

Docket File

DISTRIBUTION:

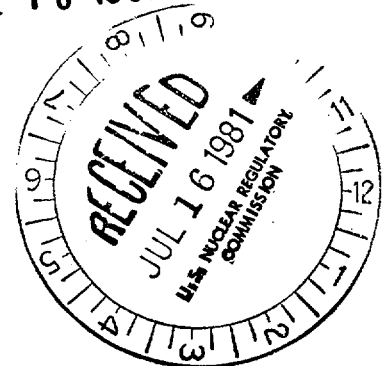
Docket File
NRG PDR
L PDR
TERA

NSIC
ORB#4 Rdg
DEisenhut
Jolshinski
SNorris
PPolk
OELD
IE-3
GDeegan-4

ACRS-10
CMiles
SECY
RDiggs
AEOD
Gray File +4
GLainas
DVerrelli

Docket No. 50-220

JUL 10 1981



Mr. Donald P. Dise
Vice President - Engineering
Niagara Mohawk Power Corporation
300 Erie Boulevard, West
Syracuse, New York 13202

Dear Mr. Dise:

The Commission has issued the enclosed Order confirming your commitments for TMI related requirements for the Nine Mile Point Nuclear Station, Unit 1. This Order is based on your letter dated December 31, 1980, to implement those requirements set forth in NUREG-0737 for which the staff requested completion before June 30, 1981. Explicit statements of these requirements are set forth in the Attachment to the Order.

It is the Commission's intent in issuing this Order to encourage the completion of items consistent with the staff's recommended schedule. However, certain of your proposed schedule exceptions are acceptable. Therefore, you should proceed with implementation of these items in accordance with your proposed schedule. Your proposed technical and schedule exceptions for other items contained in NUREG-0737 will be the subject of future correspondence.

A copy of this Order is being filed with the Office of the Federal Register for publication.

Sincerely,

ORIGINAL SIGNED BY

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Enclosure:
Order

cc w/enclosure:
See next page

AUG 5 1981

8107290135 810710
PDR ADDCK 05000220
P PDR

| | | | | | | | |
|---------|------------|----------|------------|----------|----------|----------|-----------|
| OFFICE | ORB#2:DL | ORB#2:DL | C-ORB#2:DL | AD-OR:DL | AD-OR:DL | OELD | D:DL |
| SURNAME | SNorris/cb | POLK | Ippolito | Troyak | GLainas | P. Humay | DEisenhut |
| DATE | 4/7/81 | 4/8/81 | 4/8/81 | 4/8/81 | 4/8/81 | 4/8/81 | 4/6/81 |

Mr. Donald P. Dise
Niagara Mohawk Power Corporation

cc:
Eugene B. Thomas, Jr., Esquire
LeBoeuf, Lamb, Leiby & MacRae
1333 New Hampshire Avenue, N. W.
Suite 1100
Washington, D. C. 20036

T. K. BeBoer, Director
Technological Development Programs
State of New York
Energy Office
Swan Street Building
CORE 1 - Second Floor
Empire State Plaza
Albany, New York 12223

Mr. Robert P. Jones, Supervisor
Town of Scriba
R. D. #4
Oswego, New York 13126

Niagara Mohawk Power Corporation
ATTN: Mr. Thomas Perkins
Plant Superintendent
Nine Mile Point Plant
300 Erie Boulevard West
Syracuse, New York 13202

U.S. Environmental Protection Agency
Region II Office
ATTN: EIS COORDINATOR
26 Federal Plaza
New York, New York 10007

State University at Oswego
Penfield Library - Documents
Oswego, New York 13126

Resident Inspector
c/o U.S. NRC
P. O. Box 126
Lycoming, New York 13093

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

| | | |
|-----------------------------------|---|-------------------|
| In the Matter of |) | Docket No. 50-220 |
| |) | |
| NIAGARA MOHAWK POWER CORPORATION |) | |
| |) | |
| (Nine Mile Point Nuclear Station, |) | |
| Unit 1) |) | |
| |) | |

ORDER CONFIRMING LICENSEE COMMITMENTS
ON POST-TMI RELATED ISSUES

Niagara Mohawk Power Corporation (the licensee) is the holder of Facility Operating License No. DPR-63, which authorizes the operation of the Nine Mile Point Nuclear Station, Unit 1 (the facility) at steady-state reactor power levels not in excess of 1850 megawatts thermal. The facility is a boiling water reactor located at the licensee's site in Oswego County, New York.

II.

Following the accident at Three Mile Island Unit No. 2 (TMI-2) on March 28, 1979, the Nuclear Regulatory Commission (NRC) staff developed a number of proposed requirements to be implemented on operating reactors and on plants under construction. These requirements include Operational Safety, Siting and Design, and Emergency Preparedness and are intended to provide substantial additional protection for the operation of nuclear facilities

based on the experience from the accident at TMI-2 and the official studies and investigations of the accident. The staff's proposed requirements and schedule for implementation are set forth in NUREG-0737, "Clarification of TMI Action Plan Requirements." Among these requirements are a number of items, consisting of hardware modifications, administrative procedure implementation and specific information to be submitted by the licensee, scheduled to be completed on or before June 30, 1981 (see the Attachment to this Order). NUREG-0737 was transmitted to each licensee and applicant by an NRC letter from my office dated October 31, 1980, which is hereby incorporated by reference. In that letter, it was indicated that although the NRC staff expected each requirement to be implemented in accordance with the schedule set forth in NUREG-0737, the staff would consider licensee requests for relief from staff proposed requirements and their associated implementation dates.

III.

The licensee's submittal dated December 31, 1980 and the references stated therein, which are incorporated herein by reference, committed to complete each of the actions specified in the Attachment. The licensee's submittal included a modified schedule for submittal of certain information. The staff has reviewed the licensee's submittal and determined that the licensee's modified schedule is acceptable based on the following:

The licensee's schedule for submittal of information in some instances does not meet the staff's specified submittal dates. Most of the information requested by the staff describes how the licensee is meeting the guidance of NUREG-0737. Therefore, this deferral of the licensee submittal will not alter the implementation of plant modifications. Therefore, plant safety is not affected by this modification in schedule for the submittal of information.

I have determined that these commitments are required in the interest of public health and safety, and therefore, should be confirmed by ORDER.

IV.

Accordingly, pursuant to Sections 103, 161i, 161o, and 182 of the Atomic Energy Act of 1954, as amended, and the Commission's regulations in 10 CFR Parts 2 and 50, IT IS HEREBY ORDERED EFFECTIVE IMMEDIATELY THAT the licensee shall comply with the following conditions:

The licensee shall satisfy the specific requirements described in the Attachment to this Order (as appropriate to the licensee's facility) as early as practicable but no later than 30 days after the effective date of the ORDER.

V.

Any person who has an interest affected by this Order may request a hearing within 20 days of the date of publication of this Order in the Federal Register. Any request for a hearing shall be addressed to the Director, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555. A copy shall also be sent to the Executive Legal Director at the same address. If a hearing is requested by a person other than the licensee, that person shall describe, in accordance with 10 CFR 2.714(a)(2), the nature of the person's interest and the manner in which the interest is affected by this Order. A REQUEST FOR HEARING SHALL NOT STAY THE IMMEDIATE EFFECTIVENESS OF THIS ORDER.

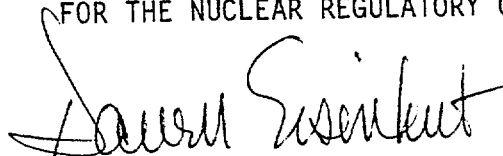
If a hearing is requested by the licensee or other persons who have an interest affected by this Order, the Commission will issue an Order designating the time and place of any such hearing.

If a hearing is held concerning this Order, the issue to be considered at the hearing shall be whether, on the basis of the information set forth in Sections II and III of this Order, the licensee should comply with the conditions set forth in Section IV of this Order.

This request for information was approved by OMB under clearance number 3150-0065 which expires June 30, 1983. Comments on burden and duplication may be directed to the Office of Management and Budget, Reports Management, Room 3208, New Executive Office Building, Washington, D. C.

This Order is effective upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in dark ink, appearing to read "Darrell G. Eisenhut", is written over the typed name.

Darrell G. Eisenhut, Director
Division of Licensing
Office of Nuclear Reactor Regulation

Dated at Bethesda, Maryland
this 10 day of July ,
1981

Attachment:
NUREG-0737 Requirements

NUREG 0737 REQUIREMENTS

| <u>Item</u> | <u>Title</u> | <u>Applicability</u> | <u>Specific Requirement</u> | <u>Impl/Doc Subm Due Dates</u> |
|-------------|---|----------------------|---|------------------------------------|
| I.A.1.1 | STA | ALL | (a) Provide STA coverage by degreed individuals (or equivalent), trained to licensee's program. | 1/1/81 |
| | | | (b) Submit description of current training program and demonstrate compliance with 10-30-79 letter. | 1/1/81 |
| | | | (c) Submit description of long term training program. | 1/1/81 |
| I.A.2.1 | Reactor Operator Qualification | ALL | Submit description of and implement Upgraded Training Program. | 8/1/80 |
| I.C.1 | Accident Procedures | ALL | (a) Submit reanalyses of inadequate core cooling and propose guidelines for emergency procedures, OR propose schedule and justify delays. | 1/1/81 |
| | | | (b) Submit reanalysis of transients and accidents for emergency procedures, OR propose schedule and justify delays. | 1/1/81 |
| I.C.5 | Feedback of Operating Experience | ALL | Implement procedures for feedback of operating experience (no documentation submittal is required). | 1/1/81 |
| I.C.6 | Correct Performance or Operating Activities | ALL | Implement procedures to verify correct performance of operating activities (no documentation submittal is required). | 1/1/81 |

| <u>Item</u> | <u>Title</u> | <u>Applicability</u> | <u>Specific Requirement</u> | <u>Impl/Doc Subm Due Dates</u> |
|-------------|----------------------------------|----------------------|--|------------------------------------|
| II.B.2 | Shielding | ALL | (a) Have available design details for vital area modifications. | 1/1/81 |
| | | | (b) Submit technical deviations to staff positions. | 1/1/81 |
| II.B.3 | Post-Accident Sampling | ALL | Submit description of deviations from staff positions. | 1/1/81 |
| II.B.4 | Training to Mitigate Core Damage | ALL | (a) Have available for review a training program for mitigating core damage (no documentation submittal required) | 1/1/81 |
| | | | (b) Implement training program (no documentation submittal required). | 4/1/81 |
| II.D.1 | Performance Testing of RV/SRV's | ALL | (a) Submit test program (both BWR/PWR's). | 7/1/80 (PWRs) 10/1/80 (BWRs) |
| | | | (b) Submit qualification program for PWR's block valve. | 1/1/81 (PWRs) |
| II.E.1.2 | Aux Feed Initiation and Flow | PWRs | (a) Submit final design and documentation on safety grade flow indication. | 1/1/81 |
| | | | (b) Submit final design and documentation on safety grade flow initiation. | 1/1/81 |
| II.E.4.2 | Containment Isolation | ALL | (a) Submit documentation justifying minimum containment pressure setpoint for isolation of non-essential penetrations. | 1/1/81 |
| | | | (b) Submit statement that purge valves not meeting CSB 6-4 (or interim position) are sealed and verification is performed every 31 days. | 1/1/81 |

| Item | Title | Applicability | Specific Requirement | Impl/Doc Subm Due Dates |
|-----------|-----------------------------------|--------------------------|--|----------------------------|
| II.F.1 | Post-Accident Monitoring | ALL | a. For noble gas monitor and Iodine/particulate sampling and analysis submit description and justification for deviations from staff requirements. b. Have available the final design information for noble gas monitor and iodine/particulate sampling and analysis. | 1/1/81 1/1/81 |
| II.F.2 | Inst. for Inadequate Core Cooling | ALL | Submit a report detailing the planned instrumentation system for monitoring inadequate core cooling. | 1/1/81 |
| II.K.2.10 | Antic. Trip on LOFW and TT | B&W | Submit final design for anticipatory trip as described in NUREG 0737. | 1/1/81 |
| II.K.2.13 | Thermal Mechanical Report | B&W | Submit report on effects of Hi Pressure Injection on vessel integrity for SB LOCA with no Aux Feedwater. | 1/1/81 |
| II.K.3.2 | PORV/SV Failures | PWR | Submit report on SB LOCA and probability of failure of PORV/SV/RV. | 1/1/81 |
| II.K.3.3 | SRV/SV Failures & Challenges | ALL | Submit report (historical and annually thereafter) of SRV/SV failures and challenges. | 1/1/81 |
| II.K.3.7 | PORV Opening Probability | B&W | Submit report on the probability of a PORV opening during an over-pressurization transient. | 1/1/81 |
| II.K.3.9 | PID Controller Modification | Selected <u>W</u> plants | Modify the Proportional Integral Derivative Controller (as recommended by <u>W</u>). Advise NRC when modification is completed. | 12/1/80 |
| II.K.3.12 | Anticipatory Trip on Turbine Trip | <u>W</u> | Submit confirmation of Anticipatory Trip. If not currently implemented, submit modification design and schedule for implementation. | 1/1/81 |

| <u>Item</u> | <u>Title</u> | <u>Applicability</u> | <u>Specific Requirement</u> | <u>Impl/Doc Subm Dates</u> |
|-------------|---|----------------------|---|--------------------------------|
| II.K.3.13 | Separation of HPCI/RCIC Initiation Levels; Auto Restart of RCIC | BWRs w/HPCI/RCIC | Submit results of evaluation and proposed modification as appropriate. | 1/1/81 |
| II.K.3.16 | Reduce Challenges to RV's | BWR | Submit report on actions planned to reduce RV challenges. | 4/1/81 |
| II.K.3.17 | ECCS System Outages | ALL | Submit report on ECCS outages and propose changes to reduce outages. | 1/1/81 |
| II.K.3.18 | ADS Logic Modifications | BWR | Submit report of feasibility of ADS system logic changes to eliminate need for manual actuation. | 4/1/81 |
| II.K.3.21 | CSS/CPCI Restart | BWR | Submit report of evaluation, proposed modifications and analysis to satisfy staff positions. | 1/1/81 |
| II.K.3.22 | RCIC Suction | BWR w/RCIC | Implement procedures and document verification of this change. | 1/1/81 |
| II.K.3.27 | Common Reference for H ₂ O Level Instruments | BWR | Implement change and submit documentation of changes. | 1/1/81 |
| II.K.3.29 | Isolation Condensor Performance | BWR w/ICs | Submit evaluation of I.C. performance. | 4/1/81 |
| II.K.3.30 | SB LOCA Methods | ALL | Submit outline of program for model. | 11/15/80 |
| II.K.3.44 | Fuel Failure | BWR | Submit evaluation to verify no fuel failure. | 1/1/81 |
| II.K.3.45 | Manual Depressur- ization | BWR | Submit evaluation on other than ADS method for depressurization. | 1/1/81 |

| <u>Item</u> | <u>Title</u> | <u>Applicability</u> | <u>Specific Requirement</u> | <u>Impl/Doc</u> <u>Subm Due Dates</u> |
|-------------|---------------------------------------|----------------------|---|--|
| III.D.3.3 | Improved Implant Iodine Monitoring | ALL | Have available means to accurately measure airborne radioiodine in plant during an accident. | 1/1/81 |
| III.D.3.4 | Control Room Habitability | ALL | a. Submit control room habitability evaluation information. | 1/1/81 |
| | | | b. Submit modifications necessary to assure CR habitability with a schedule for completion. | 1/1/81 |