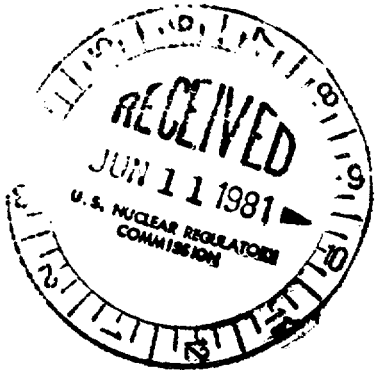
Docket File

Docket No. 50-220

JUN 9 1981



Mr. Donald P. Dise
Vice President - Engineering
c/o Miss Catherine R. Seibert
Niagara Mohawk Power Corporation
300 Erie Boulevard West
Syracuse, New York 13202

Dear Mr. Dise:

By letter dated June 1, 1981, we transmitted Amendment No. 55 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station, Unit 1.

Amendment No. 55 is an incorrect reference. All references to Amendment No. 55 should be changed to Amendment No. 45. We are sorry for any inconvenience this may have caused you.

Sincerely,

ORIGINAL SIGNED BY

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

cc: See next page

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8106170358

OFFICE	ORB#2	ORB#2	ORB#2				
SURNAME	SNorris:pbe	P.Polk	TIppolito				
DATE	6/8 /81	6/9 /81	6/9 /81				

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

June 9, 1981

Docket No. 50-220

Mr. Donald P. Dise
Vice President - Engineering
c/o Miss Catherine R. Seibert
Niagara Mohawk Power Corporation
300 Erie Boulevard West
Syracuse, New York 13202

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Amendment No. 55 is an incorrect reference. All references to Amendment No. 55 should be changed to Amendment No. 45. We are sorry for any inconvenience this may have caused you.

Sincerely,

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

cc: See next page

Mr. Donald P. Dise
Niagara Mohawk Power Corporation

cc:
Eugene B. Thomas, Jr., Esquire
LeBoeuf, Lamb, Leiby & MacRae
1333 New Hampshire Avenue, N. W.
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Washington, D. C. 20036

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Technological Development Programs
State of New York
Energy Office
Swan Street Building
CORE 1 - Second Floor
Empire State Plaza
Albany, New York 12223

Mr. Robert P. Jones, Supervisor
Town of Scriba
R. D. #4
Oswego, New York 13126

Niagara Mohawk Power Corporation
ATTN: Mr. Thomas Perkins
Plant Superintendent
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Syracuse, New York 13202

U.S. Environmental Protection Agency
Region II Office
ATTN: EIS COORDINATOR
26 Federal Plaza
New York, New York 10007

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c/o U.S. NRC
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Lycoming, New York 13093

Docket File

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Docket No. 50-220

JUN 1 1981

Mr. Donald P. Dise
Vice President - Engineering
c/o Miss Catherine R. Seibert
Niagara Mohawk Power Corporation
300 Erie Boulevard West
Syracuse, New York 13202

Dear Mr. Dise:

The Commission has issued the enclosed Amendment No. ~~55~~ 45 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station, Unit 1. The amendment consists of changes to the Technical Specifications in response to your application dated August 5, 1980.

In your application you requested approval of instrumentation scale changes in order to provide a common reactor reference level. Your request was in response to our letter of May 7, 1980. (This item was subsequently designated Item II.K.3.27 in NUREG-0737 dated October 5, 1980.) The enclosed amendment revises the Technical Specifications to approve the instrumentation scale changes necessary to provide a common reference level. In this regard, note that plant procedures shall be modified to reflect this change and operators shall be trained prior to returning to power operation at the conclusion of the current refueling outage.

The change authorized by this amendment is administrative in nature since the actual level for instrument actuation has not been altered. However, for final resolution of Action Plan Item II.K.3.27 for Nine Mile Point you are requested to provide panel layout drawings that show the reactor water level instrumentation detailing location, scales and name plates. This requested information was specified in our evaluation of the BWR Owners Group position on this item (Letter D. G. Eisenhut to D. B. Waters, dated April 6, 1981). Please provide this information within 30 days of your receipt of this letter.

Copies of the Safety Evaluation and the Notice of Issuance are enclosed.

Sincerely,
Original Signed by
T. A. Ippolito

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

8106170360

Enclosures:
1. Amendment No. 55 to DPR-63
2. Safety Evaluation See paragraph concerning letter about
3. Notice

cc w/encls:
See next page

Concurrence only, as requested, to form of Amendment and/or Notice
OELD
B.M.Bordenick
5/27/81

OFFICE	DL:ORB#2	DL:ORB#2	HFEB	HFEB	ORAB
SURNAME	SNorris	PPolk:ms	ARSmith	DTondi	DVerrelli
DATE	5/21/81	5/29/81	5/ /81	5/ /81	5/ /81

DL:ORB#2 TAIppolito 5/21/81
DL:OR TMNovak /81

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Docket No. 50-220

Mr. Donald P. Dise
Vice President - Engineering
c/o Miss Catherine R. Seibert
Niagara Mohawk Power Corporation
300 Erie Boulevard West
Syracuse, New York 13202

Dear Mr. Dise:

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Insert 1

Copies of the Safety Evaluation and the Notice of Issuance are enclosed.

Sincerely,

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Enclosures:
1. Amendment No. to DPR-63
2. Safety Evaluation
3. Notice

OELD

5/ /81

OFFICE	DL:ORB#2	DL:ORB#2	HFEB	HFEB	ORAB	DL:ORB#2	DL:OR
SURNAME	S. Norris	PPolk:pbe	ARSmith	DTondi	DVerrelli	TAIppolito	TNovak
DATE	5/19/81	5/14/81	5/20/81	5/20/81	5/20/81	5/ /81	5/ /81

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

June 1, 1981

Docket No. 50-220

Mr. Donald P. Dise
Vice President - Engineering
c/o Miss Catherine R. Seibert
Niagara Mohawk Power Corporation
300 Erie Boulevard West
Syracuse, New York 13202

Dear Mr. Dise:

The Commission has issued the enclosed Amendment No. 55 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station, Unit 1. The amendment consists of changes to the Technical Specifications in response to your application dated August 5, 1980.

In your application you requested approval of instrumentation scale changes in order to provide a common reactor reference level. Your request was in response to our letter of May 7, 1980. (This item was subsequently designated Item II.K.3.27 in NUREG-0737 dated October 5, 1980). The enclosed amendment revises the Technical Specifications to approve the instrumentation scale changes necessary to provide a common reference level. In this regard, note that plant procedures shall be modified to reflect this change and operators shall be trained prior to returning to power operation at the conclusion of the current refueling outage.

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Copies of the Safety Evaluation and the Notice of Issuance are enclosed.

Sincerely,

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Enclosures:
1. Amendment No. 55 to DPR-63
2. Safety Evaluation
3. Notice

cc w/encls:
See next page

Mr. Donald P. Dise
Niagara Mohawk Power Corporation

cc:
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LeBoeuf, Lamb, Leiby & MacRae
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T. K. BeBoer, Director
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Mr. Robert P. Jones, Supervisor
Town of Scriba
R. D. #4
Oswego, New York 13126

Niagara Mohawk Power Corporation
ATTN: Mr. Thomas Perkins
Plant Superintendent
Nine Mile Point Plant
300 Erie Boulevard West
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Resident Inspector
c/o U.S. NRC
P. O. Box 126
Lycoming, New York 13093

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 55
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated August 5, 1980 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility License No. DPR-63 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 55, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 1, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 55

FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Remove	Insert
6	6
13	13
52	52
53a	53a
59	59
159	159
213	213

SAFETY LIMIT

c. The neutron flux shall not exceed its scram setting for longer than 1.5 seconds as indicated by the process computer. When the process computer is out of service, a safety limit violation shall be assumed if the neutron flux exceeds the scram setting and control rod scram does not occur.

To ensure that the Safety Limit established in Specifications 2.1.1a and 2.1.1b is not exceeded, each required scram shall be initiated by its expected scram signal. The Safety Limit shall be assumed to be exceeded when scram is accomplished by a means other than the expected scram signal.

d. Whenever the reactor is in the shutdown condition with irradiated fuel in the reactor vessel, the water level shall not be more than 7 feet 11 inches (-30 inches indicator scale) below minimum normal water level (Elevation 302'9"), except as specified "e" below.

e. For the purpose of performing major maintenance (not to exceed 12 weeks in duration) on the reactor vessel; the reactor water level may be lowered 9' below the minimum normal water level (Elevation 302'9"). Whenever the reactor water level is to be lowered below the low-low-low level set point redundant instrumentation will be provided to monitor the reactor water level.

Amendment No. 55

LIMITING SAFETY SYSTEM SETTING

d. The reactor water low level scram trip setting shall be no lower than -12 inches (53 inches indicator scale) relative to the minimum normal water level (302'9").

e. The reactor water low-low level setting for core spray initiation shall be no less than -5 feet (5 inches indicator scale) relative to the minimum normal water level (Elevation 302'9").

f. The flow biased APRM rod block trip settings shall be less than or equal to that shown in Figure 2.1.1.

During periods when the reactor is shut down, consideration must also be given to water level requirements, due to the effect of decay heat. If reactor water level should drop below the top of the active fuel during this time, the ability to cool the core is reduced. This reduction in core cooling capability could lead to elevated cladding temperatures and clad perforation. The core will be cooled sufficiently to prevent clad melting should the water level be reduced to two-thirds of the core height.

The lowest point at which the water level can normally be monitored is approximately 4 feet 8 inches above the top of the active fuel. This is the low-low-low water level trip point, which is 7 feet 11 inches (-30 inches indicator scale) below minimum normal water level (Elevation 302'9"). The safety limit has been established here to provide a point which can be monitored and also can provide adequate margin. However, for performing major maintenance as specified in Specification 2.1.1.e, redundant instrumentation will be provided for monitoring reactor water level below the low-low-low water level set point. (For example, by installing temporary instrument lines and reference pots to redundant level transmitters, so that the reactor water level may be monitored over the required range.) In addition written procedures, which identify all the valves which have the potential of lowering the water level inadvertently, are established to prevent their operation during the major maintenance which requires the water level to be below the low-low level set point.

The thermal power transient resulting when a scram is accomplished other than by the expected scram signal (e.g., scram from neutron flux following closure of the main turbine stop valves) does not necessarily cause fuel damage. However, for this specification a safety limit violation will be assumed when a scram is only accomplished by means of a backup feature of the plant design. The concept of not approaching a safety limit provided scram signals are operable is supported by the extensive plant safety analysis.

LIMITING CONDITION FOR OPERATION

c. If a redundant component in each of the core spray systems becomes inoperable, both systems shall be considered operable provided that the component is returned to an operable condition within 7 days and the additional surveillance required is performed.

d. If a copy spray system becomes inoperable and all the components are operable in the other system, the reactor may remain in operation for a period not to exceed 7 days.

e. If Specifications a, b, c and d are not met, a normal orderly shutdown shall be initiated within one hour and the reactor shall be in the cold shutdown condition within ten hours.

If both core spray systems become inoperable the reactor shall be in the cold shutdown condition within ten hours and no work (except as specified in "f" and "h" below) shall be performed on the reactor or its connected systems which could result in lowering the reactor water level to more than seven feet eleven inches below minimum normal water level (-30 inches indicator scale).

Amendment No. 55

SURVEILLANCE REQUIREMENT

d. Core spray header ΔP instrumentation

check	Once/day
calibrate	Once/3 months
test	Once/3 months

e. Surveillance with Inoperable Components

When a component or system becomes inoperable its redundant component or system shall be demonstrated to be operable immediately and daily thereafter.

f. Surveillance during control rod drive maintenance which is simultaneous with the suppression chamber unwatered shall include at least hourly checks that the conditions listed in 3.1.4f are met.

LIMITING CONDITION FOR OPERATION

SURVEILLANCE REQUIREMENTS

h. For the purpose of performing major maintenance (not to exceed 12 weeks in duration) on the reactor vessel, the reactor water level may be lowered to 9' below the minimum normal water level (elevation 302'9"). Whenever the reactor water level is to be lowered below the low-low-low level set point redundant instrumentation will be provided to monitor the reactor water level and written procedures will be developed and followed whenever the reactor water level is lowered below the low-low level set point. The procedures will define the valves that will be used to lower the vessel water level. All other valves that have the potential of lowering the vessel water level will be identified by valve number in the procedures and these valves will be red tagged to preclude their operation during the major maintenance with the water level below the low-low level set point.

During the period of major maintenance requiring lowering the water level to more than 7 feet 11 inches below minimum normal water level (-30 inches indicator scale), either both Core Spray Systems must be operable or, if one Core Spray System is inoperable because of the maintenance, all of the redundant components of the other Core Spray System must be operable.

Pressure Blowdown

In the event of a small line break, substantial coolant loss could occur from the reactor vessel while it was still at relatively high pressures. A pressure blowdown system is provided which in conjunction with the core spray system will prevent significant fuel damage for all sized line breaks (Appendix E-11.2.0*).

Operation of three solenoid-actuated pressure relief valves is sufficient to depressurize the primary system to 110 psig which will permit full flow of the core spray system within required time limits (Appendix E-11.2*). Requiring all six of the relief valves to be operable, therefore, provides twice the minimum number required. Prior to or following refueling at low reactor pressure, each valve will be manually opened to verify valve operability. The malfunction analysis (Section II.XV, "Technical Supplement to Petition to Increase Power Level, "dated April 1970) demonstrates that no serious consequences result if one valve fails to close since the resulting blowdown is well within design limits.

In the event of a small line break, considerable time is available for the operator to permit core spray operation by manually depressurizing the vessel using the solenoid-actuated valves. However, to ensure that the depressurization will be accomplished, automatic features are provided. The relief valves shall be capable of automatic initiation from simultaneous low-low-low water level (7'-11" below minimum normal water level at Elevation 302'9", -30 inches indicator scale) and high containment pressure (3.5 psig). The system response to small breaks requiring depressurization is discussed in Section VII-A.3.3* and the time available to take operator action is summarized in Table VII-1*. Additional information is included in the answers to Questions III-1 and III-5 of the First Supplement.

Steam from the reactor vessel is discharged to the suppression chamber during valve testing. Conducting the tests with the reactor at low pressure such as just prior to or just after refueling minimizes the stress on the reactor coolant system.

The test interval of once per operating cycle results in a system failure probability of 7.0×10^{-7} (Fifth Supplement, p. 115)* and is consistent with practical consideration.

*FSAR

Amendment No. 55

LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENT
c. If a redundant component in each of the containment spray systems or their associated raw water systems become inoperable, both systems shall be considered operable provided that the component is returned to an operable condition within 7 days and that the additional surveillance required is performed.	c. Raw Water Cooling Pumps At least once per quarter manual startup and operability of the raw water cooling pumps shall be demonstrated.
d. If a containment spray system or its associated raw water system becomes inoperable and all the components are operable in the other systems, the reactor may remain in operation for a period not to exceed 7 days.	d. Surveillance with Inoperable Components When a component or system becomes inoperable its redundant component or system shall be demonstrated to be operable immediately and daily thereafter.
e. If Specifications "a" or "b" are not met, shutdown shall begin within one hour and the reactor coolant shall be below 215F within ten hours. If both containment spray systems become inoperable the reactor shall be in the cold shutdown condition within ten hours and no work (except as specified in "f" below) shall be performed on the reactor which could result in lowering the reactor water level to more than seven feet eleven inches (-30 inches indicator scale) below minimum normal water level (Elevation 302'9").	e. Surveillance during control rod drive maintenance which is simultaneous with the suppression chamber unwatered shall include at least hourly checks that the conditions listed in 3.3.7.f are met.

Table 3.6.2f

INSTRUMENTATION THAT INITIATES AUTO DEPRESSURIZATION

Limiting Condition for Operation

Parameter	Minimum No. of Tripped or Operable Trip Systems	Minimum No. of Operable Instrument Channels per Operable Trip System	Set-Point	Reactor Mode Switch Position in Which Function Must Be Operable			
				Shutdown	Refuel	Startup	Run
INITIATION							
(1) a. Low-Low-Low Reactor Water Level	2 (a)	2 (a)	$\geq$ -30 inches (Indicator scale)	(b)		(b)	x
and							
b. High Drywell Pressure	2 (a)	2 (a)	$\leq$ 3.5 psig	(b)		(b)	x

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 55 TO FACILITY OPERATING LICENSE NO. DPR-63

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION, UNIT 1

DOCKET NO. 50-220

1.0 Introduction

By letter dated August 5, 1980 (reference 1) the Niagara Mohawk Power Corporation (licensee) forwarded a proposed Technical Specification change that establishes revised reactor vessel water level setpoints that are consistent with a new common instrument zero level. The proposed common reference level is 7' 11" below the minimum normal water level in the reactor vessel. This level corresponds to 12' 7" above the active fuel and will result in an indicator reading of -30". (At Nine Mile Point Unit 1 numbering is referenced to plant elevation. Plant grade is 261'0". On this basis the minimum normal water level in the reactor is 302'9".) Establishment of the common zero level for all reactor vessel water level instrumentation was required by item II.K3.27 of NUREG-0737, Three Mile Island (TMI) Action Plan (reference 2). The evaluation of the licensee's compliance with this requirement is provided below.

2.0 Evaluation

We have reviewed the proposed revised setpoints necessary to establish a common zero level for all reactor level instrumentation. The common reference level is 65 inches below the minimum normal level of 302' 9". The proposed changes to the Technical Specifications will not change previously established safety settings, i.e., the setpoints for instrument safety functions will not change. Since no change in actual level for any function is involved, and since no instrumentation is being modified, we find the proposed Technical Specification revisions acceptable.

3.0 Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR Section 51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

4.0 Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: June 1, 1981

References

1. Letter dated August 5, 1980 from E. B. Thomas to H. R. Denton.

2. NUREG-0737, "Clarification of TMI Action Plan Requirements," forwarded by letter dated October 31, 1980 from D. G. Eisenhut (NRC) to All Licensees.

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-220

NIAGARA MOHAWK POWER CORPORATION

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 55 to Facility Operating License No. DPR-63 to Niagara Mohawk Power Corporation (the licensee) which revised the Technical Specifications for operation of the Nine Mile Point Nuclear Station, Unit No. 1 (the facility) located in Oswego County, New York. The amendment is effective as of its date of issuance.

The amendment revises the Technical Specifications to approve the instrumentation scale changes necessary to provide a common reactor reference level.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated August 5, 1980, (2) Amendment No. 55 to License No. DPR-63, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Penfield Library, State University College at Oswego, Oswego, New York 13126. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland this 1st day of June 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing