i.e. wind speed and direction.

*FSAR Effective after March 31, 1981

In the answers to Questions II-3 and IV-5 of the Second Supplement and also in the Fifth Supplement,* the relationships among wind speed direction, pressure distribution outside the building, building internal pressure, and reactor building leakage are discussed. The curve of pressure in Figure 3.4.1 represents the wind direction which results in the least building leakage. It is assumed that when the test is performed, the wind direction is that which gives the least leakage.

If the wind direction was not from the direction which gave the least reactor building leakage, building internal pressure would not be as negative as Figure 3.4.1 indicates. Therefore, to reduce pressure, the fan flow rate would have to be increased. This erroneously indicates that reactor building leakage is greater than if wind direction were accounted for. If wind direction were accounted for, another pressure curve could be used which was less negative. This would mean that less fan flow (or measured leakage) would be required to establish building pressure. However, for simplicity it is assumed that the test is conducted during conditions leading to the least leakage while the accident is assumed to occur during conditions leading to the greatest reactor building leakage.

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Preoperational reactor building capability tests shall be conducted after isolating the reactor building and placing either branch of the emergency ventilation system in operation. The tests shall be performed under a number of different environmental wind conditions, i.e. wind speed and direction.

*FSAR Effective until March 31, 1981.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 38 TO FACILITY OPERATING LICENSE NO. DPR-63

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-220

Introduction

By letter dated October 29, 1980 Niagara Mohawk Power Corporation (the licensee) applied for amendment to License No. DPR-63 and the Technical Specifications (Appendix A) for Nine Mile Point Nuclear Station, Unit No. 1. The amendment would authorize plant operation with limiting conditions of operation for the secondary containment integrity revised to agree with the "Standard Technical Specifications for General Electric Boiling Water Reactors." (NUREG 0123, Rev. 3); i.e., without secondary containment, restore secondary containment integrity within 4 hours or be in at least hot shutdown within the next 12 hours and in cold shutdown within the following 24 hours.

Background

By letter dated October 29, 1980 the licensee requested operation of Nine Mile Point, Unit 1, with secondary containment (Reactor Building) leakage in excess of the 2000 CFM Technical Specification Limiting Condition of Operation. This interim relief is being requested to allow TMI related upgrading of several safety related systems.

A portion of the safety systems upgrade work consists of routing new electrical cable through secondary containment. To accomplish this work the existing Reactor Building electrical penetrations must be opened. This results in an inability of the Standby Gas Treatment System to maintain Reactor Building vacuum post accident; i.e., leakage in excess of system exhaust capability (2000 CFM).

The licensee has proposed several mitigating actions during the time upgrade work is in progress:

(1) At any one point in time no more than one electrical penetration will be opened and the time that it is open will be minimized;

(2) Activities which have the potential to result in significant radioactive releases to the secondary containment will be suspended; and

(3) Administrative controls will be in effect which ensure that secondary containment integrity can be restored immediately, if necessary.

Evaluation

The present Nine Mile Point Unit 1 Technical Specifications allow operation for 4 hours if secondary containment integrity exceeds the limiting conditions of operation. If integrity is not restored within this specified period, then an orderly shutdown must be initiated. Continued plant operation for 4 hours is based on the very small likelihood of an accident occurring during such a brief interval.

Since the licensee request does not seek approval for periods of loss of secondary containment integrity for more than 4 hours the Nine Mile Point margins of safety would not be reduced. Moreover, the licensee has proposed three mitigating actions which: (1) limit the duration and degree of loss of secondary containment integrity; (2) preclude potential radioactive release while secondary containment has been opened; and (3) imposes administrative procedures which ensure that secondary containment integrity can be restored rapidly, if necessary. Based on this, as well as the fact that once upgrades have been accomplished, plant safety will actually be enhanced, we conclude that the requested upgrade work will not endanger public health and safety and is, therefore, allowable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable

assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: December 12, 1980

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-220

NIAGARA MOHAWK POWER CORPORATION

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 38 to Facility Operating License No. DPR-63 to Niagara Mohawk Power Corporation (the licensee) which revised the Technical Specifications for operation of the Nine Mile Point Nuclear Station, Unit No. 1 (the facility) located in Oswego County, New York. The amendment is effective as of its date of issuance.

The amendment revises the Technical Specifications to permit plant operation for four hours with increased secondary containment allowable leakage. This interim approval allows improvement work on safety related systems.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated October 29, 1980, (2) Amendment No. 38 to License No. DPR-63, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D. C. and at the Oswego County Office Building, 46 E. Bridge Street, Oswego, New York 13126. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland this 12th day of December, 1980.

FOR THE NUCLEAR REGULATORY COMMISSION

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing