

May 8, 1984

Docket No. 50-220

Mr. G. K. Rhode  
Senior Vice President  
Niagara Mohawk Power Corporation  
300 Erie Boulevard West  
Syracuse, New York 13202

Dear Mr. Rhode:

The Commission has issued the enclosed Amendment No. 61 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station, Unit No. 1. The amendment changes the Technical Specifications in response to your request dated January 30, 1984.

The amendment revises the Technical Specifications to add calibration requirements for the Main Steam Isolation and Turbine Stop Valve closure automatic scram instrumentation at the facility.

A copy of the Safety Evaluation is also enclosed.

Sincerely,

Original signed by/

Robert A. Hermann, Project Manager  
Operating Reactors Branch #2  
Division of Licensing

Enclosures:

- 1. Amendment No. 61 to License No. DPR-63
- 2. Safety Evaluation

cc w/enclosures:

See next page

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DVassallo  
04/24/84

OELD  
04/23/84

DL:AD-OR  
GLaitas  
04/17/84

Mr. G. K. Rhode  
Niagara Mohawk Power Corporation  
Nine Mile Point Nuclear Station, Unit No. 1

cc:

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 61  
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated January 30, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

8405210483 840508  
PDR ADOCK 05000220  
P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 61, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: May 8, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 61

FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise the Appendix A Technical Specifications by removing and inserting the following pages:

<u>Existing Page</u>	<u>Revised Page</u>
194	194
195	195

The revised areas are indicated by marginal lines.

TABLE 4.6.2a

INSTRUMENTATION THAT INITIATES SCRAMSurveillance Requirement

<u>Parameter</u>	<u>Sensor Check</u>	<u>Instrument Channel Test</u>	<u>Instrument Channel Calibration</u>
(1) Manual Scram	None	Once per 3 months	None
(2) High Reactor Pressure	None	Once per month <sup>(1)</sup>	Once per 3 months <sup>(1)</sup>
(3) High Drywell Pressure	None	Once per month <sup>(1)</sup>	Once per 3 months <sup>(1)</sup>
(4) Low Reactor Water Level	Once/day	Once per month <sup>(1)</sup>	Once per 3 months <sup>(1)</sup>
(5) High Water Level Scram Discharge Volume	None	Once per month	Once per 3 months
(6) Main-Steam-Line Isolation Valve Position	None	Once per 3 months	Once per operating cycle
(7) High Radiation Main-Steam Line	Once/shift	Once per week	Once per 3 months

TABLE 4.6.2a (Cont'd)  
INSTRUMENTATION THAT INITIATES SCRAM  
Surveillance Requirement

<u>Parameter</u>	<u>Sensor Check</u>	<u>Instrument Channel Test</u>	<u>Instrument Channel Calibration</u>
(8) Shutdown Position of Reactor Mode Switch	None	Once during each major refueling outage	None
(9) Neutron Flux			
(a) IRM			
(i) Upscale	(f)	(f)	(f)
(ii) Inoperative	(f)	(f)	(f)
(b) APRM			
(i) Upscale	None	Once/week	Once/week
(ii) Inoperative	None	Once/week	Once/week
(iii) Downscale	None	Once/week	Once/week
(10) Turbine Stop Valve Closure	None	Once per 3 months	Once per operating cycle
(11) Generator Load Rejection	None	Once per month	Once per 3 months



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 61 TO FACILITY OPERATING LICENSE NO. DPR-63  
NIAGARA MOHAWK POWER CORPORATION  
NINE MILE POINT NUCLEAR STATION, UNIT NO. 1  
DOCKET NO. 50-220

Introduction

By application dated January 30, 1984 (Reference 1) Niagara Mohawk Power Corporation (NMP) (the licensee) requested an amendment to Appendix A of Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station, Unit No. 1. The amendment request involves addition of Reactor Protection System instrumentation surveillance requirements.

Evaluation

The licensee's proposal incorporates requirements for periodic calibration of the Main Steam Line Isolation Valve Position and Turbine Stop Valve closure automatic reactor scram instrumentation channels, where presently there are no requirements. The proposal to calibrate these instrument channels once per operating cycle has been evaluated, and the NRC staff has verified that this change is consistent with the requirements of NUREG-0800 (Standard Review Plan) Chapter 16, and NUREG-0123, Rev. 3 (G.E. Standard Technical Specifications). On this basis, we find the proposed change acceptable.

Environmental Considerations

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact, and pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.



Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: William Lazarus

Dated: May 8, 1984