

June 28, 1996

Mr. William J. Cahill, Jr.  
Chief Nuclear Officer  
Power Authority of the State of  
New York  
123 Main Street  
White Plains, NY 10601

SUBJECT: ISSUANCE OF AMENDMENT FOR JAMES A. FITZPATRICK NUCLEAR POWER PLANT  
(TAC NO. M95288)

Dear Mr. Cahill:

The Commission has issued the enclosed Amendment No. 231 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated April 24, 1996.

The amendment proposes to relocate Specification 3.11.B/4.11.B "Crescent Area Ventilation" and associated Bases from the TS to an Authority controlled procedure.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Jocelyn Mitchell for  
Karen R. Cotton, Acting Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-333

Enclosures: 1. Amendment No. 231 to DPR-59  
2. Safety Evaluation

cc w/encls: See next page

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DATED: June 28, 1996

AMENDMENT NO.        TO FACILITY OPERATING LICENSE NO. DPR-59-FITZPATRICK

Docket File

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Sincerely,

A handwritten signature in cursive script that reads "Karen R. Cotton for".

Karen R. Cotton, Acting Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-333

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cc w/encls: See next page

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 231  
License No. DPR-59

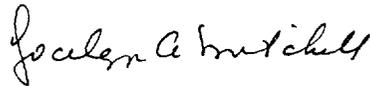
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Power Authority of the State of New York (the licensee) dated April 24, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 231, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Jocelyn A. Mitchell, Acting Director  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: June 28, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 231

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
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**JAFNPP**

**3.11 LIMITING CONDITIONS FOR OPERATION**

**3.11 ADDITIONAL SAFETY RELATED PLANT CAPABILITIES**

**Applicability:**

Applies to the operating status of the main control and relay rooms, and battery room ventilation and cooling. Applies to emergency service water system and intake deicing heaters.

**Objective:**

To assure the availability of the main control and relay room, and battery room ventilation systems, to assure the availability of the emergency service water system and intake deicing heaters, under the conditions for which the capability is an essential response to plant abnormalities.

**A. Main Control Room Ventilation**

1. The reactor shall not have a coolant temperature greater than 212 °F and fuel may not be handled unless both of the control room emergency ventilation air supply fans and fresh air filter trains are available for normal operation except that one emergency

**4.11 SURVEILLANCE REQUIREMENTS**

**4.11 ADDITIONAL SAFETY RELATED PLANT CAPABILITIES**

**Applicability:**

Applies to the surveillance requirements for the main control and relay room, battery room ventilation systems, emergency service water and intake deicing heaters.

**Objective:**

To verify the operability or availability under conditions for which these capabilities are an essential response to plant abnormalities.

**A. Main Control Room Ventilation**

1. Each of the control room emergency ventilation air supply fans and dampers shall be tested for operability every 3 months.

The fresh air filter trains shall be tested once every 6 months as follows:

- a. Pressure drop test across each filter and the filter system.

**JAFNPP**

3.11 (cont'd)

I B. DELETED

C. Battery Room Ventilation

Battery room ventilation shall be operable on a continuous basis whenever specification 3.9.E is required to be satisfied.

1. From and after the date that one of the battery room ventilation systems is made or found to be inoperable, its associated battery shall be considered to be inoperable for purposes of specification 3.9.E.

4.11 (cont'd)

B. DELETED

C. Battery Room Ventilation

Battery room ventilation equipment shall be demonstrated operable once/week.

1. When it is determined that one battery room ventilation system is inoperable, the remaining ventilation system shall be verified operable and daily thereafter.
2. Temperature transmitters and differential pressure switches shall be calibrated once/operating cycle.

3.11 & 4.11 BASES

A. Main Control Room Ventilation System

One main control room emergency ventilation air supply fan provides adequate ventilation flow under accident conditions. Should one emergency ventilation air supply fan and/or fresh air filter train be out of service during reactor operation, a repair time of 14 days is allowed because during that time, a redundant 100% capacity train is required to be operable.

The 3 month test interval for the main control room emergency ventilation air supply fan and dampers is sufficient since two redundant trains are provided and neither is normally in operation.

A pressure drop test across each filter and across the filter system is a measure of filter system condition. DOP injection measures particulate removal efficiency of the high efficiency particulate filters. A Freon-112 test of charcoal filters is essentially a leakage test. Since the filters have charcoal of known efficiency and holding capacity for elemental iodine and/or methyl iodine, the test also gives an indication of the relative efficiency of the installed system. Laboratory analysis of a sample of the charcoal filters positively demonstrates halogen removal efficiency. These tests are conducted in accordance with manufacturers' recommendations.

The purpose of the emergency ventilation air supply system capacity test is to assure that sufficient air is supplied to the main control room so that a slight positive pressure can be maintained, thereby minimizing in-leakage.

B. DELETED

C. Battery Room Ventilation

Engineering analyses indicate that the temperature rise and hydrogen buildup in the battery, and battery charger compartments without adequate ventilation is such that continuous operation of equipment in these compartments cannot be assured.

D. Emergency Service Water System

The ESWS has two 100 percent cooling capacity pumps, each powered from a separate standby power supply. The ESW system supplies lake water to cool equipment required to function following an accident. This equipment consists of: emergency diesel generators, electric bay unit coolers, cable tunnel/emergency switchgear room coolers, crescent area coolers, control room air handling units and relay room air handling units. Emergency service water is initially supplied to the control room chillers and chiller room air handling units unless ESW is manually realigned to supply the control room and relay room air handling units. ESW will also supply water to the control rod drive pump coolers which are not automatically isolated following an accident. The surveillance requirement compares pump performance with the pump curve to determine pump operability. It also specifies testing at a



Page 237, 3.11 Objective

Replace the following:

"To assure the availability of the main control and relay room, battery room, and crescent area ventilation systems, to assure the availability of the emergency service water system and intake deicing heaters, under the conditions for which the capability is an essential response to plant abnormalities."

with

"To assure the availability of the main control and relay room, and battery room ventilation systems, to assure the availability of the emergency service water system and intake deicing heaters, under the conditions for which the capability is an essential response to plant abnormalities."

Page 237, 4.11 Applicability

Replace the following:

"Applies to the surveillance requirements for the main control and relay room, battery room, crescent area ventilation systems, emergency service water and intake deicing heaters."

with

"Applies to the surveillance requirements for the main control and relay room, battery room ventilation systems, emergency service water and intake deicing heaters."

Page 239, 3.11.B.

Replace the following:

"B. Crescent Area Ventilation

Crescent area ventilation and cooling equipment shall be operable on a continuous basis whenever specification 3.5.A, 3.5.B, and 3.5.C are required to be satisfied.

1. From and after the date that more than one unit cooler serving ECCS compartments in the same half of the crescent area are made or found to be inoperable, all ECCS components in that half of the crescent area shall be considered to be inoperable for purposes of specification 3.5.A, 3.5.B, and 3.5.C.
2. If 3.11.B.1 cannot be met, the reactor shall be placed in a cold condition within 24 hours."

with

"B. DELETED"

Page 239, 4.11.B.

Replace the following:

"B. Crescent Area Ventilation

1. Unit coolers serving ECCS components shall be demonstrated operable once/3 months.
2. Each unit cooler's temperature control instrument shall be calibrated once/operating cycle."

with

"B. DELETED"

Page 243, 3.11.B. and 4.11.B. Bases

Replace the following:

"B. Crescent Area Ventilation

Engineering analyses indicate that the temperature rise in safeguards compartments without adequate ventilation flow or cooling is such that continued operation of the safeguards equipment or associated auxiliary equipment cannot be assured."

with

"B. DELETED"

## 2.0 BACKGROUND AND EVALUATION

The purpose of this proposed change is to relocate Specification 3.11.B/4.11.B "Crescent Area Ventilation" and associated Bases from the TS to the FSAR.

The definition of OPERABLE in FitzPatrick TS Section 1.0.J was revised with Amendment Number 83 to include the following requirements:

*"Operable - A system, subsystem, train, component or device shall be OPERABLE or have OPERABILITY when it is capable of performing its specified function(s). Implicit in this definition shall be the assumption that all necessary attendant instrumentation, controls, normal and emergency electrical power sources, cooling or seal water, lubrication or other auxiliary equipment that are required for the system, subsystem, train, component or device to perform its function(s) are also capable of performing their related support function(s)."*

Section 182a of the Atomic Energy Act (the "Act") requires applicants for nuclear power plant operating licenses to state technical specifications (TS) to be included as part of the license. The Commission's regulatory requirements related to the content of technical specifications are set forth in 10 CFR 50.36. That regulation requires that the TS include items in five specific categories, including (1) safety limits, limiting safety system settings and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. However, the regulation does not specify the particular requirements to be included in a plant's TS.

The rule with respect to limiting conditions for operation identifies four criteria to be used in determining whether a particular matter is required to be included in the TS, as follows: (1) Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary; (2) a process variable, design feature, or operating restriction that is an initial condition of a Design Basis Accident or Transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; (3) a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a Design Basis Accident or Transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; (4) a structure, system, or component which operating experience or probabilistic safety assessment has shown to be significant to public health and safety. As a result, existing limiting conditions for operation requirements which fall within or satisfy any of the criteria in 10 CFR 50.36 must be retained in the TS, while those TS requirements which do not fall within or satisfy these criteria may be relocated to other, licensee-controlled documents.

The requirements of the existing TS for the Crescent Area Ventilation system provide a means to assure that safety-related equipment will operate properly during normal plant operation. This support system function is reflected in operability requirements associated with the TS limiting conditions for operation for the specific low-pressure coolant injection (LPCI), residual heat removal (RHR), high-pressure coolant injection (HPCI), and reactor core isolation cooling (RCIC) equipment that the ventilation system serves. However, the Crescent Area Ventilation system is not associated with a process variable that constitutes an initial condition of a design basis accident or transient to support operation of the LPCI, RHR, HPCI, or RCIC equipment. Therefore, the Crescent Area Ventilation system does not meet criterion 2 of 10 CFR 50.36. The staff has also determined that the remainder of the four criteria do not apply to this system. The operation and surveillance of the Crescent Area Ventilation System will be sufficiently described in the FSAR that 10 CFR 50.59 will apply to future changes in these requirements. Therefore, the requirements specified in these existing TS do not satisfy the criteria for TS, and future changes to the requirements for the Crescent Area Ventilation system can be adequately controlled under 10 CFR 50.59.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (61 FR 25710). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Karen R. Cotton

Date: June 28, 1996