

APR 12 1974

Docket No. 50-220

Niagara Mohawk Power Corporation
ATTN: Mr. Philip D. Raymond
Vice President - Engineering
300 Erie Boulevard West
Syracuse, New York 13202

Gentlemen:

Your letter dated January 31, 1974, submitted a request to change the existing Nine Mile Point Unit 1 reactor building by the addition of an extension to facilitate spent fuel handling operations.

We have reviewed the information which you submitted. We have compared the design of the proposed addition with that of the original structure and concluded that if the addition is designed and constructed in accordance with the criteria described in your FSAR for your secondary containment that the addition will not involve a significant hazards consideration and that there is reasonable assurance that the health and safety of the public will not be endangered. Therefore, pursuant to Section 50.59 of 10 CFR Part 50, you may construct the building addition as described in your letter of January 31, 1974.

Because of the added volume to the secondary containment (i.e. reactor building), the performance characteristics of your Standby Gas Treatment System must be verified when the extension is completed. This test is necessary to assure that the reactor building design internal vacuum of 1/4 inch water can be met. Please notify the USAEC Regulatory Operations Region I when you intend to perform this test. A copy of our related Safety Evaluation is enclosed.

Sincerely,

Donald J. Skovholt
Assistant Director
for Operating Reactors
Directorate of Licensing

JJShea
RMDiggs
NDube
MJinks
BScharf (15)
SKari
SVarga
RBoyd

Enclosure and cc: See next page

OFFICE →	L:ORB #2	L:ORB #2	L:ORB #2	L:ORB #2	L:ORB	
X7391	<i>CD for</i>	<i>RMD</i>	<i>CD</i>	<i>DLZ</i>	<i>DJS</i>	<i>CP 4/8</i>
SURNAME →	JCSnell:rwg	RMDiggs	CJDeBevec	DLZiemann	DJSkovholt	<i>kg</i>
DATE →	4/12/74	4/12/74	4/12/74	4/12/74	4/12/74	

April 12, 1974

Enclosure:
Safety Evaluation

cc w/enclosure:
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Oswego City Library

cc w/enclosure and cy of NMP's
ltr dtd 1/31/74:
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UNITED STATES ATOMIC ENERGY COMMISSION
SAFETY EVALUATION BY THE DIRECTORATE OF LICENSING

NINE MILE POINT UNIT 1

DOCKET NO. 50-220

ANALYSIS OF THE ADDITION TO EXISTING REACTOR BUILDING

INTRODUCTION

In a letter dated January 31, 1974, Niagara Mohawk requested authority to change the existing Nine Mile Point Unit 1 reactor building by the addition of an extension to facilitate spent fuel handling operations.

DESCRIPTION OF CHANGE

The modification involves the addition of a 20-foot by 20-foot by 80-foot long air lock with a swing door and an airtight seal. Sufficient room will be made available by the addition to enclose a rail car with a spent fuel shipping cask and provide adequate maneuvering room for the rail car and cask during cask loading and unloading. The track bay extension will be attached to the existing reactor building by an airtight seal formed between the two buildings. The track bay extension will be equipped with a motor-operated, double swing door 16 feet 0 inches wide by 17 feet 6 inches high. The door can also be operated manually and is designed to resist an internal or external load of 40 psf. A one piece inflatable seal is provided around the perimeter of each half of the door. An airtight access passageway will be constructed from the existing access door to this door in the reactor building extension. The doors will have local alarms such that personnel about to enter or leave are alerted to the condition of the alternate doors.

EVALUATION

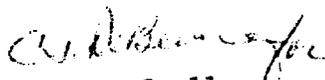
This addition to the reactor building will be designed to meet all the requirements of the original reactor building as described in the NMP-1 FSAR. The planned modifications should reduce the time spent in fuel cask handling operations and, therefore, the probability of occurrence of an accident or malfunction during these operations is reduced. No change to the basic functions of the reactor building and railroad bay have been made. Since the additions to the reactor building will be designed to

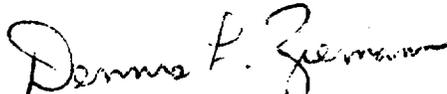
the same leaktight criteria of the original design, no change in the margin of safety defined in the Technical Specifications is necessary.

A retest of secondary containment leak tightness is to be performed after the addition has been completed to assure that leakage characteristics have not changed and that the performance characteristics of the Standby Gas Treatment System have not been degraded.

CONCLUSION

On the basis that the proposed change is designed and constructed in accordance with the criteria described in the FSAR for the secondary containment of the original structure, we have concluded that the addition will not involve a significant hazards consideration and that there is reasonable assurance that the health and safety of the public will not be endangered.


James J. Snell
Operating Reactors Branch #2
Directorate of Licensing


Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Directorate of Licensing

Date: April 12, 1974