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Docket No. 50-220

Niagara Mohawk Power Corporation
ATTN: Mr. Gerald K. Rhode
Vice President - Engineering
300 Erie Boulevard West
Syracuse, New York 13202

Gentlemen:

The Commission has issued the enclosed Amendment No. 15 to Facility License No. DPR-63 for Unit No. 1 of the Nine Mile Point Nuclear Station. This amendment consists of changes to the Technical Specifications and is in response to your request dated May 6, 1977.

The amendment will modify the Technical Specifications to permit the reactor vessel water level to be below the low-low-low level set point for an interval not to exceed twelve weeks.

Copies of the related Safety Evaluation and the FEDERAL REGISTER Notice also are enclosed.

Sincerely,

Original signed by

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:

1. Amendment No. 15 to License DPR-63
2. Safety Evaluation
3. FEDERAL REGISTER Notice

cc w/enclosures:
See next page

+ clarification of 2d new paragraph p. 2 per Nowicki

CONCURRENCE PARAGRAPH CHANGES 5 DOWN ON EVALUATION COPY OF ARE MADE AS AGREED WITH LPM

010990

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|---------|----------|----------|---------|---------|--|
| OFFICE | ORB #3 | ORB #3 | OELD | ORB #3 | |
| SURNAME | CParrish | SNowicki | Barnes | GLear | |
| DATE | 5/13/77 | 5/12/77 | 5/13/77 | 5/13/77 | |



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

May 13, 1977

Docket No. 50-220

Niagara Mohawk Power Corporation
ATTN: Mr. Gerald K. Rhode
Vice President - Engineering
300 Erie Boulevard West
Syracuse, New York 13202

Gentlemen:

The Commission has issued the enclosed Amendment No. 15 to Facility License No. DPR-63 for Unit No. 1 of the Nine Mile Point Nuclear Station. This amendment consists of changes to the Technical Specifications and is in response to your request dated May 6, 1977.

The amendment will modify the Technical Specifications to permit the reactor vessel water level to be below the low-low-low level set point for an interval not to exceed twelve weeks.

Copies of the related Safety Evaluation and the FEDERAL REGISTER Notice also are enclosed.

Sincerely,

A handwritten signature in cursive script that reads "George Lear".

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:

1. Amendment No. 15 to License DPR-63
2. Safety Evaluation
3. FEDERAL REGISTER Notice

cc w/enclosures:
See next page

Niagara Mohawk Power Corporation - 2 -

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Niagara Mohawk Power Corporation
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20545

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 15
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated May 6, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 15, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 13, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 15

FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

6
53a

Replace

6
53a

SAFETY LIMIT

- c. The neutron flux shall not exceed its scram setting for longer than 1.5 seconds as indicated by the process computer. When the process computer is out of service, a safety limit violation shall be assumed if the neutron flux exceeds the scram setting and control rod scram does not occur.

To ensure that the Safety Limit established in Specifications 2.1.1a and 2.1.1b is not exceeded, each required scram shall be initiated by its expected scram signal. The Safety Limit shall be assumed to be exceeded when scram is accomplished by a means other than the expected scram signal.

- d. Whenever the reactor is in the shutdown condition with irradiated fuel in the reactor vessel, the water level shall not be more than 7 feet 11 inches (127.1 inches indicator scale) below minimum normal water level (Elevation 302'9"), except as specified in "e" below.
- e. For the purpose of performing major maintenance (not to exceed 12 weeks in duration) on the reactor vessel; the reactor water level may be lowered 9' below the minimum normal water level (Elevation 302'9"). Whenever the reactor water level is to be lowered below the low-low-low level set point redundant instrumentation will be provided to monitor the reactor water level.

LIMITING SAFETY SYSTEM SETTING

- d. The reactor water low level scram trip setting shall be no lower than -12 inches (53 inches indicator scale) relative to the minimum normal water level (302'9").
- e. The reactor water low-low level setting for core spray initiation shall be no less than -5 feet (5 inches indicator scale) relative to the minimum normal water level (Elevation 302'9").
- f. The flow biased APRM rod block trip settings shall be less than or equal to that shown in Figure 2.1.1.

LIMITING CONDITION FOR OPERATION

SURVEILLANCE REQUIREMENTS

- h. For the purpose of performing major maintenance (not to exceed 12 weeks in duration) on the reactor vessel, the reactor water level may be lowered to 9' below the minimum normal water level (elevation 302'9"). Whenever the reactor water level is to be lowered below the low-low level set point redundant instrumentation will be provided to monitor the reactor water level and written procedures will be developed and followed whenever the reactor water level is lowered below the low-low level set point. The procedures will define the valves that will be used to lower the vessel water level. All other valves that have the potential of lowering the vessel water level will be identified by valve number in the procedures and these valves will be red tagged to preclude their operation during the major maintenance with the water level below the low-low level set point.

During the period of major maintenance requiring lowering the water level to more than 7 feet 11 inches below minimum normal water level (127.1 inches indicator scale), either both Core Spray Systems must be operable or, if one Core Spray System is inoperable because of the maintenance, all of the redundant components of the other Core Spray System must be operable.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20545

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 15 TO FACILITY OPERATING LICENSE NO. DPR-13

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT UNIT NO. 1

DOCKET NO. 50-220

Introduction

By letter dated May 6, 1977, Niagara Mohawk Power Corporation (NMPC) requested an amendment to Facility Operating License No. DPR-63 for the Nine Mile Point Unit No. 1. The amendment would modify the Technical Specifications to permit the reactor vessel water level to be below the low-low-low level set point for an interval not to exceed twelve weeks. Amendment No. 14, issued March 25, 1977, permitted NMPC to lower the reactor vessel water level below the low-low-low level set point during major maintenance for an interval not to exceed six weeks.

Discussion and Evaluation

The licensee had agreed to perform the Feedwater Nozzle Inspection and repair program during the current refueling outage. The Nine Mile Point Unit No. 1 inspection is consistent with the generic concern of the staff for Feedwater Nozzle Inspection to be performed at all BWR facilities. The purpose of the Feedwater Nozzle Inspection is to assure that the integrity of the feedwater nozzles has not been compromised, with the major objective of enhancing safe operation of the plant. Since the facility does not currently have the capacity to fully unload the core, the inspection is being performed with the fuel in the reactor vessel. In order to be able to perform the inspection and maintenance program, the water level must be lowered approximately one foot more than was specified in the Technical Specifications. The staff has previously evaluated the procedures and requirements associated with the modification of Technical Specifications and found them acceptable; as a result, Amendment No. 14 to Facility License No. DPR-63 was issued on March 25, 1977.

The maintenance work on the spargers and the nozzles has taken more time than anticipated by the licensee. The new machine developed to perform the nozzle boring has required additional fine-tuning. Also, the results of inspection of the Control Rod Drive (CRD) return line nozzle have

indicated that more extensive maintenance is required than anticipated or scheduled. The licensee has decided to continue the maintenance program to bring the vessel into the safest condition possible. However, in order to bring the maintenance program to a successful completion, the licensee is required to amend the Technical Specification to extend the period of authorization for lowering the reactor water level below the low-low-low level set point. The additional time needed is six weeks.

The staff has previously reviewed: the instrumentation being used for water level monitoring during the maintenance, the procedures for lowering the water level, and the equipment and procedures available to replace lost water in the reactor vessel. The procedures and equipment used to move maintenance equipment into and out of the reactor vessel were also reviewed. All of the above were found acceptable in our Safety Evaluation issued in connection with Amendment No. 14.

Even though the time limit proposed doubles (from six to a total twelve weeks) the time interval at which the vessel water level is in the new low condition, the probability of inadvertently lowering the water level below the new low condition is only minimally increased. Only those valves required to initially lower the vessel water level are permitted to be manipulated during the maintenance operation and operation of these valves is carefully monitored and procedurally controlled. Since the water level was adjusted at the beginning of the maintenance program, this critical period of the operation has been completed.

In view of the foregoing, the safety evaluation performed for Amendment No. 14 remains valid. Since the Feedwater Nozzle and CRD return line nozzle maintenance program is consistent with the major objective of enhancing safe operation of the plant, we find this proposed change acceptable.

Environmental Considerations

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusions

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does

not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: May 13, 1977

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-220

NIAGARA MOHAWK POWER CORPORATION

NOTICE OF ISSUANCE OF FACILITY LICENSE AMENDMENT

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 15 to Facility Operating License No. DPR-63 to the Niagara Mohawk Power Corporation (the licensee) which revised Technical Specifications for operation of the Nine Mile Point Nuclear Station, Unit No. 1 (the facility) located in Oswego County, New York. The amendment is effective as of its date of issuance.

The amendment consists of changes to the Technical Specifications to permit the reactor vessel water level to be below the low-low-low level set point for an interval not to exceed twelve weeks. Amendment No. 14, issued March 25, 1977, permitted MNPC to lower the reactor vessel water level below the low-low-low level set point during major maintenance for an interval not to exceed six weeks.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to

10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated May 6, 1977, (2) Amendment No. 15 to License No. DPR-63, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Oswego City Library, 120 E. Second Street, Oswego, New York 13126.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 13 day of May 1977.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors