

FEBRUARY 28 1978

Docket No. 50-220

Niagara Mohawk Power Corporation
ATTN: Mr. Donald P. Dise
Vice President - Engineering
300 Erie Boulevard West
Syracuse, New York 13202

Gentlemen:

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The Commission has issued the enclosed Amendment No. 22 to Facility License No. DPR-63 for Unit No. 1 of the Nine Mile Point Nuclear Station. This amendment consists of changes to the Technical Specifications and is in response to your request dated February 28, 1977 as revised by your letters dated July 13, 1977 and July 22, 1977 and our letter dated November 23, 1977.

The amendment consists of changes in the Technical Specifications that incorporate the Fire Protection System into the Limiting Conditions for Operation, Surveillance Requirements and Administrative Controls.

The Specifications are the same as those transmitted to you on November 23, 1977 and found acceptable to you by letter dated December 13, 1977. The amendment is supported by the related Safety Evaluation which also was attached to the November 23, 1977 letter.

A copy of the FEDERAL REGISTER Notice also is enclosed.

Sincerely,

Original signed by

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:

- Amendment No. 22 to License DPR-63
- FEDERAL REGISTER Notice

cc w/enclosures:
see next page

*SEE PREVIOUS YELLOW FOR CONCURRENCES

OFFICE >	ORB#3	ORB#3	ORB#1	OELD	ORB#3	
SURNAME >	*SNowicki	*RClark:acr	*TWambach	*LBrenner	GLear <i>GL</i>	
DATE >	2/21/78	2/22/78	2/21/78	2/27/78	2/28/78	

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 Syracuse, New York 13202

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Sincerely,

George Lear, Chief
 Operating Reactors Branch #3
 Division of Operating Reactors

Enclosures:

1. Amendment No. to License DPR-63
2. FEDERAL REGISTER Notice

cc w/enclosures: See next page

OFFICE →	ORB #3 <i>gr</i>	ORB #3 <i>gla</i>	ORB #1 <i>JVM</i>	OELD	ORB #3	
SURNAME →	SNowicki:mjf	RClark	TWambach	L.Brenner	GLear	
DATE →	2/21/78	2/24/78	2/21/78	2/27/78	1/78	



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

February 28, 1978

Docket No. 50-220

Niagara Mohawk Power Corporation
ATTN: Mr. Donald P. Dise
Vice President - Engineering
300 Erie Boulevard West
Syracuse, New York 13202

Gentlemen:

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A copy of the FEDERAL REGISTER Notice also is enclosed.

Sincerely,

A handwritten signature in cursive script that reads "George Lear".

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:

1. Amendment No. 22 to
License DPR-63
2. FEDERAL REGISTER Notice

cc w/enclosures:
see next page

Niagara Mohawk Power Corporation - 2 -

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R. D. #4
Oswego, New York 13126

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U. S. Environmental Protection Agency
Region II Office
ATTN: EIS COORDINATOR
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New York, New York 10007

Oswego County Office Building
46 E. Bridge Street
Oswego, New York 13126



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 22
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated February 28, 1977 (as revised by letter dated July 31, 1977), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

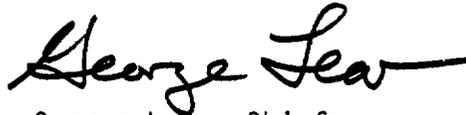
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 22, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 28, 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 22
TO THE TECHNICAL SPECIFICATIONS
FACILITY OPERATING LICENSE NO. DPR-63
DOCKET NO. 50-220

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised page is identified by Amendment number and contains vertical lines indicating the area of change. Add pages iiii, 4a, 241m through 241dd, 245a and 268.

Remove

v
246
247
249
250
251
252
255
256

Replace

v
246
247
249
250
251
252
255
256

SECTION

DESCRIPTION

PAGE

3.6.5	Radioactive Material Sources	4.6.5	Radioactive Material Sources	241k
3.6.6	Fire Detection	4.6.6	Fire Detection	241m
3.6.7	Fire Suppression	4.6.7	Fire Suppression	241q
3.6.8	Carbon Dioxide Suppression System	4.6.8	Carbon Dioxide Suppression System	241u
3.6.9	Fire Hose Stations	4.6.9	Fire Hose Stations	241y
3.6.10	Fire Barrier Penetration Fire Seals	4.6.10	Fire Barrier Penetration Fire Seals	241cc

SECTION	DESCRIPTION	PAGE
6.11	Radiation Protection Program	260
6.12	Respiratory Protection Program	260
6.13	High Radiation Area	263
6.14	Fire Protection Inspection	268

1.16 Fire Suppression Water System

A Fire Suppression Water System shall consist of: a water supply system, fixed extinguishing systems of both automatic sprinklers and sprays, and manual fire fighting equipment consisting of standpipe risers with hose connections and hose reels.

LIMITING CONDITION FOR OPERATION

3.6.6 FIRE DETECTION

Applicability:

Applies to the operational status of the fire detection system.

Objective:

To assure the capability of fire detection instrumentation for each fire detection zone shown in Table 3.6.6a to provide fire detection.

Specification:

- a. With the number of detectors OPERABLE less than the number required by Table 3.6.6a.
 1. Within one hour, establish a fire watch patrol to inspect the zone with the inoperable detector(s) at least once per hour; and
 2. Restore the inoperable detector(s) to OPERABLE status within 14 days
OR
 3. Prepare and submit a special report to the Commission, within the next 30 days outlining the cause of the malfunction and the plans for restoring the detector(s) to OPERABLE status.

SURVEILLANCE REQUIREMENT

4.6.6 FIRE DETECTION

Applicability:

Applies to the periodic surveillance of the fire detection system.

Objective:

To assure the operability of the fire detection instrumentation for each fire detection zone shown in Table 3.6.6a to provide fire detection.

Specification:

- a. Each of the fire detectors shall be demonstrated OPERABLE:
 1. By performance of an instrument channel test at least once per six months for smoke and heat activated devices.

Table 3.6.6a

FIRE DETECTORS PROTECTING SAFETY-RELATED EQUIPMENT

<u>Detector Location</u>	<u>Minimum Detectors Required Operable</u>	
1. Motor Generator Sets for the Reactor Recirculation Pumps:	Heat	Smoke
a. Five (5) reactor recirculation pump M-G sets on El. 261' located along west wall of the Turbine Building. (5 detectors for each M-G set.)	3 out of 5 detectors per M-G set	Note 1
2. Power Board 102 (2 detectors)		
a. Elevation 261'	1 out of 2	Note 1
3. Power Board 103 (2 detectors)		
a. Elevation 261'	1 out of 2	Note 1
4. Diesel Generator 102 (4 detectors)		
a. Elevation 261' along east side of Turbine Building	2 out of 4	Note 1
5. Diesel Generator 103 (4 detectors)		
a. Elevation 261' along east side of Turbine Building	2 out of 4	Note 1

Table 3.6.6a

FIRE DETECTORS PROTECTING SAFETY-RELATED EQUIPMENT

(Continued)

<u>Detector Location</u>	<u>Minimum Detectors Required Operable</u>	
	Heat	Smoke
6. Auxiliary Control Room		
a. Elevation 261' along east side of Turbine Building	Note 1	5 out of 9
7. Cable Spreading Room		
a. Elevation 250'	Note 1	5 out of 9

Note 1 - These types of detectors are not installed in this area.

Automatic fire detection instruments are provided in certain areas where early fire detection is desirable. The fire detection instruments activate an annunciator system in the main control room which informs the operator of the fire location.

The two types of detection instruments employed in the plant are rate compensation thermal devices and ionization type smoke-detection devices.

The configuration of the fire detection instrument locations have been examined and found satisfactory to detect a fire with the minimum number of detectors operable as indicated in Table 3.6.5a.

LIMITING CONDITION FOR OPERATION

3.6.7 FIRE SUPPRESSION

Applicability:

Applies to the operational status of the fire suppression system.

Objective:

To assure the capability of the fire suppression system to provide fire suppression in the event of a fire.

Specification:

- a. The FIRE SUPPRESSION WATER SYSTEM shall be OPERABLE with;
 - 1. Two high pressure pumps each with a capacity of 2500 gal./min. with their discharge aligned to the fire suppression header.
 - 2. Automatic initiation logic for each fire pump.

SURVEILLANCE REQUIREMENT

4.6.7 FIRE SUPPRESSION

Applicability:

Applies to the surveillance of the fire suppression system.

Objective:

To assure the operability of the fire suppression system to provide fire suppression in the event of a fire.

Specification:

- a. The FIRE SUPPRESSION WATER SYSTEM shall be demonstrated OPERABLE:
 - 1. At least once per 31 days by starting each pump and operating it for 15 minutes on recirculation flow.
 - 2. At least once per year by cycling each manually-operable valve through one complete cycle.

LIMITING CONDITION FOR OPERATION

- b. With an inoperable redundant pump or water supply line inoperable, restore the inoperable equipment to OPERABLE status within 7 days or prepare and submit a Special Report to the Commission within the next 30 days outlining the plans and procedures to be used for the loss of redundancy in this system.
- c. With no FIRE SUPPRESSION WATER SYSTEM operable, within 24 hours
 - 1. Establish a backup fire suppression system and submit a Special Report to the Commission by telephone within 24 hours confirmed by telephone mailgram or facsimile transmission, and in writing no later than 14 days following the event, outlining the actions to be taken and the plans and schedule for restoring the system to OPERABLE status, or
 - 2. Place the reactor in Hot Standby within one hour and in Cold Shutdown within the following 36 hours.

SURVEILLANCE REQUIREMENT

- 3. At least once per year by a flush of the hydrants.
- 4. At least once per operating cycle:
 - (a) By performing a system automatic start on low header pressure.
 - (b) By verifying that each pump will develop a flow of at least 2500 gpm at a pump discharge of 115 psig.
 - (c) Cycling each valve in the flow path that is not testable during plant operation through at least one complete cycle of full travel.
- 5. At least once per 3 years by performing a flow test of the system in accordance with Chapter 5, Section 11 of the Fire Protection Handbook, 14th Edition, published by the National Fire Protection Association.
- 6. At least once per 31 days by verifying that each valve (manual, power operated or automatic) in the flow path is in its correct position.
- b. The fire pump diesel engine shall be demonstrated OPERABLE:
 - 1. At least once per 31 days by verifying that the fuel storage tank contains at least 150 gallons of fuel.

LIMITING CONDITION FOR OPERATION

- e. The spray and sprinkler systems located in the following areas shall be OPERABLE:
 - 1. Automatic water spray systems
 - (a) Reserve Transformer 101N
 - (b) Reserve Transformer 101S
 - 2. Automatic Sprinkler System for the Diesel Fire Pump Room in the Screen House.
- f. With a spray or sprinkler system inoperable, establish a continuous fire watch with backup fire suppression equipment for the unprotected area(s), within one hour.
- g. Restore the system to OPERABLE status within 14 days or prepare and submit a Special Report to the Commission within the next 30 days outlining the cause of inoperability and the plans for restoring the system to OPERABLE status.

SURVEILLANCE REQUIREMENT

- 2. At least once per 92 days by verifying that a sample of diesel fuel from the fuel storage tank, obtained in accordance with ASTM-D270-65, is within the acceptable limits specified in Table 1 of ASTM D975-74 with respect to viscosity, water control, and sediment.
 - 3. At least once per 18 months, during shutdown, subjecting the diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations for the class of service, and verifying the diesel starts from ambient conditions on the auto-start signal and operates for ≥ 20 minutes while loaded with the fire pump.
- c. The spray systems shall be demonstrated to be OPERABLE:
- 1. At least once per year by cycling each manually operable valve through one complete cycle.
 - 2. At least once per operating cycle:
 - (a) By performing a system functional test which includes simulated automatic actuation of the system and verifying that the automatic deluge valves in the flow path actuate to their correct positions.
 - (b) By visual inspection of spray headers to verify their integrity.
 - (c) By visual inspection of each nozzle to verify no blockage.

LIMITING CONDITION FOR OPERATION

SURVEILLANCE REQUIREMENT

- (d) At least once per 3 years by performing an air flow test through each open head spray/sprinkler header and verifying each open head spray/sprinkler nozzle is unobstructed.
- d. The sprinkler system shall be demonstrated to be OPERABLE:
 - 1. At least once per operating cycle:
 - a) By performing a system functional test which includes simulated automatic actuation of the system.
 - b) By visual inspection of sprinkler headers to verify their integrity.
 - c) By visual inspection of each nozzle to verify no blockage.

The fire water supply is provided by two vertical turbine fire pumps, one electric and a diesel-driven unit which are design rated at 2500 gpm at 125 psig pump discharge head. These pumps are located in the screen house and take suction from the station cooling water intake tunnel and have relief valves set at 140 psig.

The automatic initiation logic for each fire pump indicated in Specification 3.6.7.a.2 requires that these pumps are automatically started together upon a drop in discharge header pressure. Each pump can also be manually started.

The verification of the hydraulic performance of the fire suppression water system required once per 3 years in Surveillance Requirement 4.6.7.a.5 will be done by means of a measured hydrant flow test.

The redundant components in the fire water supply system are the fire pumps, which discharge to the same header. They are the only components addressed in Specification 3.6.7.b.

The backup water supply system referenced in Specification 3.6.7.c.2 is the Oswego City water system, which can be connected to the fire main if required.

The water spray systems provide fire protection for the safety-related reserve transformers 101N and 101S. Supply for these systems is provided by the fire line. The systems employ open nozzles and are controlled by deluge valves. Valve actuation is by pneumatic type rate-of-rise devices installed over the protected equipment.

In addition to the automatic operation, systems may be tripped manually either at the deluge valves on elevation 250' or at remote cable pull stations on elevation 261'.

The fire control panel annunciator records system operation, low supervisory air pressure and valve closure.

In addition to the spray systems described above, a closed head wet pipe automatic sprinkler system is provided for the diesel fire pump room in the Screen House on Elevation 261'. The sprinkler heads used have fusible elements rated at 165°F. The system has flow alarms connected to the fire control panel annunciator.

LIMITING CONDITION FOR OPERATION

3.6.8 CARBON DIOXIDE SUPPRESSION SYSTEM

Applicability:

Applies to the operational status of the carbon dioxide suppression system.

Objective:

To assure the capability of the carbon dioxide suppression system to provide fire suppression in the event of a fire.

Specification:

- a. The CO₂ system, which supplies the Recirculation Pumps Motor-Generator Sets, Power Boards 103 and 104, Diesel Generators 102 and 103, Cable Room and the Auxiliary Control Room fire hazards, shall be OPERABLE with a minimum level of 40% of tank and a minimum pressure of 250 psig in the storage tank.
- b. With a CO₂ system inoperable establish a continuous fire watch with backup fire suppression equipment for the unprotected area(s) within one hour.

SURVEILLANCE REQUIREMENT

4.6.8 CARBON DIOXIDE SUPPRESSION SYSTEM

Applicability:

Applies to the periodic surveillance requirements of the carbon dioxide suppression system.

Objective:

To verify the operability of the carbon dioxide suppression system.

Specification:

- a. The CO₂ system shall be demonstrated OPERABLE:
 1. At least once per 7 days by verifying the CO₂ storage tank level and pressure.
 2. At least once per operating cycle by verifying the system valves and associated ventilation dampers actuate automatically to a simulated actuation signal. A brief flow test shall be made to verify flow from each nozzle. ("Puff Test.")

LIMITING CONDITION FOR OPERATION

3.6.8 CARBON DIOXIDE SUPPRESSION SYSTEM (Continued)

- c. Restore the system to OPERABLE status within 14 days or prepare and submit a Special Report to the Commission within the next 30 days outlining the cause of inoperability and the plans for restoring the system to operable status.

A low pressure carbon dioxide system is installed to serve seven different safety-related hazard points in the station indicated in Specification 3.6.8.a.

Supply is provided by a 10 ton tank of liquid carbon dioxide located on elevation 261 ft. The self-contained refrigeration unit maintains the liquid at 0°F with a resultant pressure of 300 psig. Carbon dioxide to the individual hazards is controlled by a series of carbon dioxide operated, pilot type master valves at the tank. Each of these valves serve a group of selector valves of similar construction located at the individual areas.

Fire extinguishment by carbon dioxide is either by total flooding or local application. In total flooding, sufficient CO₂ is injected into a closed room or space to inert the atmosphere and suppress combustion. Local application is employed for unenclosed areas and involves application of CO₂ on the equipment protected to extinguish the fire with additional discharge to permit cooling and inhibit reflash.

The automatically actuated CO₂ systems employ either thermostats set at 225°F or smoke detectors to trip a timer located in the main cardox control cabinet. One or more sirens in the hazard area are initially operated for a pre-discharge period of 30 seconds to enable personnel to leave the area. The related master and selector valves are then opened for a timed discharge period. Restoration of the CO₂ hazard area to service is accomplished manually by push-button at the fire control panel. Manual push-button stations are also located at the individual areas to initiate the cycle. The control switch for each area on the fire control panel has three positions and is normally set for "Automatic" operation. An "Alarm only" position permits greater safety when men are working in the hazard area and the 30 second delay may be insufficient. A "Manual" position permits the operator to actuate the discharge cycle on his own initiative. An area push-button station will override the "Alarm only" setting on the Fire Control Panel.

All CO₂ systems except hose reels are provided with odorizing devices as a safety measure. A glass flask of wintergreen concentrate is inserted in a capped tee beyond each selector valve. This flask ruptures upon operation of the hazard and must be replaced after each use.

In the event of total loss of D.C. control power to the CO₂ system, all master valves will open since their pilot valve solenoids are normally energized. The CO₂ system selector valves remain closed since their pilot valve solenoids are normally de-energized. CO₂ can be discharged into a area by operating the manual lever provided in each pilot valve cabinet. This is a manual operation within pre-discharge alarm or timer.

The flow test ("Puff Test") of the CO₂ system is performed by closing the CO₂ tank valve, which automatically trips the system (opens the master and selector valves). This allows only the CO₂ vapor in the line to be discharged to the various designated areas in the plant.

Carbon dioxide hose reels are provided at various points throughout the Turbine Building. These reels are provided with 150 feet of 1" high pressure hose with manual shutoff at the nozzle. Removal of the nozzle

from its mounting bracket trips a switch which opens the master valve serving the hose reels. Carbon dioxide then flows to the nozzles of all hose reels. No odorant capsules are provided for hose reels. Certain hose stations are provided with timer operated bleeder valves to discharge vapor and speed arrival of liquid CO₂ at the hose station.

All system operations are monitored on the annunciator on the fire control panel.

LIMITING CONDITION FOR OPERATION

3.6.9 FIRE HOSE STATIONS

Applicability:

Applies to the operational status of the fire hose stations.

Objective:

To assure the capability of the fire hose stations to provide fire suppression in the event of a fire.

Specification:

- a. The fire hose stations in the locations shown in Table 3.6.9a shall be operable.
- b. With a hose station inoperable, provide an additional hose of equivalent capacity where necessary to reach the effected area from an OPERABLE hose station within one hour.

SURVEILLANCE REQUIREMENT

4.6.9 FIRE HOSE STATIONS

Applicability:

Applies to the periodic surveillance of the fire hose stations.

Objective:

To assure the operability of the fire hose stations to provide fire suppression in the event of a fire.

Specification:

- a. Each fire hose station shall be verified to be OPERABLE:
 1. At least once per 30 days by visual inspection of the station to assure all equipment is available.
 2. At least once per operating cycle by removing the hose for inspection and racking and replacing all gaskets in the couplings that are degraded.
 3. At least once per 3 years, partially open hose station valves to verify valve operability and no blockage and conducting a hose hydrostatic test at a pressure at least 50 psig greater than the maximum pressure available at that hose station.

Table 3.6.9a
FIRE HOSE STATIONS

	<u>Building</u>	<u>Elevation (feet)</u>	<u>Column</u>	<u>Station Number</u>
1.	TURBINE	267	Aa-7	FS-144
2.	TURBINE	267	C -3	FS-132
3.	TURBINE	267	G -2	FS-128
4.	TURBINE	267	H -9	FS-123
5.	REACTOR	346	L -12	FS-112
6.	REACTOR	346	P -4	FS-106
7.	REACTOR	324	K -11	FS-111
8.	REACTOR	324	P -5	FS-105
9.	REACTOR	309	K -11	FS-110
10.	REACTOR	304	P -5	FS-104
11.	REACTOR	287	K -11	FS-109
12.	REACTOR	287	P -5	FS-103
13.	REACTOR	267	K -11	FS-108
14.	REACTOR	267	P -5	FS-102
15.	REACTOR	243	K -11	FS-107
16.	REACTOR	243	P -5	FS-101
17.	WASTE	267	H -19	FS-301
18.	WASTE	267	MB-19	FS-300
19.	WASTE	267	MB-16	FS-116
20.	TURBINE	375	H -8	FS-126
21.	TURBINE	339	H -10	FS-121
22.	TURBINE	311	H -9	FS-125
23.	TURBINE	311	G -2	FS-130
24.	TURBINE	311	G -3	FS-134
25.	TURBINE	297	H -9	FS-124
26.	TURBINE	297	G -2	FS-129
27.	TURBINE	267	F -15	FS-117
28.	DIESEL	267	C -18	FS-164
29.	DIESEL	267	Ba-17	FS-166
30.	TURBINE	256	Aa-13	FS-152
31.	DIESEL	256	Aa-17	FS-163
32.	DIESEL	256	C -17	FS-165
33.	TURBINE	256	H -9	FS-122

Table 3.6.9a
 FIRE HOSE STATIONS
 (Continued)

<u>Building</u>	<u>Elevation (feet)</u>	<u>Column</u>	<u>Station Number</u>
34. TURBINE	267	Aa-14	FS-156
35. TURBINE	267	B -2	FS-139
36. TURBINE	267	P -14	FS-114
37. TURBINE	297	C -3	FS-133
38. TURBINE	283	B -2	FS-140
39. TURBINE	283	Aa-7	FS-145
40. TURBINE	283	Aa-13	FS-153
41. TURBINE	283	F -15	FS-118
42. TURBINE	256	Aa-7	FS-143
43. TURBINE	256	B -2	FS-138
44. TURBINE	256	C -3	FS-131
45. TURBINE	256	G -2	FS-127
46. TURBINE	256	M -13	FS-115
47. TURBINE	283	Be-14	FS-162

BASES FOR 3.6.9 AND 4.6.9 FIRE HOSE STATIONS

Standpipe risers at various locations in the turbine, reactor, waste and diesel buildings serve hose connections. This equipment is located to permit hose stream coverage of safety-related equipment in the buildings. Each hose connection is equipped with 100 feet of 1 1/2 inch hose mounted on a reel.

All hand line nozzles are of the adjustable spray type which can be varied down to 10° minimum spray pattern to render them safe for use on electrical equipment. Eight foot long applicator spray nozzles and foam induction nozzles with five gallon cans of foam solution are also provided for use on hose lines as required.

LIMITING CONDITION FOR OPERATION

3.6.10 FIRE BARRIER PENETRATION FIRE SEALS

Applicability:

Applies to the condition of the fire barrier penetration fire seals.

Objective:

To assure the capability of the fire barrier penetration fire seals to perform their intended function.

Specification:

- a. All fire barrier penetration fire seals protecting safety related areas shall be intact.
- b. With a fire barrier penetration fire seal not intact, a continuous fire watch shall be established on at least one side of the affected penetration within one hour.

SURVEILLANCE REQUIREMENTS

4.6.10 FIRE BARRIER PENETRATION FIRE SEALS

Applicability:

Applies to the periodic surveillance requirements for the fire barrier penetration fire seals.

Objective:

To verify the condition of the fire barrier penetration fire seals.

Specification:

- a. Fire barrier penetration fire seals shall be verified to be functional by:
 1. A visual inspection at least once per operating cycle, and
 2. A visual inspection of a fire barrier penetration seal after repair or maintenance.

BASES FOR 3.6.10 AND 4.6.10 FIRE BARRIER PENETRATION FIRE SEALS

Cable penetrations of the primary containment (drywell and pressure suppression chamber), reactor building, auxiliary control room and the cable room have been designed to provide adequate fire stop and to prevent a fire from spreading through the penetration. Drywell and pressure suppression chamber penetrations are double-sealed, 12-inch pipes that are inerted with nitrogen. Reactor building penetrations consist of standard conduit (pipe) sleeves, which vary in diameter from 3/4" to 4" and which are sealed at both ends. The auxiliary control room and the cable room have formed pipe sleeves and cable tray penetrations. These sleeves and penetrations are sealed at the ends with rock-wool filler and externally applied fire-resistant material for fire proofing.

The local leak test required in Surveillance Requirement 4.6.10.a.2 will be performed by a non-hazardous method to ensure penetration integrity (an example of an acceptable local leak testing method is the "Downy Wand Test" or equivalent).

Facility Staff (Cont'd)

- f. A Fire Brigade of four (5) members shall be maintained on site at all times. This excludes two (2) members of the minimum shift crew necessary for safe shutdown and any personnel required for other essential functions during a fire emergency.

Figure 6.2-1
Nine Mile Point Nuclear Station
Management Organization Chart

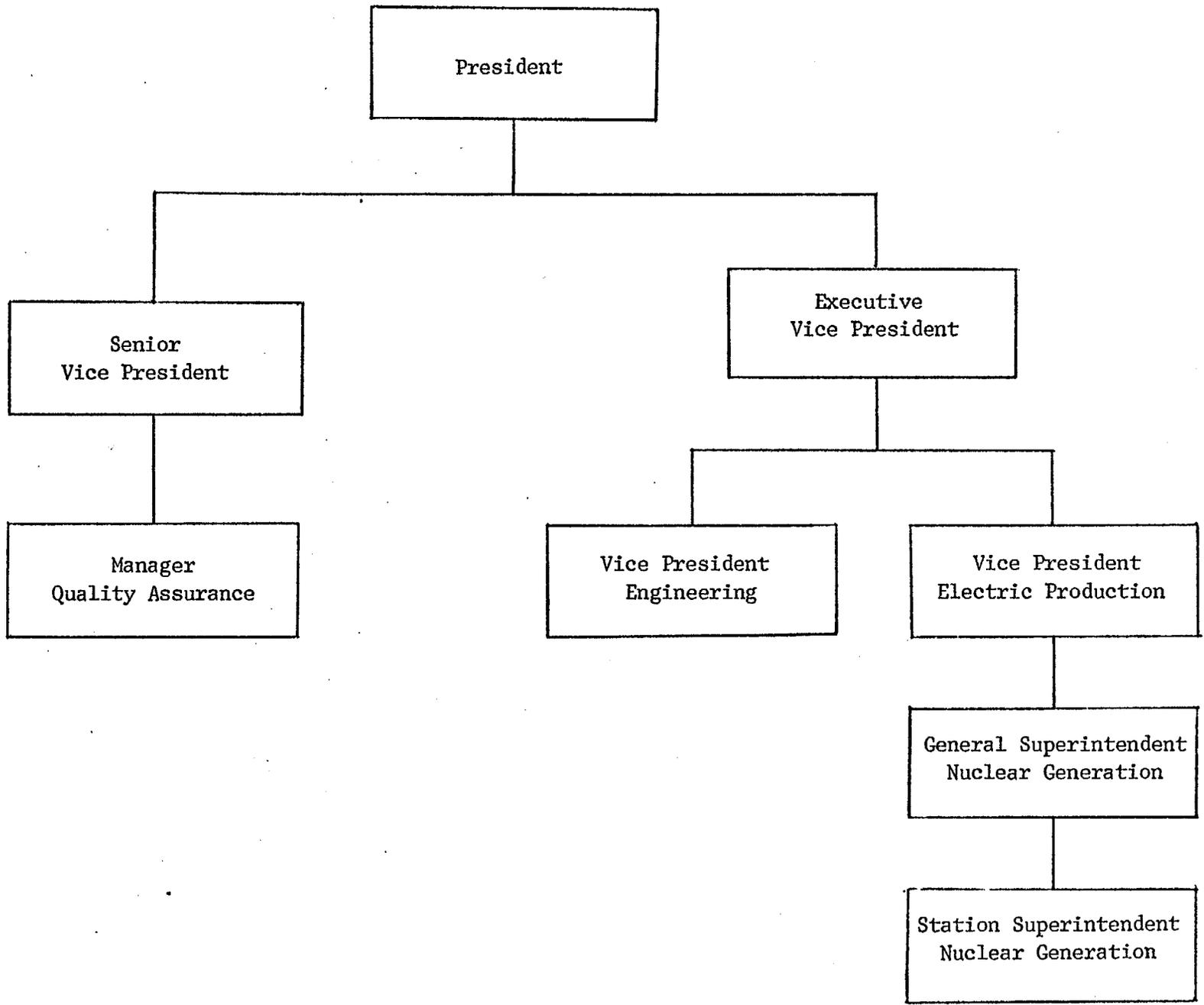
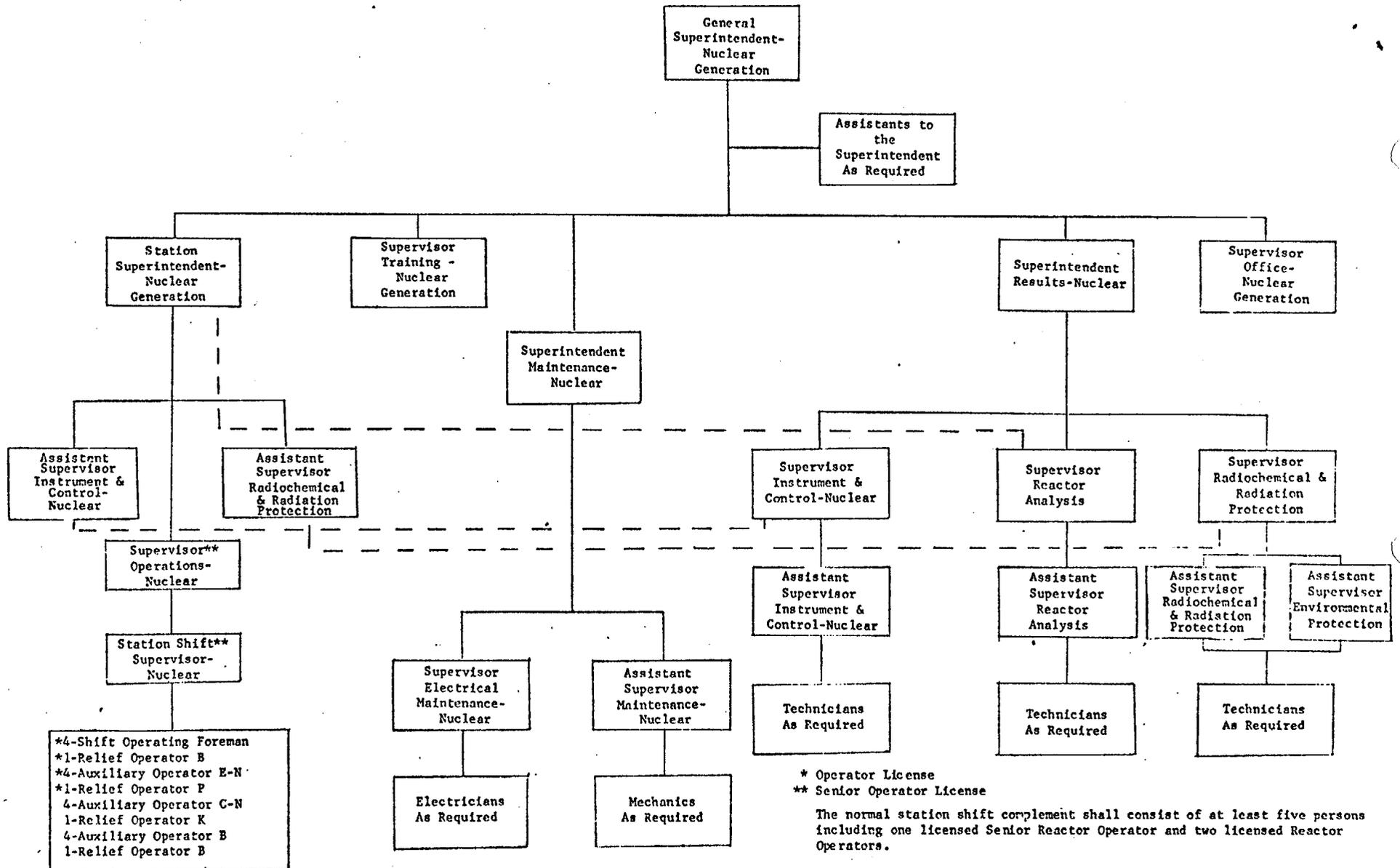


FIGURE 6.2-2
 NINE MILE POINT NUCLEAR SITE
 OPERATION ORGANIZATION



6.3 Facility Staff Qualifications

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions.

6.4 Training

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Training Supervisor and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Training Supervisor and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975 except for Fire Brigade Training sessions which shall be held at least quarterly.

6.5 Review and Audit

6.5.1 Site Operations Review Committee (SORC)

Function

6.5.1.1 The Site Operations Review Committee shall function to advise the General Superintendent Nuclear Generation on all matters related to nuclear safety.

Composition

6.5.1.2 The Site Operations Review Committee shall be composed of the:

Chairman:	General Superintendent Nuclear Generation
Member:	Station Superintendent-Nuclear Generation
Member:	Superintendent Results-Nuclear
Member:	Supervisor Reactor Analysis
Member:	Superintendent Maintenance-Nuclear
Member:	Supervisor Instrument & Control-Nuclear
Member:	Supervisor Radiochemical & Radiation Protection

Alternates

6.5.1.3 Alternate members shall be appointed in writing by the SORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in SORC activities at any one time.

Meeting Frequency

6.5.1.4 The SORC shall meet at least once per calendar month and as convened by the SORC Chairman.

Quorum

6.5.1.5 A quorum of the SORC shall consist of the Chairman and four members including alternates.

Responsibilities

6.5.1.6 The Site Operations Review Committee shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the General Superintendent Nuclear Generation to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to the Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications and shall prepare and forward a report covering evaluation and recommendations to prevent recurrence to the Vice President - Electric Production and to the Chairman of the Safety Review and Audit Board.

Responsibilities (cont'd)

- f. Review of facility operations to detect potential safety hazards.
- g. Performance of special reviews and investigations and reports thereon as requested by the Chairman of the Safety Review and Audit Board.
- h. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Chairman of the Safety Review and Audit Board.
- i. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Chairman of the Safety Review and Audit Board.

Authority

6.5.1.7 The Site Operations Review Committee shall:

- a. Recommend to the General Superintendent Nuclear Generation written approval or disapproval of items considered under 6.5.1.6 (a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6 (a) through (e) above constitutes an unreviewed safety question.
- c. Provide immediate written notification to the Vice President - Electric Production and the Safety Review and Audit Board of disagreement between the SORC and the General Superintendent Nuclear Generation; however, the General Superintendent Nuclear Generation shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

Records

6.5.1.8 The Site Operations Review Committee shall maintain written minutes of each meeting and copies shall be provided to the Vice President - Electric Production and Chairman of the Safety Review and Audit Board.

6.5.2 Safety Review and Audit Board (SRAB)

Function

6.5.2.1 The Safety Review and Audit Board shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices
- i. (other appropriate fields associated with the unique characteristics of the nuclear power plant)

Composition

6.5.2.2 The Safety Review and Audit Board shall be composed of the:

- | | |
|-----------|---|
| Chairman: | Staff Engineer or Manager |
| Member: | General Superintendent Nuclear Generation |
| Member: | Staff Engineer - Nuclear |
| Member: | Staff Engineer - Mechanical or Electrical |
| Member: | Staff Engineer - Environmental |
| Member: | Consultant (See 6.5.2.4) |

Audits (cont'd)

- d. The performance of all activities required by the Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per two years.
- e. The Facility Emergency Plan and implementing procedures at least once per two years.
- f. The Facility Security Plan and implementing procedures at least once per two years.
- g. The Facility Fire Protection Program and implementing procedures at least once per two years.
- h. Any other area of facility operation considered appropriate by the SRAB, the Vice President - Electric Operations or the Vice President - Engineering.

Authority

- 6.5.2.9 The SRAB shall report to and advise the Vice President - Electric Production and Vice President - Engineering on those areas of responsibility specified in Section 6.5.2.7 and 6.5.2.8.

Records

- 6.5.2.10 Records of SRAB activities shall be prepared, approved and distributed as indicated below:
- a. Minutes of each SRAB meeting shall be prepared, approved and forwarded to the Vice President - Electric Production and Vice President - Engineering within 30 days following each meeting.
 - b. Reports of reviews encompassed by Section 6.5.2.7 e,f,g and h above, shall be prepared, approved and forwarded to the Vice President - Electric Production and Vice President - Engineering within 14 days following completion of the review.
 - c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Vice President - Electric Production and Vice President - Engineering and to the management positions responsible for the areas audited within 90 days after completion of the audit.

6.6 Reportable Occurrence Action

6.6.1 The following actions shall be taken in the event of a REPORTABLE OCCURRENCE:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each Reportable Occurrence Report submitted to the Commission shall be reviewed by the SORC and submitted to the SRAB and the Vice President - Electric Production.

6.7 Safety Limit Violation

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR 50.36(c)(1)(i) shall be complied with immediately.
- b. The Safety Limit violation shall be reported to the Commission, the Vice President-Electric Production and to the SRAB immediately.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the SRAB and the Vice President - Electric Production within 10 days of the violation.

6.8 Procedures

6.8.1 Written procedures and administrative policies shall be established, implemented and maintained that meet or exceed the requirements and recommendations of Sections 5.1 and 5.3 of ANSI N18.7-1972 and Appendix "A" of USAEC Regulatory Guide 1.33 except as provided in 6.8.2 and 6.8.3 below.

6.14 Fire Protection Inspection

- 6.14.1 An independent fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified off-site licensee personnel or an outside fire protection firm.
- 6.14.2 An inspection and audit by an outside qualified fire consultant shall be performed at intervals no greater than 3 years.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-220NIAGARA MOHAWK POWER CORPORATIONNOTICE OF ISSUANCE OF FACILITY LICENSE AMENDMENT

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 22 to Facility Operating License No. DPR-63 to the Niagara Mohawk Power Corporation (the licensee) which revised Technical Specifications for operation of the Nine Mile Point Nuclear Station, Unit No. 1 (the facility) located in Oswego County, New York. The amendment is effective as of its date of issuance.

The amendment incorporated fire protection Technical Specifications on the existing fire protection equipment and adds administrative controls related to fire protection at the facility. This action is being taken pending completion of the Commission's overall fire protection review of the facility.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

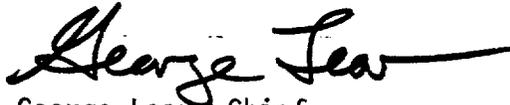
The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

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For further details with respect to this action, see (1) the application for amendment dated February 28, 1977, as revised by letters dated July 13, 1977 and July 22, 1977, (2) Amendment No. 22 to License No. DPR-63, (3) the Commission's related Safety Evaluation transmitted to licensee by letter dated November 23, 1977, and (4) the licensee's letter of acceptance dated December 13, 1977. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Oswego County Office Building, 46 E. Bridge Street, Oswego, New York 13126. A copy of items (2), (3) and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland this 28th day of February 1978.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors