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APR 13 1981

Docket No. 50-220

April 13, 1981

Mr. Donald P. Dise
Vice President - Engineering
c/o Miss Catherine R. Seibert
Niagara Mohawk Power Corporation
300 Erie Boulevard West
Syracuse, New York 13202

Dear Mr. Dise:

The Commission has issued the enclosed Amendment No. 42 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Power Station Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application dated September 17, 1980.

These changes to the Technical Specifications as agreed to by members of your staff involve incorporation of certain of the TMI-2 Lessons Learned Category "A" requirements. These requirements are related to: (1) Emergency Power Supply/Inadequate Core Cooling, (2) Valve Position Indication, (3) Containment Isolation, (4) Shift Technical Advisors, (5) Integrity of Systems Outside Containment, and (6) Iodine Monitoring.

Copies of the Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

Original Signed by
T. A. Ippolito

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

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U.S. NUCLEAR REGULATORY COMMISSION

Enclosures:
1. Amendment No. 42 to DPR-63
2. Safety Evaluation
3. Notice

cc w/encls:
See next page

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Comments as to form of Amendment and FR notice only as noted and is subject to correction as noted

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SURNAME	SNorris	PPolk:ms	TAIppolito	TMNovak	[illegible]		
DATE	3/31/81	3/31/81	4/1/81	4/1/81	4/7/81		

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

April 13, 1981

Docket No. 50-220

Mr. Donald P. Dise
Vice President - Engineering
c/o Miss Catherine R. Seibert
Niagara Mohawk Power Corporation
300 Erie Boulevard West
Syracuse, New York 13202

Dear Mr. Dise:

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These changes to the Technical Specifications as agreed to by members of your staff involve incorporation of certain of the TMI-2 Lessons Learned Category "A" requirements. These requirements are related to: (1) Emergency Power Supply/Inadequate Core Cooling, (2) Valve Position Indication, (3) Containment Isolation, (4) Shift Technical Advisors, (5) Integrity of Systems Outside Containment, and (6) Iodine Monitoring.

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Sincerely,

TAIppolito

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Enclosures:
1. Amendment No. 42 to DPR-63
2. Safety Evaluation
3. Notice

cc w/encls:
See next page

Mr. Donald P. Dise
Niagara Mohawk Power Corporation

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Mr. Robert P. Jones, Supervisor
Town of Scriba
R. D. #4
Oswego, New York 13126

Niagara Mohawk Power Corporation
ATTN: Mr. Thomas Perkins
Plant Superintendent
Nine Mile Point Plant
300 Erie Boulevard West
Syracuse, New York 13202

Director, Criteria and Standards
Division
Office of Radiation Programs (ANR-460)
U. S. Environmental Protection Agency
Washington, D. C. 20460

U. S. Environmental Protection Agency
Region II Office
ATTN: EIS COORDINATOR
26 Federal Plaza
New York, New York 10007

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Resident Inspector
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P. O. Box 126
Lycoming, New York 13093

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 42
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated September 17, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 42 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 13, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 42

FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise Appendix A as follows:

Remove	Insert
--	iiia
v	v
--	241ee
--	241ff
--	241gg
--	241hh
--	241ii
249	249
264	264
265	--

LIMITING CONDITION FOR OPERATION

3.6.11 ACCIDENT MONITORING INSTRUMENTATION

Applicability:

Applies to the operability of the plant instrumentation that performs an accident monitoring function.

Objective:

To assure high reliability of the accident monitoring instrumentation.

Specification:

a. During the power operating condition, the accident monitoring instrumentation sensors shown in Table 3.6.11 shall be operable except as specified in 3.6.11 b or c.

b. Safety and Relief Valves

(1) With the number of operable accident monitoring instrumentation sensors (for parameters 1 and 2) 1 less than the number shown in Table 3.6.11 restore to an operable status during the next cold shutdown when there is access to the drywell.

(2) With the number of operable accident monitoring instrumentation sensors (for parameters 1 and 2) 2 less than the number shown in Table 3.6.11 restore an inoperable sensor to an operable status within 30 days or be in at least HOT SHUTDOWN within the next 12 hours.

(3) The total number of sensors shown in Table 3.6.11 will be operable prior to the beginning of each cycle.

SURVEILLANCE REQUIREMENT

4.6.11 ACCIDENT MONITORING INSTRUMENTATION

Applicability:

Applies to the surveillance of the instrumentation that performs an accident monitoring function.

Objective:

To verify the operability of accident monitoring instrumentation.

Specification:

Instrument channels shall be tested and calibrated at least as frequently as listed in Table 4.6.11.

3.6.11 ACCIDENT MONITORING INSTRUMENTATION (continued)

Specification: (continued)

c. Reactor Vessel Water Level

(1) With the number of operable accident monitoring instrumentatin sensors less than the total number of sensors (for parameter 3) shown in Table 3.6.11, either restore the inoperable sensor(s) to operable status within 7 days, or be in at least hot shutdown within the next 12 hours.

(2) With the number of operable accident monitoring instrumentation sensors (for parameter 3) less than the minimum number of operable sensors of Table 3.6.11, either restore the inoperable sensor(s) to operable status within 48 hours or be in at least hot shutdown within the next 12 hours.

Table 3.6.11

Accident Monitoring Instrumentation

Parameter	Total Number of Sensors	Minimum Number of Operable Sensors
(1) Relief valve position indicator	2/valve	1/valve
(2) Safety valve position indicator	2/valve	1/valve
(3) Reactor vessel water level	2	1

Table 4.6.11

Accident Monitoring Instrumentation

Surveillance Requirement

Parameter	Instrument Channel Test	Instrument Channel Calibration
(1) Relief valve position indicator (Primary - Acoustic)	Once per month	Once during each major refueling outage
Relief valve position indicator (Backup - Thermocouple))	Once per month	Once during each major refueling outage
(2) Safety valve position indicator (Primary - Acoustic)	Once per month	Once during each major refueling outage
Safety valve position indicator (Backup - Thermocouple)	Once per month	Once during each major refueling outage
(3) Reactor vessel water level	Once per month	Once during each major refueling outage

Amendment No. 42

Accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables during and following an accident. This capability is consistent with the recommendations of NUREG-0578, "TMI-2 Lessons Learned Task Force Status Report and Short-Term Recommendations."

6.3 Facility Staff Qualifications

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 Training

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Training Supervisor and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Training Supervisor and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975 except for Fire Brigade Training sessions which shall be held at least quarterly.

6.5 Review and Audit

6.5.1 Site Operations Review Committee (SORC)

Function

6.5.1.1 The Site Operations Review Committee shall function to advise the General Superintendent Nuclear Generation on all matters related to nuclear safety.

Composition

6.5.1.2 The Site Operations Review Committee shall be composed of the:

Chairman:	General Superintendent - Nuclear Generation
Member:	Station Superintendent - Nuclear Generation
Member:	Superintendent Results - Nuclear
Member:	Supervisor Reactor Analysis
Member:	Superintendent Maintenance - Nuclear
Member:	Supervisor Instrument & Control - Nuclear
Member:	Supervisor Radiochemical & Radiation Protection

6.14 Fire Protection Inspection

6.14.1 An independent fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified off-site licensee personnel or an outside fire protection firm.

6.14.2 An inspection and audit by an outside qualified fire consultant shall be performed at intervals no greater than 3 years.

6.15 Environmental Qualification

A. By no later than June 30, 1982 all safety-related electrical equipment in the facility shall be qualified in accordance with the provisions of Division of Operating Reactors "Guidelines for Evaluating Environmental Qualification of Class IE Electrical Equipment in Operating Reactors" (DOR Guidelines); or, NUREG-0588 "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment", December 1979. Copies of these documents are attached to Order for Modification of License DPR-63 dated October 24, 1980.

B. By no later than December 1, 1980, complete and auditible records must be available and maintained at a central location which describe the environmental qualification method used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with the DOR Guidelines or NUREG-0588. Thereafter, such records should be updated and maintained current as equipment is replaced, further tested, or otherwise further qualified.

6.16 Systems Integrity

Procedure shall be established, implemented and maintained to meet or exceed the requirements and recommendations of section 2.1.6.a of NUREG 0578.

6.17 Iodine Monitoring

Procedures shall be established, implemented and maintained to meet or exceed the requirements and recommendations of section 2.1.8.c of NUREG 0578.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 42 TO FACILITY OPERATING LICENSE NO. DPR-63

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-220

1.0 Introduction

By letter dated September 17, 1980 the Niagara Mohawk Power Corporation (licensee) proposed changes to the Technical Specifications (TS) appended to Facility Operating License No. DPR-63. The changes involve the incorporation of certain of the TMI-2 Lessons Learned Category "A" requirements. The licensee's request is in response to the NRC's letter dated July 2, 1980. These changes are discussed in Sections 2 and 3 of this evaluation.

2.0 Background Information

By our letter dated September 13, 1979 we issued to all operating nuclear power plants requirements established as a result of our review of the Three Mile Island (Unit 2) accident. Certain of these requirements, designated Lessons Learned Category "A", were to have been completed by the licensee prior to any operation subsequent to January 1, 1980. Our evaluation of the licensee's compliance with these Category "A" items was attached to our letter to Niagara Mohawk dated March 21, 1980.

In order to provide reasonable assurance that operating reactor facilities are maintained within the limits determined acceptable following the implementation of the TMI-2 Category "A" requirements, we requested that licensees amend their Technical Specifications to incorporate additional Surveillance Requirements and Limiting Conditions of Operation, as appropriate. This request was transmitted to all licensees on July 2, 1980. Included therein were model specifications that we had determined to be acceptable. The licensee's application, dated September 17, 1980, is in response to our request. Each of the issues identified by the NRC and the licensee's response is discussed in the following evaluation.

3.0 Evaluation

1) Emergency Power Supply/Inadequate Core Cooling

As applicable to Boiling Water Reactors (BWR's), we indicated that water level instrumentation is important to post-accident monitoring and that surveillance of this instrumentation should be performed.

The licensee's response to this item stated that the requirement for returning an inoperable level instrument to operable within 7 days is too restrictive. In lieu of the 7 day corrective action requirement the licensee proposed 15 days based upon the fact that during this period a redundant channel is available. We do not agree with the licensee's finding that 15 days is more appropriate in that 7 days is more than adequate to take the necessary corrective action. The licensee has verbally agreed to the 7 day requirement.

We have reviewed the proposed Technical Specifications for water level instrumentation. The surveillance requirements for instrument checks (once per month) and calibration (once per fuel cycle) meet our guidelines. Based on this, we conclude that the licensee's response satisfies our request.

2) Valve Position Indication

Our requirements for installation of a reliable position indicating system for relief and safety valves was based on the need to provide the operator with a diagnostic aid to reduce the ambiguity between indications that might indicate either an open relief/safety valve or a small line break. Such a system did not need to be safety grade provided that backup methods of determining valve position are available.

The required indication was to be provided to plant operators located in the control room. The staff has found acceptable two methods of satisfying this requirement: (1) Separate audible and visual indication in the control room for each valve, or (2) Use of the control room computer to obtain information for each specific valve. The licensee has agreed verbally to satisfy this requirement.

We have reviewed the proposed Technical Specifications for safety relief valve position indication. The licensee requested that the plant be allowed to continue to operate until the next plant shutdown with inoperable instrumentation. We agree that an extended period of inoperability for one of the two indicators for each valve is acceptable. However, if both indicators for a particular valve are inoperable, repairs should be accomplished within 30 days or an orderly shutdown should be initiated. The licensee has verbally agreed to this requirement which is reflected in the attached Technical Specifications. Based on this, we conclude that the licensee has satisfied this requirement.

3) Containment Isolation

Our request indicated that the specifications should include a Table of Containment Isolation Valves which reflect the diverse isolation signal requirement of this Lessons Learned issue. The licensee response indicated that this requirement is presently covered in the Nine Mile Point Technical Specifications 3.2.7, 4.2.7, 3.4.2 and 4.4.2 and Bases. We have reviewed these Technical Specifications and Bases and conclude that the licensee has adequately responded to this requirement.

4) Shift Technical Advisor (STA)

Our request indicated that the TSs related to minimum shift manning should be revised to reflect the augmentation of an STA. The STA function includes both accident and operating experience assessment. The licensee response proposed TS changes which provide for the Shift Technical Advisor. We have reviewed these changes and conclude that the licensee has satisfied this requirement.

5) Integrity of Systems Outside Containment

Our letter dated July 2, 1980, indicated that the license should be amended by adding a license condition related to a Systems Integrity Measurements Program. Such a condition would require the licensee to effect an appropriate program to eliminate or prevent the release of significant amounts of radioactivity to the environment via leakage from engineered safety systems and auxiliary systems, which are located outside reactor containment.

By letter dated December 31, 1979 the licensee proposed a program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This program includes (1) provisions establishing preventive maintenance and periodic visual inspection requirements, and (2) leak test requirements for each system at a frequency not to exceed refueling cycle intervals. We have reviewed this program and conclude that the licensee has satisfied this requirement. The proposed Technical Specifications will ensure compliance.

6) Iodine Monitoring

Our letter dated July 2, 1980, indicated that the license should be amended by adding a license condition related to iodine monitoring. Such a condition would require the licensee to effect a program which would ensure the capability to determine the airborne iodine concentration in areas requiring personnel access under accident conditions.

By letter dated December 31, 1979, the licensee proposed a program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions.

This program includes (1) Training of personnel, (2) Procedures for monitoring, and (3) Provisions for maintenance of sampling and analysis equipment. We have reviewed this program and conclude that the licensee has satisfied this requirement. The proposed Technical Specifications will ensure compliance.

4.0 Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

5.0 Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: April 13, 1981

7590-01

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-220

NIAGARA MOHAWK POWER CORPORATION

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 42 to Facility Operating License No. DPR-63 issued to Niagara Mohawk Power Corporation (the licensee) which revised the Technical Specifications for operation of the Nine Mile Point Nuclear Station, Unit No. 1 (the facility) located in Oswego County, New York. The amendment is effective as of its date of issuance.

The revisions to the Technical Specifications involve incorporation of certain of the TMI-2 Lessons Learned Category "A" requirements. These requirements are related to (1) Emergency Power Supply/Inadequate Core Cooling, (2) Valve Position Indication, (3) Containment Isolation, (4) Shift Technical Advisors, (5) Integrity of Systems Outside Containment, and (6) Iodine Monitoring.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated September 17, 1980, (2) Amendment No. 42 to License No. DPR-63, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D. C. and at the Penfield Library, State University College at Oswego, Oswego, New York 13126. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland this 13th day of April 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

TA Ippolito

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing