

2/12/76

Docket No. 50-220

Niagara Mohawk Power Corporation  
ATTN: Mr. Gerald K. Rhode  
Vice President - Engineering  
300 Erie Boulevard West  
Syracuse, New York 13202

Gentlemen:

The Commission has issued the enclosed Amendment No. 7 to Facility License No. 63 for the Nine Mile Point Unit 1. This amendment is in response to your request dated August 11, 1975.

This amendment revised the provisions in the Environmental Technical Specifications relating to reporting requirements in order to establish consistency with current NRC guidelines. In addition, editorial corrections and changes have been made to reflect the reorganization of the Atomic Energy Commission to form the Nuclear Regulatory Commission.

We have evaluated the potential for environmental impact of plant operation in accordance with the enclosed amendment. The amendment applies only to administrative details. We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level, and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental statement, negative declaration or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Since the amendment applies only to administrative details, it does not involve significant new safety information of a type not considered by a previous Commission safety review of the facility. It does not involve a

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significant increase in the probability or consequences of an accident, does not involve a significant decrease in a safety margin, and therefore does not involve a significant hazards consideration. We have also concluded that there is reasonable assurance that the health and safety of the public will not be endangered by this action.

A copy of the related Federal Register Notice is also enclosed.

Sincerely,

George Lear, Chief  
Operating Reactors Branch #3  
Division of Operating Reactors

Enclosures:

- 1. Amendment No. 7 to DPR-63
- 2. Federal Register Notice

cc: See Attached List

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cc:

Arvin E. Upton, Esquire  
LeBoeuf, Lamb, Leiby & MacRae  
1757 N Street, N. W.  
Washington, D. C. 20036

Anthony Z. Roisman, Esquire  
Roisman, Kessler and Cashdan  
1712 N Street, N. W.  
Washington, D. C. 20036

Dr. William Seymour, Staff Coordinator  
New York State Atomic Energy Council  
New York State Department of Commerce  
112 State Street  
Albany, New York 12207

Oswego City Library  
120 E. Second Street  
Oswego, New York 13126

Mr. Robert P. Jones, Supervisor  
Town of Scriba  
R. D. #4  
Oswego, New York 13126



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT UNIT 1

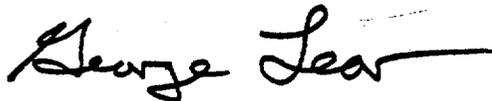
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 7  
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated August 11, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION.

A handwritten signature in cursive script that reads "George Lear". The signature is written in dark ink and is positioned above the typed name and title.

George Lear, Chief  
Operating Reactors Branch #3  
Division of Operating Reactors

Attachment:  
Changes to the  
Technical Specifications

Date of Issuance: FEB 12 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 7  
TO THE ENVIRONMENTAL TECHNICAL SPECIFICATIONS  
FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Replace pages 4, 5, 29, 38, 52, 60, 69, 72, 75, 78, 80, 81, 82, 84, 85,  
88, 88a, 92, and 93 with the attached revised pages.

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## BASES FOR INTERIM GASEOUS WASTE SPECIFICATIONS

- a. Detailed studies were conducted to establish a calculated rate for stack emission to the uncontrolled environment in accordance with the limits of 10 CFR 20 and are described in Appendix D of the FSAR. These calculations consider site meteorology, buoyancy characteristics, statistical tolerance for the environmental monitoring program, and isotopic content of the effluent as given in Table A-12 of the FSAR. Independent dose calculations for several locations offsite have been made by the NRC staff. The method utilized onsite meteorological data developed by the licensee and utilized diffusion assumptions appropriate to the site.

The method utilized by the NRC staff is described in Section 7-5.2.5 of "Meteorology and Atomic Energy - 1968", equation 7.63 being used. The results of these calculations were equivalent to those generated by the licensee provided the average gamma energy per disintegration for the assumed noble gas mixture with a 30-minute holdup is 0.7 MeV per disintegration. Based on these calculations, a maximum release rate limit of gross activity, except for iodines and particulates with half lives longer than eight days, in the amount of  $0.57/\bar{E}$  curies per second will not result in offsite annual doses in excess of the limits specified in 10 CFR 20. The  $\bar{E}$  determination need consider only the average gamma energy per disintegration since the controlling whole body dose is due to the cloud passage over the receptor and not cloud submersion in which the beta dose could be additive.

- b. Detailed calculations of ground level air concentrations of iodines and particulates with half lives longer than eight days at several offsite locations have been made as described in Appendix D of the FSAR. These calculations consider site meteorology and buoyancy characteristics of the effluent. Based on these calculations, the release rate limit for these isotopes in the equation in Specification 2.4.3-Ib is obtained. Use of this equation assures that releases will not result in offsite doses in excess of those specified in 10 CFR 20.

The assumptions used by the NRC staff for these calculations were: (1) onsite meteorological data for the most critical 22-1/2 degree sector, (2) no building wake credit used, and (3) to consider possible reconcentration effects a reduction factor of 700 was applied to allow for the milk production and consumption mode of uptake. The reduction factor of 700 has been incorporated into the equation in Specification 2.4.3-Ib.

## BASES FOR LIQUID WASTE SPECIFICATIONS (Cont'd)

Specification 2.4.1.f requires that the licensee maintain and operate the equipment installed in the liquid waste systems to reduce the release of radioactive materials in liquid effluents to as low as practicable consistent with the requirements of 10 CFR Part 50.36a. Normal use and maintenance of installed equipment in the liquid waste system provides reasonable assurance that the quantity released will not exceed the design objective. In order to keep releases of radioactive materials as low as practicable, the Specification requires, as a minimum, operation of equipment whenever it appears that the projected cumulative discharge rate will exceed one-fourth of this design objective annual quantity during any calendar quarter.

Specification 2.4.1.g limits the amount of radioactive material that could be inadvertently released to the environment to an amount that will not exceed the Technical Specification limit.

In addition to limiting conditions for operation listed under Specification 2.4.1.b and 2.4.1.c, the reporting requirements of Specification 2.4.1.h delineate that the licensee shall identify the cause whenever the cumulative release of radioactive materials in liquid waste effluents exceeds one half the design objective annual quantity during any calendar quarter and describe the proposed program of action to reduce such releases to design objective levels on a timely basis. This report must be filed within 30 days following the calendar quarter in which the release occurred.

The sampling and monitoring requirements given under Specification 2.4.2 provide assurance that radioactive materials in liquid wastes are properly controlled and monitored in conformance with the requirements of Design Criteria 60 and 64. These requirements provide the data for the licensee and the Commission to evaluate the plant's performance relative to radioactive liquid wastes released to the environment. Reports on the quantities of radioactive materials released in liquid waste effluents are furnished to the Commission in conformance with 10 CFR 50.36(a)(2) on a semi-annual basis. Data is summarized on a quarterly basis in the annual Environmental Operating Report. additional information the Commission may obtain from the licensee or others, the Commission may from time to time require the licensee to take such action as the Commission deems appropriate.

The points of release to the environment to be monitored in Section 2.4.2 include all the monitored release points as provided for in Table 2.4-3.

BASES FOR GASEOUS WASTE SPECIFICATIONS (Cont'd)

The sampling and monitoring requirements given under Specification 2.4.4 provide assurance that radioactive materials released in gaseous waste effluents are properly controlled and monitored in conformance with the requirements of Design Criteria 60 and 64. These requirements provide the data for the licensee and the Commission to evaluate the plant's performance relative to radioactive wastes released to the environment. Reports on the quantities of radioactive materials released in gaseous effluents are furnished to the Commission in conformance with 10 CFR 50.36(a)(2) on a semi-annual basis. Data is summarized on a quarterly basis in the annual Environmental Operating Report. On the basis of such reports and any additional information the Commission may obtain from the licensee or others, the Commission may from time to time require the licensee to take such action as the Commission deems appropriate.

The points of release to the environment to be monitored in Section 2.4.4 include all the monitored release points as provided for in Table 2.4-4.

### 3.1.1.b (cont'd)

#### (3) Meteorological Monitoring

##### OBJECTIVE

The objective of meteorological monitoring is to adequately measure and document meteorological conditions at the site.

##### SPECIFICATION

The meteorological monitoring system shall conform to the recommendations in Regulatory Guide 1.23, Onsite Meteorological Programs, shall include instruments to sense wind speed and direction, air temperature and vertical air temperature difference at heights above ground that are representative of atmospheric conditions that exist at all gaseous effluent release points.

##### REPORTING REQUIREMENTS

Meteorological data shall be summarized and reported in a format consistent with the recommendations of Regulatory Guides 1.21 and 1.23, and observations in a form consistent with National Weather Service procedures. Summaries of data and observations shall be available to the U. S. Nuclear Regulatory Commission upon request. If the outage time of any of the meteorological instruments exceeds seven consecutive days, the total outage time and dates of outage, the cause of the outage, and the instrument(s) involved shall be reported within 30 days of the initial time of the outage to the U. S. Nuclear Regulatory Commission, Office of Nuclear Reactor Regulation, for modifications to the meteorological monitoring program as described above, and shall have the written approval of the U. S. Nuclear Regulatory Commission, Office of Nuclear Reactor Regulation, prior to initiation of the modification..

##### BASES

The collection of meteorological data at the plant site will provide information which may be used to develop atmospheric diffusion parameters

Within 1 Mile Radius

- Station 1 - In 20 ft of water on the west
- Station 2 - In 40 ft of water on the west
- Station 3 - In 80 ft of water off the plant.
- Station 4 - In 40 ft of water on the east
- Station 5 - In 20 ft of water on the east

Within 3 Mile Radius

- Station 1 - In 20 ft of water on the west
- Station 2 - In 40 ft of water on the west
- Station 3 - In 100 ft of water off the plant
- Station 4 - In 40 ft of water on the east
- Station 5 - In 20 ft of water on the east

Samples shall be collected at surface, mid-depth and bottom with metered nets. Samples shall be collected weekly during day and night from June through August and bi-monthly during the day from April through June and from September through December.

Macrozooplankton samples shall be analyzed once per month from April through December from the ichthyoplankton samples. The surface, mid-depth and bottom samples at each station shall be composited and organisms shall be identified and enumerated. Density in numbers per cubic meter shall be determined for major taxa.

REPORTING REQUIREMENTS

At the end of 1975 (after one year's operation of the FitzPatrick Power Plant) the results from the General Ecological Surveys conducted during 1973, 1974, and 1975 shall be analyzed, evaluated by the licensee and submitted to the Nuclear Regulatory Commission. The report shall include an assessment of the impact of both power stations on the aquatic environment and appropriate substantiated recommendations for modifications to or discontinuance of the various portions of this site monitoring program.

BASES

The information outlined in the Specification will provide data for interpretative analysis of the situation existing in the aquatic ecosystem of Lake Ontario in the Nine Mile Point area. The information collected through 1974 has provided

3.1.2.a(2) (conc'd)

submitted to the Nuclear Regulatory Commission, Office of Nuclear Reactor Regulation, during April 1975.

If the number of fish collected during a 24-hour period exceeds 20,000, notification shall be made by telephone or telegraph within 24 hours to the Director of the Regional NRC Office.

BASES

The program described in the Specification was developed to determine the magnitude of fish impingement at Nine Mile Point Unit 1.

Radiological Environmental MonitoringOBJECTIVE

An environmental radiological monitoring program shall be conducted to evaluate the effects of Station operation on the environs and to verify the effectiveness of the controls on radioactive materials sources.

SPECIFICATION

An environmental radiological monitoring program shall be conducted as follows:

- a. The environmental radiation monitoring program specified in Table 3.2 shall be conducted. Variations from the frequency and location of samples are permitted if due to sample unavailability or seasonal conditions.
- b. Reporting requirements for the environmental radiological monitoring program are outlined in Section 5.6.
- c. During the seasons that animals producing milk for human consumption are on pasture at locations that may be significantly affected\* by emissions from Nine Mile Point-1, samples of fresh milk shall be obtained monthly. For those animals on pasture for which the milk chain dose has been calculated to exceed 15 mrem/year, sampling shall be done weekly. Samples shall be analyzed for their radioiodine content, calculated as I-131. Analysis shall be carried out within eight days (one I-131 half-life) of sampling. Suitable analytical procedures shall be used to determine the radioiodine concentration to a sensitivity of 0.5 picocuries per liter of milk at the time of sampling. For activity levels at or above 0.5 picocuries per liter, overall error (one sigma confidence level) of the analysis shall be within  $\pm 25$  percent. Results shall be reported with associated calculated error, as picocuries of I-131 per liter of milk at the time of sampling.

Special attention shall be paid to those locations where milk is produced for direct consumption by humans; e.g., the family farm.

- d. A census shall be conducted twice annually, (during the beginning and midpoint of the grazing season) to determine the location of cows in potentially affected areas.

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\* For the purposes of this requirement, "Significantly affected" means that calculations, using standard NRC staff assumptions, predict that a two year old child drinking milk produced by animals at that location may receive a thyroid dose of 1 mrem/year or greater.

If it is learned from this census that animals producing milk for human consumption are present at a location which yields a calculated infant thyroid dose greater than from previously sampled animals, the new location shall be added to the surveillance program as soon as practicable. The sampling location having the lowest calculated dose may then be dropped from the surveillance program at the end of the grazing season during which the census was conducted. Also, any location from which milk can no longer be obtained may be dropped from the surveillance program after notifying the Nuclear Regulatory Commission in writing that animals producing milk for human consumption are no longer present at that location.

#### BASES

The Monitoring Requirements given under 2.4.2 and 2.4.4 provide assurance that liquid and gaseous wastes are properly controlled and monitored during releases of radioactive materials in liquid and gaseous effluents. These Monitoring Requirements provide data for the licensee and the Commission to evaluate the Station's performance relative to radioactive liquid and gaseous wastes released to the environment. Reports on the quantities of radioactive materials released in liquid and gaseous effluents shall be furnished to the Commission in conformance with 10 CFR 50.36(a)(2). On the basis of such reports, and any additional information the Commission may obtain from the licensee or others, the Commission may from time to time require the licensee to take such action as the Commission deems appropriate.

The number and distribution of sampling locations and the various types of measurements described in Table 3.2 together with the preoperational background data, will provide verification of the effectiveness of Station effluent control and indication of measurable changes in the activity of the environment.

A concentration of I-131 in milk of 2.4 picocuries per liter will result in a dose to the thyroid of a 0-2 year old child of 15 mrem/year, based upon consumption of one liter per day for the year. To assure that no child will receive a dose of greater than 15 mrem/year to the thyroid, it is necessary to know the radioiodine concentration in the milk to the sensitivity given above, 0.5 pCi/liter.

Ground water sampling is not required because ground water in the vicinity of the station flows north to the lake, away from any nearby wells.<sup>4</sup>

## 5.0 Administrative Controls

### OBJECTIVE

Administrative controls for implementation of the Environmental Technical Specifications are the means by which environmental protection is subject to Station management control and independent review and audit. These measures ensure that the Environmental Technical Specifications will be properly implemented.

## 5.1 Responsibility

5.1.1 The General Superintendent-Nuclear Generation shall have direct responsibility for assuring that operation of all generating units at the site is conducted in such a manner as to provide continuing protection to the environment. During periods when the General Superintendent is unavailable, he may delegate his responsibilities to either the Station Superintendent Nine Mile Point Unit 1 or the Plant Superintendent James A. Fitzpatrick Nuclear Power Plant.

5.1.2 Operation of the Station in compliance with the Environmental Technical Specifications is the responsibility of the Station Superintendent with the assistance of the Station staff organization.

5.1.3 The structure of corporate responsibility is shown in Figure 6.2-1 of Appendix A.

## 5.2 Organization

5.2.1 The Station organization and its relationship to the site organization is shown in Figure 6.2-2 of Appendix A.

5.2.2 Environmental monitoring will be performed by site technical personnel and when requested by environmental consultant personnel. Engineers from the corporate staff will be available for technical assistance when required.

## 5.3 Review and Audit

5.3.1 Units for review and audit of environmental matters shall be organized as shown in Figure 5.1. In addition to the regular members of the Site Operations Review Committee, a responsible supervisor from each consultant organization performing environmental monitoring shall participate in Committee meetings as required.

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5.3.2 The responsibilities and authorities of the environmental review and audit units are shown in Table 5.3.

5.4 Action to be Taken if a Limiting Condition for Operation is Exceeded

5.4.1 Remedial action as permitted by the Technical Specifications shall be taken until the Limiting Condition can be met.

5.4.2 Exceeding a Limiting Condition for Operation shall be promptly reviewed as specified in Section 5.3.

5.4.3 As specified in Section 5.6.3, a separate report for each occurrence shall be prepared and submitted to the Nuclear Regulatory Commission. This report shall include an evaluation of the cause of the occurrence, a record of the corrective action taken, and recommendations for appropriate action to prevent or reduce the probability of a recurrence.

5.5 Operating Procedures

5.5.1 Detailed written procedures, including applicable checkoff lists and instructions, shall be prepared, approved as specified in Section 5.5.2 and adhered to for operation of all systems and components involved in carrying out the environmental monitoring program. Procedures shall include sampling, instrument calibration, analysis, and actions to be taken when specified limits or report levels are approached or exceeded. Calibration frequencies for instruments used in performing the measurements required by the environmental Technical Specifications shall be included. Testing frequency of any alarms shall be included. These frequencies shall be determined from experience with similar instruments in similar environments and from manufacturers' technical manuals.

5.5.2 Prior to implementation, all procedures described in 5.5.1 above, and changes hereto, shall be reviewed as provided in Section 5.3 and approved by the Station Superintendent. Temporary changes to procedures which do not change the intent of the original procedure may be made, provided such changes are approved by the Station Superintendent and one of the following Supervisors: Assistant Radiochemistry and Radiation Protection Supervisor, Radiochemistry and Radiation Protection Supervisor, or Result Supervisor. Such changes shall be documented, subsequently reviewed as provided in Section 5.3 and approved on a timely basis.

TABLE 5.3

RESPONSIBILITIES AND AUTHORITY FOR ENVIRONMENTAL REVIEW ORGANIZATION

Site Operations Review Committee

Safety Review and Audit Board

Vice President-Engineering  
Vice President-Electric Operations

RESPONSIBILITIES

Review results of environmental monitoring programs prior to submittal in the annual Environmental Operating Report.

Review proposed changes to the environmental technical specifications and the evaluated impact of the change.

Review proposed changes or modifications to the station systems or equipment and the evaluated impact which would require a change in the procedures or which would affect the evaluation of the station's environmental impact.

Review the environmental technical specification development with the safety technical specifications to avoid conflicts and for consistency.

Review all proposed procedures or changes which as determined by the station Superintendent may affect the station's environmental impact.

Review proposed environmental technical specification changes or license amendments. Submit analysis to the Vice President-Engineering and Vice President-Electric Operations for the submittal to the NRC.

The function of the Vice President-Engineering and the Vice President-Electric Operations are identical for environmental matters with those described for safety in the Technical Specifications of Appendix A.

## Station Reporting Requirements

### 5.6.1 Annual Environmental Operating Report

A report on the environmental surveillance programs for the previous 12 months of operation shall be submitted to the Director of the NRC Regional Office (with a copy to the Director, Office of Nuclear Reactor Regulation, as a separate document within 60 days of January 1 of each year. In the event that some results are not available within the 60-day period, the report shall be submitted noting and explaining the reasons for the missing results. The missing data shall be submitted as soon as possible in a supplementary report. In addition, results of all radiological samples taken shall be summarized on a quarterly basis following the format of Table 5.6 for inclusion in the annual report.

These reports shall include the following:

- a. Records of monitoring the required surveys and samples.
- b. Analysis of environmental data.
- c. Records of changes in survey procedures.
- d. Records of any special environmental studies not required by the Environmental Technical Specifications.
- e. Records of changes as described in Section 5.6.3.2.1 and 5.6.3.2.2.

If statistically significant variations of offsite environmental radionuclide concentrations are observed, a comparison of these results with effluent releases will be provided.

Individual samples which show higher than normal levels (25 percent above background for external dose, or twice background for radionuclide content) should be noted in the reports.

### 5.6.2 Radioactive Effluent Release Report

A report on the radioactive discharges released from the site during the previous 6 months of operation shall be submitted to the Director of the NRC Regional Office (with a copy to the Director, Office of Nuclear Reactor Regulation) within 60 days after January 1 and July 1 of each year. The report shall include a summary of the quantities of radioactive liquid and gaseous effluents and solid waste released from the plant as outlined in Regulatory Guide 1.21, "Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants".

### 5.6.3 Non-Routine Reports

#### 5.6.3.1 Violations

In the event that a Specification limit or a report level is exceeded, or other violations of the Technical Specifications occur, a report shall be made within 24 hours by telephone, telegraph, or facsimile transmission to the Director of the NRC Regional Office, followed by a written report within 10 days to the Director of the Regional NRC Office (with a copy to the Director, Office of Nuclear Reactor Regulation).

- \* Telegraph notification may be sent on the next working day in the event of a violation during a weekend or holiday period.

### 5.6.3.1 Violations (Cont'd)

To the extent possible, the written report and the preliminary telephone and telegraph report should:

- (a) describe, analyze and evaluate implications,
- (b) determine the cause of the occurrence and
- (c) indicate the corrective action (including any significant changes made in the procedures) taken to preclude repetition of the occurrence and to prevent similar occurrences involving similar components or systems.

The following conditions will be considered as violations unless otherwise specified by a particular specification:

- a. The occurrence of any condition in violation of an Environmental Technical Specification.
- b. If levels of Iodine-131 in the air-milk pathway indicate that the resultant annual dose to an individual from these levels could equal or exceed 120 mrem, a plan will be submitted within one week advising the NRC of the proposed action to ensure the plant-related annual doses will be within the design objective of 15 mrem/yr.

For example, with an I-131 design objective of 15 mrem/yr to the thyroid of any individual, if individual milk samples show I-131 concentrations of 19 pCi/l or greater, the results will be reported along with a proposed plan of action, as discussed above.

- c. If samples of the air-milk environmental pathway collected over a calendar quarter show levels of I-131 that could result in accumulated Station-related doses to the thyroid of an individual of 7.5 mrem for that quarter, the results shall be reported and a plan submitted and implemented within 30 days to limit conditions so that the annual dose to the thyroid of an individual will not exceed 15 mrem.

### 5.6.3.2 Changes

- 5.6.3.2.1 When a change to the Station (that affects the environmental impact evaluation contained in the Environmental Report or the Environmental Statement) or to the environmental monitoring procedures or equipment is planned, a report of the change shall be submitted to the NRC for information prior to implementation of the

### 5.6.3.2 Changes (Cont'd)

change. This is not intended to preclude making changes on short notice that are significant in terms of decreasing adverse environmental impact, etc. However, these changes will be promptly reported. The report will include an evaluation of the impact of the change.

5.6.3.2.2 Changes or additions to permits and certifications required by Federal, State, local and regional authorities for the protection of the environment will be reported. When the required changes are submitted to the concerned agency for approval, they will also be submitted to the Nuclear Regulatory Commission for information. The submittal should include an evaluation of the environmental impact associated with the change.

5.6.3.2.3 Request for changes in Environmental Technical Specifications shall be submitted to the Director, Division of Operating Reactors, Nuclear Regulatory Commission for prior review and authorization. The request will include an evaluation of the environmental impact associated with the change.

## 5.7 Records Retention

5.7.1 Records and logs relative to Specifications contained in Section 5.0 of the Environmental Technical Specifications shall be retained for five years except as described in 5.7.2.

5.7.2 All records and logs relative to the following areas shall be retained for the life of the Station:

a. Records and drawing changes reflecting Station design modifications made to systems and equipment as described in Section 5.6.2.2.

b. Records of environmental monitoring data.

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-220

NIAGARA MOHAWK POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 7 to Facility Operating License No. DPR-63 issued to Niagara Mohawk Power Company which revised Technical Specifications for operation of the Nine Mile Point Unit 1, located in Oswego County, New York. The amendment is effective as of its date of issuance.

The amendment revises the provisions in the Environmental Technical Specifications relating to report requirements. In addition, editorial corrections and changes have been made to reflect the reorganization of the Atomic Energy Commission to form the Nuclear Regulatory Commission.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental statement, negative declaration or

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environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated August 11, 1975 and (2) Amendment No. 7 to License No. DPR-63. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street N.W., Washington, D.C. and at the Oswego City Library, 120 E. Second Street, Oswego, New York 13126.

A copy of item (2) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director Division of Operating Reactors.

Dated at Bethesda, Maryland, this            day of

FOR THE NUCLEAR REGULATORY COMMISSION

George Lear, Chief  
Operating Reactors Branch #3  
Division of Operating Reactors

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SURNAME >	CParrish:acr	GLear				
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