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 TBAbernathy
 JRBuchanan

Docket No. 50-220

Niagara Mohawk Power Corporation
 ATTN: Mr. Gerald K. Rhode
 Vice President - Engineering
 300 Erie Boulevard West
 Syracuse, New York 13202

Gentlemen:

The Commission has issued the enclosed Amendment No. 19 to Facility License No. DPR-63 for Unit No. 1 of the Nine Mile Point Nuclear Station. This amendment consists of changes to the Technical Specifications and is in response to your request dated September 16, 1977.

The amendment modifies the Technical Specifications for the Nine Mile Point Unit No. 1 to extend the operation of the facility with one isolation condenser continuously inoperable until the end of Cycle 5.

Copies of the related Safety Evaluation and the FEDERAL REGISTER Notice are also enclosed.

Sincerely,

Original signed by

George Lear, Chief
 Operating Reactors Branch #3
 Division of Operating Reactors

Enclosures:

1. Amendment No. 19 to License DPR-63
2. Safety Evaluation
3. FEDERAL REGISTER Notice

cc w/enclosures:
 See page 2

*Suggest to make any
 pencil changes noted
 on working copy of Evaluation*

OFFICE →	ORB #3	ORB #3	OELD	ORB #3	
SURNAME →	CParrish	SNowicki/mjf	B... ..	GLear	
DATE →	10/ /77	10/13/77	10/13/77	10/14/77	

Niagara Mohawk Power Corporation - 2 -

cc: Eugene B. Thomas, Jr., Esquire
LeBoeuf, Lamb, Leiby & MacRae
1757 N Street, N. W.
Washington, D. C. 20036

Anthony Z. Roisman, Esquire
Sheldon, Harmon and Roisman
1025 15th Street, N. W.
5th Floor
Washington, D. C. 20005

T. K. DeBoer, Director
Technological Development Programs
State of New York
Energy Office
Swan Street Building
CORE 1 - Second Floor
Empire State Plaza
Albany, New York 12223

Mr. Robert P. Jones, Supervisor
Town of Scriba
R. D. #4
Oswego, New York 13126

Niagara Mohawk Power Corporation
ATTN: Mr. Thomas Perkins
Plant Superintendent
Nine Mile Point Plant
300 Erie Boulevard West
Syracuse, New York 13202

Dr. Neill Thomasson (AW-459)
Chief, Energy Systems Analysis Branch
Office of Radiation Programs
Environmental Protection Agency
401 H Street S. W.
Washington, D. C. 20460

Paul A. Giardina
Regional Radiation Representative
U.S. Environmental Protection Agency
26 Federal Plaza
New York, New York 10007

Oswego County Office Building
46 E. Bridge Street
Oswego, New York 13126



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 19
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated September 16, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

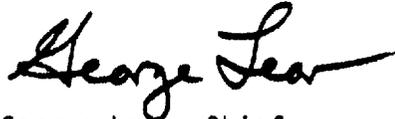
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 19, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 14, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 19

FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains vertical lines indicating the area of change.

Remove

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Replace

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LIMITING CONDITION FOR OPERATION

3.1.3 EMERGENCY COOLING SYSTEM

Applicability:

Applies to the operating status of the emergency cooling system.

Objective:

To assure the capability of the emergency cooling system to cool the reactor coolant in the event the normal reactor heat sink is not available.

Specification:

- a. During power operating conditions and whenever the reactor coolant temperature is greater than 212F, both emergency cooling system shall be operable except as specified in 3.1.3.b and c.
- b. During Cycle 5, with one emergency cooling system inoperable, specification 3.1.3.a shall be considered fulfilled, provided that the additional surveillance required is performed.

SURVEILLANCE REQUIREMENT

4.1.3 EMERGENCY COOLING SYSTEM

Applicability:

Applies to periodic testing requirements for the emergency cooling system.

Objective:

To assure the capability of the emergency cooling system for cooling of the reactor coolant.

Specification:

The emergency cooling system surveillance shall be performed as indicated below:

- a. At least once every five years -
The system heat removal capability shall be determined.
- b. At least once daily -
The shell side water level and makeup tank water level shall be checked.
- c. At least once per month -
The makeup tank level control valve shall be manually opened and closed.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 19 TO FACILITY OPERATING LICENSE NO. DPR-63
NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT UNIT NO. 1
DOCKET NO. 50-220

Introduction

By letter dated September 16, 1977, Niagara Mohawk Power Corporation (NMPC) requested an amendment to Facility Operating License No. DPR-63. The amendment would modify the Technical Specifications for the Nine Mile Point Unit No. 1 (NMP-1) to permit operation of the facility with one isolation condenser continuously inoperable until the end of Cycle 5.

Background

By letter dated July 15, 1977, the NRC issued License Amendment No. 18 for NMP-1 to allow operation with one isolation condenser inoperable provided that the inoperable system is returned to an operable condition at the first cold shutdown after August 11, 1977.

NMPC has submitted the new application in order to remove the requirement to return the inoperable system to an operable condition prior to end of Cycle 5. Cycle 5 operation began on July 10, 1977 and is anticipated to end approximately January 1979.

The Safety Evaluation of License Amendment No. 18 considered: (1) the effect on Emergency Core Cooling Systems (ECCS) following a postulated loss-of-coolant accident (LOCA), (2) the effect on transient analyses, and (3) the effect on reactor shutdown, both normal and with the primary system isolated from the main condenser.

The conclusions reached for the above considerations were: (1) the LOCA analyses submitted by the licensee for the current cycle are acceptable for operation with one isolation condenser system inoperable, (2) the transient results remain applicable with one isolation condenser continuously inoperable, since credit was taken for only one emergency cooling system in these analyses, and (3) plant shutdown both with and without isolation of the primary system from the main condenser is acceptable with an isolation condenser system out of service.

Evaluation

NMPC proposes to repair the inoperable isolation condenser system during the refueling outage at end of Cycle 5. During the refueling outage NMPC plans to remove all of the fuel from the reactor core and place the fuel in the spent fuel pool during maintenance. At present NMP-1 does not have sufficient spent fuel storage capacity to off-load the core. At the end of Cycle 5 the spent fuel pool will be modified to allow complete off-loading. In addition, MNPC's proposal to repair the inoperable isolation condenser during the next refueling outage will have the added advantage of keeping annual occupational exposures as low as possible.

Although using a freeze plug in the piping in order to isolate the isolation condenser during repairs has been a successful and safe technique at NMP-1, NMPC prefers to increase further the safety of this maintenance activity by off-loading the core for this repair. This off-loading removes any probability, however small, of uncovering the core.

Since the issuance of Amendment 18, NMPC has evaluated the transient which has the greatest potential for core uncovering using the staff approved SAFE code. The resulting transient (the sequence of events is: loss of feedwater flow, scram, reactor isolation, and turbine trip) reduces the water level to approximately 8 feet above the active fuel and releases 15×10^6 BTU through the relief valves into the suppression pool. Assuming the initial pool water temperature is at the highest operating temperature allowed by the Technical Specifications, the resulting temperature would be 96°F, which is still 14°F below the 110°F limit requiring reactor scram.

Considering that: (1) previous accident and abnormal operating transient analyses remain applicable, (2) shutdown both normal and with the primary isolated from the main condenser is acceptable, and (3) repair of the isolation condenser system with the core off-loaded removes any probability, no matter how remote, of core uncovering during repair, we conclude that operation with one isolation condenser system inoperable for the remainder of Cycle 5 is acceptable. However, with one isolation condenser system inoperable, some abnormal transients may release additional energy to the suppression pool; therefore, we require that the inoperable isolation condenser system be repaired at end of Cycle 5.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: October 14, 1977

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-220

NIAGARA MOHAWK POWER CORPORATION

NOTICE OF ISSUANCE OF FACILITY LICENSE AMENDMENT

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 19 to Facility Operating License No. DPR-63 to the Niagara Mohawk Power Corporation (the licensee) which revised Technical Specifications for operation of the Nine Mile Point Nuclear Station, Unit No. 1 (the facility) located in Oswego County, New York. The amendment is effective as of its date of issuance.

The amendment modifies the Technical Specifications to permit operation of the facility with one isolation condenser continuously inoperable until the end of Cycle 5.

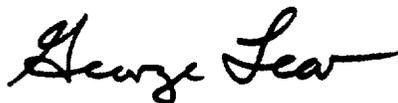
The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated September 16, 1977, (2) Amendment No. 19 to License No. DPR-63, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Oswego County Office Building, 46 E. Bridge Street, Oswego, New York 13126. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 14th day of October 1977.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors