

Docket No. 50-220

May 13, 1981

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Mr. Donald P. Dise
 Vice President - Engineering
 Niagara Mohawk Power Corporation
 300 Erie Boulevard West
 Syracuse, New York 13202

Dear Mr. Dise:

The Commission has issued the enclosed Amendment No. 43 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Power Station Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your applications dated October 15, 1980, and April 1, 1981.

The enclosed Technical Specification changes will further ensure control rod drive system operability during plant operation and reflect the modifications to the scram discharge system presently being undertaken. These Technical Specifications will allow plant operation at the completion of the current refueling outage. Please note that several of the modifications still require a future post-implementation design review by the staff.

Copies of the Safety Evaluation and Notice of Issuance are also enclosed.

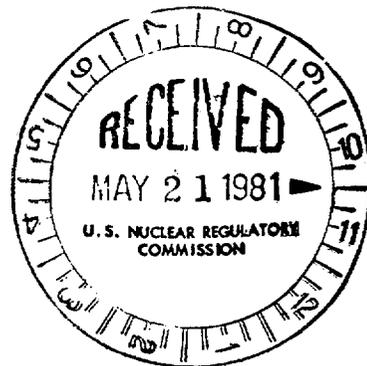
Sincerely,
 Original Signed by
 T. A. Ippolito

Thomas A. Ippolito, Chief
 Operating Reactors Branch #2
 Division of Licensing

Enclosures:

1. Amendment No. 43 to DPR-63
2. Safety Evaluation
3. Notice

cc w/encls:
 See next page



No legal objection

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DL-ORB#2
 TA Ippolito
 5/13/81

OELD
 R. L. Long
 5/13/81

OFFICE	DL:ORB#2	DL:ORB#2	DL:ORB#2	DL:OR	DSI	DSI	DSI
SURNAME	SNorris	PPolk:ms	JHannon	AMNovak	HHolz	BMann	OSarr
DATE	5/13/81	5/13/81	5/13/81	5/15/81	5/ /81	5/ /81	5/13/81

Docket file



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

May 13, 1981

Docket No. 50-220

Mr. Donald P. Dise
Vice President - Engineering
Niagara Mohawk Power Corporation
300 Erie Boulevard West
Syracuse, New York 13202

Dear Mr. Dise:

The Commission has issued the enclosed Amendment No. 43 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Power Station Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your applications dated October 15, 1980, and April 1, 1981.

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Copies of the Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Enclosures:

1. Amendment No. 43 to DPR-63
2. Safety Evaluation
3. Notice

cc w/encls:
See next page

Mr. Donald P. Dise
Niagara Mohawk Power Corporation

cc:

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Oswego, New York 13126

Niagara Mohawk Power Corporation
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Plant Superintendent
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Syracuse, New York 13202

U.S. Environmental Protection Agency
Region II Office
ATTN: EIS COORDINATOR
26 Federal Plaza
New York, New York 10007

State University at Oswego
Penfield Library - Documents
Oswego, New York 13126

Resident Inspector
c/o U.S. NRC
P. O. Box 126
Lycoming, New York 13093



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 43
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Niagara Mohawk Power Corporation (the licensee) dated October 15, 1980 and April 1, 1981, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that ~~the~~ activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

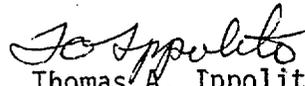
(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 43, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 13, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 43

FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise Appendix A as follows:

Remove

117

119

191

Insert

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LIMITING CONDITION FOR OPERATION

SURVEILLANCE REQUIREMENT

c. If Specifications 3.2.7a and b above are not met, initiate normal orderly shutdown within one hour and have reactor in the cold shutdown condition within ten hours.

c. At least twice per week the feedwater and main-steam-line power-operated isolation valves shall be exercised by partial closure and subsequent re-opening.

d. At least once per quarter the scram discharge system air operated vent and drain valves shall be fully closed and reopened.

LIMITING CONDITION FOR OPERATION

Table 3.2.7 (Continued)

REACTOR COOLANT SYSTEM ISOLATION VALVES

<u>Line or System</u>	<u>No. of Valves (Each Line)</u>	<u>Location Relative to Primary Containment</u>	<u>Normal Position</u>	<u>Motive Power</u>	<u>Maximum Oper. Time (Sec)</u>	<u>Action on Initiating Signal</u>	<u>Initiating Signal (All Valves Have Remote Manual Backup)</u>
<u>Reactor Head Spray (One Line)</u>	1 1	Inside Outside	- Closed	Self Act. Ck. R.H.P.O.	-- 30	- -	- -
<u>Liquid Poison (One Line)</u>	1 1	Inside Outside	- -	Self Act. Ck. Self Act. Ck.	-- --	- -	- -
<u>Control Rod Drive Hydraulic (One Line)</u>	1 1	Inside Outside	- -	Self Act. Ck. Self Act. Ck.	-- --	- -	- -
<u>Scram Discharge System Vent (One Line)</u>	2	Outside	Open	A.I.A.O.	10	Close	High neutron flux, High reactor pressure, High primary contain- ment pressure, Low water level in the re- actor, High level in the scram discharge volume, Low vacuum in condenser, High radiation in main steam line, Closure of main steam isolation valves, Loss of normal and reserve AC power.
<u>Scram Discharge System Drain (One Line)</u>	2	Outside	Open	A.I.A.O.	10	Close	

*A.I.P.O. - Automatically Initiated Power Operated

*R.M.P.O. - Remote Manual Power Operated

A.I.A.O. - Automatically Initiated Air Operated

Table 3.6.2a

INSTRUMENTATION THAT INITIATES SCRAMLimiting Condition for Operating

<u>Parameter</u>	<u>Minimum No. of Tripped or Operable Trip Systems</u>	<u>Minimum No. of Operable Instrument Channels per Operable Trip System</u>	<u>Set Point</u>	<u>Reactor Mode Switch Position in Which Function Must Be Operable</u>			
				<u>Shutdown</u>	<u>Refuel</u>	<u>Startup</u>	<u>Run</u>
(1) Manual Scram	2	1			X	X	X
(2) High Rector Pressure	2	2	1080 psig		X	X	X
(3) High Drywell Pressure	2	2	3.5 psig		X	(a)	(a)
(4) Low Reactor Water Level	2	2	53 inches (Indicator scale)		X	X	X
(5) High Water Level Scram Discharge Volume	2	2	≤ 45 gal.		(b)	X	X



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 43 TO FACILITY OPERATING LICENSE NO. DPR-63

NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-220

1.0 Introduction

By letter dated October 15, 1980 the Niagara Mohawk Power Corporation (licensee) proposed changes to the Technical Specifications (TS) appended to Facility Operating License No. DPR-63. The proposed changes were submitted in response to a July 7, 1980 request by the NRC staff which was intended to strengthen the provisions for assuring continued operability of the control rod drive system during reactor operation. A subsequent letter dated April 1, 1981 proposed additional changes to the TSs in support of modifications being made to the control rod drive scram discharge system at Nine Mile Point 1 during the spring 1981 refueling outage. These changes are necessary before plant operation with the modified system.

2.0 Discussion

2.1 Scram Discharge Volume Limit Switches and Valves

Our letter of July 7, 1980 requested all operating BWR licensees to propose TS changes that would provide surveillance requirements for scram discharge volume (SDV) vent and drain valves and LCO/surveillance requirements for the RPS scram and control rod block scram discharge volume limit switches on instrument volume level. Model Standard Technical Specifications (STS) were included as guidance to the licensees in an enclosure to the July 7 letter. This request was generated as a result of events at operating BWR's involving common cause failures of SDV limit switches and SDV drain valve operability.

2.2 Scram Discharge Volume Modifications

Our letter dated December 9, 1980 forwarded the staff's Generic Safety Evaluation Report (SER) regarding the BWR Scram Discharge System to all BWR licensees. The SER provided design criteria for SDV modifications proposed as the result of the Browns Ferry 3 partial scram event of June 28, 1980. Certain BWRs were determined to have insufficient drainage

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capability between the scram discharge volume and the instrument volume (IV). Subsequently, these licensees received Orders dated January 9, 1981 requiring interim measures to support continued operation until such time as the SDV-IV hydraulic coupling improvement modifications were completed.

The Niagara Mohawk Power Corporation filed a request for modification of the Order for Nine Mile Point Nuclear Station, Unit No. 1, on February 6, 1981. A modified Order was issued on March 31, 1981 which permitted the installation of permanent modifications to improve the hydraulic coupling between the scram discharge volume and the scram level instrumentation in lieu of the temporary measures otherwise required to be in place by April 9, 1981. As a result of the required system modifications, the licensee found it necessary to request TS changes in advance of return to power operation post-refueling. The licensee's letter dated April 1, 1981 proposed the necessary TS changes.

3.0 Evaluation

3.1 SDV Vent & Drain Valves

We required periodic surveillance testing on the scram discharge volume drain and vent valves to verify operability. The licensee's proposal provided the requested surveillance requirements. The licensee's proposal further imposed limiting conditions for operation (LCO's) with bases on the SDV vent/drain valves which we find acceptable. We find that the licensee's submittal of October 15, 1980 exceeds our minimum requirements and is, therefore, acceptable.

3.2 IV Level Limit Switches

We required LCO's and periodic surveillance testing on the Reactor Protection System scram level switches and the control rod block level switches. The licensee's response indicated that, with the exception of the monthly instrument channel test on the scram discharge volume water level scram bypass, the Model STS requirements were covered by the present Technical Specifications.

We have reviewed the current specifications (LCO 3.6.2, Tables 3.6.2a, 4.6.2a, 3.6.2g) and determined that adequate provisions are in place for the installed level instrumentation. We also agree that the current (refueling interval) test frequency for the SDV water level bypass function is adequate and a monthly test is not required. A future revision

to the Standard Technical Specification will reflect this change. Based on this review, we conclude that no changes are necessary to satisfy our request.

3.3 Scram Discharge Volume Modifications

By Order dated March 31, 1981, the licensee was allowed the option of installing permanent modifications to improve the SDV-IV hydraulic coupling, provided that: (1) the modifications are installed prior to power operation after April 9, 1981, (2) the modifications meet Design Criterion 1 in the staff's generic SER, and (3) appropriate Technical Specification changes are approved prior to power operation after April 9, 1981. The licensee's letter of April 1, 1981 proposed to: (1) include LCO/surveillance for the SDV vent/drain valves as reactor coolant system isolation valves, and (2) revise the setpoint for the high SDV water level scram.

The licensee is modifying the scram discharge system vent and drain lines to provide redundant air operated valves to insure isolation capability. This appears to be consistent with the intent of Safety Criterion 2 in the staff's Generic SER and will be subject to a post-implementation design verification by the NRC. The proposed Technical Specifications include LCO/surveillance requirements for the SDV vent/drain valves as a subset of the reactor coolant system isolation valves, and are acceptable.

The licensee is modifying the Instrument Volume by replacing a portion of the 2 inch drain line with 8 inch pipe. This modification will be subject to a post-implementation design review by the NRC to verify conformance with Functional Criterion 1 and Design Criterion 1 in the staff's Generic SER. The proposed Technical Specifications revise the scram level setpoint to conform to the modified instrument volume, and are acceptable.

4.0 Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

5.0 Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: May 13, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-220NIAGARA MOHAWK POWER CORPORATIONNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 43 to Facility Operating License No. DPR-63 issued to Niagara Mohawk Power Corporation (the licensee) which revised the Technical Specifications for operation of the Nine Mile Point Nuclear Station, Unit No. 1 (the facility) located in Oswego County, New York. The amendment is effective as of its date of issuance.

The revisions to the Technical Specifications will further ensure control rod drive system operability during plant operation and reflect the modifications to the scram discharge system presently being undertaken.

The applications for the amendment comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §1.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the applications for amendment dated October 15, 1980 and April 1, 1981, (2) Amendment No. 43 to License No. DPR-63, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D. C. and at the Penfield Library, State University College at Oswego, Oswego, New York 13126. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland this 13th day of May 1981.

FOR THE NUCLEAR REGULATORY COMMISSION


Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing