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Docket No. 50-220

Niagara Mohawk Power Corporation
 ATTN: Mr. Gerald K. Rhode
 Vice President - Engineering
 300 Erie Boulevard West
 Syracuse, New York 13202

Gentlemen:

The Commission has issued the enclosed Amendment No. 10 to Facility License No. DPR-63 for Unit No. 1 of the Nine Mile Point Nuclear Station. This amendment consists of changes to the Technical Specifications and is in accordance with your application. dated February 19, 1976.

This amendment will revise the shock suppressor surveillance and operability requirements to conform with the current NRC standard Technical Specifications.

Copies of the Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

George Lear, Chief
 Operating Reactors Branch #3
 Division of Operating Reactors

Enclosures:

1. Amendment No. 10
2. Safety Evaluation
3. Federal Register Notice

9

OFFICE	ORB#3	ORB#3 <i>209</i>	OELD <i>[Signature]</i>	ORB#3	
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 10
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated February 19, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, appearing to read "George Lear".

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the
Technical Specifications

Date of Issuance: July 8, 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 10

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Replace pages 241a through 241j with the attached revised pages.

LIMITING CONDITION FOR OPERATION

SURVEILLANCE REQUIREMENT

3.6.4 Shock Suppressors (Snubbers)

Applicability

) Applies to the operational status of shock suppressors (snubbers).

Objective

To assure the capability of the snubbers to:

Prevent unrestrained pipe motion under dynamic loads as might occur during an earthquake or severe transient, and

Allow normal thermal motion during startup and shutdown.

) Specification

- a. During all modes of operation except Cold Shutdown and Refuel, all snubbers which are required

4.6.4 Shock Suppressors (Snubbers)

Applicability

) Applies to the periodic testing requirement for shock suppressors (snubbers).

Objective

To assure the operability of the snubbers to perform their intended functions.

Specification

The following surveillance requirements apply to all hydraulic snubbers listed in Table 3.6.4.

- a. All hydraulic snubbers whose seal material has been demonstrated by operating experience, lab testing or analysis to be compatible with the normal operating environment (<150F) shall be visually inspected. This inspection shall include but not necessarily be limited to inspection of the hydraulic fluid

LIMITING CONDITION FOR OPERATION

to protect the primary coolant system or any other safety related system or component shall be operable except as noted in 3.6.4.b through 3.6.4.d below. These safety related snubbers are listed in Table 3.6.4

SURVEILLANCE REQUIREMENT

reservoir, fluid connections and linkage connections to the piping and anchor to verify snubber operability in accordance with the following schedule:

Number of Snubbers Found Inoperable During Inspection or During Inspection Interval	Next Required Inspection Interval
0	18 months ± 25%
1	12 months ± 25%
2	6 months ± 25%
3,4	124 days ± 25%
5,6,7	62 days ± 25%
≥ 8	31 days ± 25%

The required inspection interval shall not be lengthened more than one step at a time.

Snubbers may be categorized in two groups, "accessible" or "inaccessible" based on their accessibility for inspection during reactor operation. These two groups may be inspected independently according to the above schedule.

LIMITING CONDITION FOR OPERATION

- b. From and after the time that a snubber is determined to be inoperable, continued reactor operation is permissible only during the succeeding 72 hours unless the snubber is sooner made operable or replaced.
- c. If the requirements of 3.6.4.a and 3.6.4.b cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a cold shutdown condition within 36 hours.
- d. If a snubber is determined to be inoperable while the reactor is in the shutdown or refuel mode, the snubber shall be made operable or replaced prior to reactor startup.
- e. Snubbers may be added to safety related systems without prior License Amendment to Table 3.6.4 provided that safety evaluations, documentation and reporting are provided in accordance with 10 CFR 50.59, and that a revision to Table 3.6.4 is included with a subsequent License Amendment request.

SURVEILLANCE REQUIREMENT

- b. All hydraulic snubbers whose seal materials are other than ethylene propylene or other material that has been demonstrated to be compatible with the normal operating environment ($\leq 150^{\circ}\text{F}$) shall be visually inspected for operability every 31 days.
- c. Once each refueling cycle, a representative sample of 10 hydraulic snubbers or approximately 10% of the hydraulic snubbers, whichever is less, shall be functionally tested for operability including verification of proper piston movement and lock up. For each unit and subsequent unit found inoperable, an additional 10% or ten hydraulic snubbers shall be so tested until no more failures are found or all units have been tested. Snubbers of rated capacity greater than 50,000 lb. need not be functionally tested.

TABLE 3.6.4

SAFETY RELATED SHOCK SUPPRESSORS (SNUBBERS)

Snubber No.	Location	Elevation	Snubbers In High Radiation Area During Shutdown (1)	Especially Hard To Remove Snubbers	Inaccessible Snubbers	Accessible Snubbers
1-HS-1	Main Steam	264'-0 1/2"			X	
1-HS-2	Main Steam	264'-0 1/2"			X	
1-HS-3	Main Steam	264'-0 1/2"			X	
1-HS-4	Main Steam	264'-0 1/2"			X	
1-HS-4	Main Steam	264'-0 1/2"			X	
1-HS-5	Main Steam	264'-0 1/2"			X	
1-HS-6	Main Steam	264'-0 1/2"			X	
1-HS-7	Main Steam	264'-0 1/2"			X	
1-HS-8	Main Steam	264'-0 1/2"			X	
9-HS-1	Feedwater	279'-0"				X
9-HS-2	Feedwater	301'-0"				X
9-HS-3	Feedwater	301'-0"				X
9-HS-4	Feedwater	301'-0"				X
9-HS-5	Feedwater	301'-0"				X
9-HS-6	Feedwater	301'-0"				X
9-HS-7	Feedwater	301'-0"				X

TABLE 3.6.4

SAFETY RELATED SHOCK SUPPRESSORS (SNUBBERS)

Snubber No.	Location	Elevation	Snubbers In High Radiation Area During Shutdown (1)	Especially Hard To Remove Snubbers	Inaccessible Snubbers	Accessible Snubbers
29-HS-8	Feedwater	305'-6"				X
29-HS-9	Feedwater	305'-6"				X
29-HS-10	Feedwater	305'-6"				X
29-HS-11	Feedwater	301'-0"				X
29-HS-12	Feedwater	291'-6"				X
29-HS-13	Feedwater	291'-6"				X
29-HS-14	Feedwater	291'-6"				X
29-HS-15	Feedwater	291'-6"				X
29-HS-16	Feedwater	303'-9"				X
29-HS-17	Feedwater	303'-9"				X
30-HS-1	Feedwater	325'-6"				X
30-HS-2	Feedwater	325'-6"				X
30-HS-3	Feedwater	325'-6"				X
30-HS-4	Feedwater	325'-6"				X
30-F	Feedwater	325'-6"				X
31-HS-1	Feedwater	263'-6 1/2"			X	
31-HS-2	Feedwater	263'-6 1/2"			X	
32-HS-1	Reactor Recirculation	230'-2 5/8"			X	
32-HS-2	Reactor Recirculation	230'-2 5/8"			X	
32-HS-3	Reactor Recirculation	245'-5 3/8"			X	
32-HS-4	Reactor Recirculation	245'-5 3/8"			X	
32-HS-5	Reactor Recirculation	225'-6"			X	
32-HS-6	Reactor Recirculation	230'-2 5/8"			X	
32-HS-7	Reactor Recirculation	230'-2 5/8"			X	
32-HS-8	Reactor Recirculation	245'-5 3/8"			X	
32-HS-9	Reactor Recirculation	245'-5 3/8"			X	
32-HS-10	Reactor Recirculation	225'-6"			X	
32-HS-11	Reactor Recirculation	230'-2 5/8"			X	

TABLE 3.6.4

SAFETY RELATED SHOCK SUPPRESSORS (SNUBBERS)

Snubber No.	Location	Elevation	Snubbers In High Radiation Area During Shutdown (1)	Especially Hard To Remove Snubbers	Inaccessible Snubbers	Accessible Snubbers
32-HS-12	Reactor Recirculation	230'-2 5/8"			X	
32-HS-13	Reactor Recirculation	245'-5 3/8"			X	
32-H ^c 14	Reactor Recirculation	245'-5 3/8"			X	
32-I. 15	Reactor Recirculation	225'-6"			X	
32-HS-16	Reactor Recirculation	230'-2 5/8"			X	
32-HS-17	Reactor Recirculation	230'-2 5/8"			X	
32-HS-18	Reactor Recirculation	245'-5 3/8"			X	
32-HS-19	Reactor Recirculation	245'-5 3/8"			X	
32-HS-20	Reactor Recirculation	225'-6"			X	
32-HS-21	Reactor Recirculation	230'-2 5/8"			X	
32-HS-22	Reactor Recirculation	230'-2 5/8"			X	
32-HS-23	Reactor Recirculation	245'-5 3/8"			X	
32-HS-24	Reactor Recirculation	245'-5 3/8"			X	
32-HS-25	Reactor Recirculation	225'-6"			X	
33-HS-1	Cleanup	263'-6 1/2"			X	
33-HS-2	Cleanup	263'-6 1/2"			X	
33-HS-3	Cleanup	263'-6 1/2"			X	
33-H ^c 4	Cleanup	263'-6 1/2"			X	
35-HS-1	Seal Leak Detection	261'-0"			X	
37-HS-1	Reactor Vent and Drain	263'-6"			X	
38-HS-1	Shutdown Cooling	260'-10 5/8"			X	
38-HS-2	Shutdown Cooling	260'-10 5/8"			X	
38-HS-3	Shutdown Cooling	269'-3"			X	
38-HS-4	Shutdown Cooling	268'-1"				X
38-HS-5	Shutdown Cooling	268'-1"				X
38-HS-6	Shutdown Cooling	268'-1"				X
38-HS-7	Shutdown Cooling	271'-4"				X
38-HS-8	Shutdown Cooling	270'-10"				X

TABLE 3.6.4

SAFETY RELATED SHOCK SUPPRESSORS (SNUBBERS)

Snubber No.	Location	Elevation	Snubbers In High Radiation Area During Shutdown (1)	Especially Hard To Remove Snubbers	Inaccessible Snubbers	Accessible Snubbers
39-H)	Emergency Cooling	333'-0"				X
39-HS-2	Emergency Cooling	333'-0"				X
39-HS-3	Emergency Cooling	318'-0"				X
39-HS-4	Emergency Cooling	318'-0"				X
39-HS-5	Emergency Cooling	318'-0"				X
39-HS-6	Emergency Cooling	318'-0"				X
39-HS-7	Emergency Cooling	305'-9"				X
39-HS-8	Emergency Cooling	305'-9"				
39-HS-9	Emergency Cooling	269'-3"			X	
39-HS-10	Emergency Cooling	269'-3"			X	
39-HS-11	Emergency Cooling	269'-3"			X	
39-HS-12	Emergency Cooling	269'-3"			X	
39-HS-13	Emergency Cooling	308'-2 3/4"				X
39-HS-14	Emergency Cooling	315'-0"				X
39-HS-15	Emergency Cooling	315'-0"				X
39-HS-16	Emergency Cooling	325'-10 1/2"				X
39-H)7	Emergency Cooling	334'-6"				X
39-HS-18	Emergency Cooling	334'-6"				X
39-HS-19	Emergency Cooling	334'-6"				X
39-HS-20	Emergency Cooling	341'-3"				X
39-HS-21	Emergency Cooling	341'-3"				X
39-HS-22	Emergency Cooling	341'-3"				X
39-HS-23	Emergency Cooling	341'-3"				X
39-HS-24	Emergency Cooling	308'-4 3/4"				X
39-HS-25	Emergency Cooling	315'-0"				X
39-HS-26	Emergency Cooling	315'-0"				X
39-HS-27	Emergency Cooling	325'-10 1/2"				X
39-HS-28	Emergency Cooling	334'-6"				X
39-HS-29	Emergency Cooling	334'-6"				X
39-HS-30	Emergency Cooling	334'-6"				X
39-HS-31	Emergency Cooling	341'-3"				X

TABLE 3.6.4

SAFETY RELATED SHOCK SUPPRESSORS (SNUBBERS)

Snubber No.	Location	Elevation	Snubbers In High Radiation Area During Shutdown (1)	Especially Hard To Remove Snubbers	Inaccessible Snubbers	Accessible Snubbers
39-H 12	Emergency Cooling	341'-3"				X
39-HS-33	Emergency Cooling	341'-3"				X
39-HS-34	Emergency Cooling	341'-3"				X
40-HS-1	Core Spray	240'-0"			X	
40-HS-2	Core Spray	240'-0"			X	
40-HS-3	Core Spray	263'-10"			X	
40-HS-4	Core Spray	261'-6"			X	
40-HS-5	Core Spray	261'-6"			X	
40-HS-6	Core Spray	261'-6"			X	
40-HS-7	Core Spray	248'-0"			X	
40-HS-8	Core Spray	248'-0"			X	
40-HS-9	Core Spray	248'-0"			X	
40-HS-10	Core Spray	272'-5"			X	
40-HS-11	Core Spray	261'-6"			X	
51-H 1	Feedwater	301'-0"				X
51-HS-2	Feedwater	313'-6"				X
51-HS-3	Feedwater	313'-6"				X
51-HS-4	Feedwater	313'-6"				X
51-HS-5	Feedwater	291'-0"				X
51-HS-6	Feedwater	291'-0"				X
51-HS-7	Feedwater	291'-0"				X
51-HS-8	Feedwater	291'-0"				X
51-HS-10	Feedwater	291'-6"				X
51-HS-11	Feedwater	291'-6"				X
51-HS-12	Feedwater	291'-6"				X

TABLE 3.6.4

SAFETY RELATED SHOCK SUPPRESSORS (SNUBBERS)

Snubber No.	Location	Elevation	Snubbers In High Radiation Area During Shutdown (1)	Especially Hard To Remove Snubbers	Inaccessible Snubbers	Accessible Snubbers
66-HS-1	Main Steam	232'-6"			X	
66-HS-2	Main Steam	232'-3"			X	
66-HS-3	Main Steam	231'-9"			X	
66-HS-4	Main Steam	232'-6"			X	
66-HS-5	Main Steam	232'-3"			X	
66-HS-6	Main Steam	261'-9"			X	
66-HS-7	Main Steam	232'-6"			X	
66-HS-8	Main Steam	232'-3"			X	
66-HS-9	Main Steam	231'-9"			X	
66-HS-10	Main Steam	232'-6"			X	
66-HS-11	Main Steam	232'-3"			X	
66-HS-12	Main Steam	231'-9"			X	
66-HS-13	Main Steam	232'-6"			X	
66-HS-14	Main Steam	232'-3"			X	
66-HS-15	Main Steam	231'-9"			X	
66-HS-16	Main Steam	232'-6"			X	
66-HS-17	Main Steam	232'-3"			X	
66-HS-18	Main Steam	231'-9"			X	
66-HS-19	Main Steam	253'-0"			X	
66-HS-20	Main Steam	253'-0"			X	
66-HS-21	Main Steam	265'-3"			X	

NOTES:

1. Modifications to this Table due to changes in high radiation areas should be submitted to the NRC as part of the next license amendment.

Snubbers are designed to prevent unrestrained pipe motion under dynamic loads as might occur during an earthquake or severe transient, while allowing normal thermal motion during startup and shutdown. The consequence of an inoperable snubber is an increase in the probability of structural damage to piping as a result of a seismic or other event initiating dynamic loads. It is therefore required that all hydraulic snubbers required to protect the primary coolant system or any other safety system or component be operable during reactor operation.

Because the snubber protection is required only during relatively low probability events, a period of 72 hours is allowed for repairs or replacements. In case a shutdown is required, the allowance of 36 hours to reach a cold shutdown condition will permit an orderly shutdown consistent with standard operating procedures. Since plant startup should not commence with knowingly defective safety related equipment, Specification 3.6.4.d prohibits startup with inoperable snubbers.

All safety related hydraulic snubbers are visually inspected for overall integrity and operability. The inspection shall include verification of proper orientation, adequate hydraulic fluid level and proper attachment of snubber to piping (and structures).

The inspection frequency is based upon maintaining a constant level of snubber protection. Thus the required inspection interval varies inversely with the observed snubber failures. The number of inoperable snubbers found during a required inspection determines the time interval for the next required inspection. Inspections performed before that interval has elapsed shall be used as a new reference point to determine the next inspection. However, the results of such early inspections performed before the original required time interval has elapsed (nominal time less 25%) shall not be used to lengthen the required inspection interval. Any inspection whose results require a shorter inspection interval will override the previous schedule.

Experience at our operating facility has shown that the required surveillance program should assure an acceptable level of snubber performance because the seal materials have demonstrated their compatibility with the normal operating environment. To date, six years operating experience, has resulted in no seal failures. Also, laboratory test data are available on seal material for all snubbers installed as of September 1975. These data show their compatibility with the normal operating environment.

To increase the assurance of snubber reliability, functional tests shall be performed once each refueling cycle. These tests will include stroking of the snubbers to verify proper piston movement and lock-up. Ten percent or ten snubbers, whichever is less, represents an adequate sample for such tests. Observed failures on these samples should require testing of additional units. Those snubbers designated in Table 3.6.4 as being in high radiation areas or those especially difficult to remove need not be selected for functional tests provided operability was previously verified. Snubbers of rated capacity greater than 50,000 lb. are exempt from the functional testing requirements because of the impracticality of testing such large units.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 10 TO FACILITY OPERATING LICENSE NO. DPR-63

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT UNIT NO. 1

DOCKET NO. 50-220

Introduction

By letter dated February 19, 1976, Niagara Mohawk Power Corporation (NMPC) submitted an application for amendment to Facility Operating License No. DPR-63 for Nine Mile Point Unit No. 1 (NMP-1). The proposed amendment involves changes to the Technical Specification requirements related to shock suppressors (snubbers). The changes would:

1. Establish simplified administrative procedures for adding new snubbers to safety-related systems;
2. Clarify the requirements for periodic visual inspections of hydraulic snubbers; and
3. Delete the requirement for periodic disassembly of snubbers for inspection purposes.

Selected modifications were made to the original application with the mutual approval of the NRC staff and NMPC, in order to assure that the format of the Technical Specification changes is consistent with the current standard licensing format.

Discussion

On November 18, 1974, NMPC proposed Technical Specifications for hydraulic snubbers at NMP-1. Concurrent with our review of NMPC's November 18, 1974 submittal, the NRC staff developed model technical specifications which provide additional assurance of satisfactory snubber operation. The model technical specifications reflect the accumulated industry-wide experience with respect to snubber performance and reliability. By letter dated July 25, 1975, we sent NMPC a copy of the model technical specifications for hydraulic snubbers and requested that NMPC adapt the model to NMP-1. On September 17, 1975, NMPC submitted an application for a license amendment to incorporate the model technical specifications for hydraulic snubbers into Appendix A of Facility Operating License No. DPR-63 for NMP-1. On October 24, 1975, the NRC issued License Amendment No. 3 to Facility

Operating License No. DPR-63 which incorporated hydraulic snubber surveillance and operability requirements into the NMP-1 Technical Specifications. Subsequently, as a result of industry comments and further consideration by the NRC staff, we revised the model Technical Specifications for shock suppressors in order to provide clarification of the surveillance and operability requirements and to include mechanical shock suppressors. By letter dated December 24, 1975, we requested NMPC to submit an application for license amendment which would reflect the changes to the model technical specifications. NMPC's application of February 19, 1976 was responsive to our request.

Evaluation

The proposed changes in the Technical Specifications establish a procedure whereby additional safety related snubbers may be installed in the plant without prior NRC approval, provided that such additions are included, for completeness in administering the surveillance program, in a subsequent request for license amendment. It is anticipated that the addition of new snubbers to existing systems would not (1) increase the probability of occurrence or consequences of an accident previously evaluated in the NMP-1 safety analysis report, (2) create the possibility of an accident or malfunction of a different type than previously evaluated in the NMP-1 safety analysis report, or (3) reduce the margin of safety as defined in the bases for the NMP-1 Technical Specifications. Therefore, pursuant to 10 CFR Part 50, §50.59, such additions would not require prior NRC approval. The proposed changes also include a procedure for updating the "Table of Safety Related Snubbers" in the Technical Specifications without requiring a separate license amendment prepared specifically for this purpose; instead, such changes may be incorporated with any subsequent request for license amendment.

The requirements for visual inspections of hydraulic snubbers have been clarified by:

1. Specifying the snubber components which must be inspected; and
2. Identifying snubbers which are: (a) located in high radiation areas during shutdown, (b) especially difficult to remove, (c) inaccessible during normal operation, and (d) accessible during normal operation.

These changes serve to: (1) assure the quality of the snubbers inspections, and (2) identify which snubbers may be exempted from functional testing requirements.

The requirement to disassemble two snubbers per refueling cycle has been deleted due to the high probability of damaging the snubbers during such an inspection. Functional testing and visual inspection requirements provide adequate surveillance to assure continued operability of snubbers.

Summary

The proposed Technical Specification changes are essentially administrative in nature and represent clarifications and/or improvements to existing requirements.

Environmental Aspects

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §1.5(d)(4), that an environmental statement, negative declaration, or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: July 8, 1976

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-220

NIAGARA MOHAWK POWER CORPORATION

NOTICE OF ISSUANCE OF FACILITY LICENSE AMENDMENT

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 10 to Facility Operating License No. DPR-63 to the Niagara Mohawk Power Corporation (the licensee) which revised Technical Specifications for operation of the Nine Mile Point Nuclear Station, Unit No. 1 (the facility) located in Oswego County, New York. The amendment is effective as of its date of issuance.

The amendment will revise the shock suppressor surveillance and operability requirements to conform with the current NRC standard Technical Specifications.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental statement, negative declaration or environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated February 19, 1976, (2) Amendment No. 10 to License No. DPR-63, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street N. W., Washington, D. C. and at the Oswego City Library, 120 E. Second Street, Oswego, New York 13126.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 8 day of July 1976.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors