



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

July 12, 1993

Docket No. 50-333

Mr. Ralph E. Beedle
Executive Vice President - Nuclear Generation
Power Authority of the State of New York
123 Main Street
White Plains, New York 10601

Dear Mr. Beedle:

SUBJECT: ISSUANCE OF AMENDMENT FOR JAMES A. FITZPATRICK NUCLEAR POWER PLANT
(TAC NO. M86249)

The Commission has issued the enclosed Amendment No. 192 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications (TS) in response to your application transmitted by letter dated February 25, 1993.

The amendment revises certain TS Limiting Conditions for Operation (LCO) action statements to adopt consistent terminology for the action statements and to more clearly distinguish between the different actions associated with each LCO. The amendment also adds required completion times to action statements that previously did not specify completion times and revises the resulting mode to be achieved if the action specified in the LCO action statement cannot be maintained.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

A handwritten signature in black ink, appearing to read "John E. Menning".

John E. Menning, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.192 to DPR-59
2. Safety Evaluation

cc w/enclosures:
See next page

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James A. FitzPatrick Nuclear
Power Plant

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475 Allendale Road
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DATED: July 12, 1993

AMENDMENT NO. 192 TO FACILITY OPERATING LICENSE NO. DPR-59-FITZPATRICK

Docket File

NRC & Local PDRs

PDI-1 Reading

S. Varga, 14/E/4

J. Calvo, 14/A/4

R. Capra

C. Vogan

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OGC-WF

D. Hagan, 3302 MNBB

G. Hill (2), P1-22

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 192
License No. DPR-59

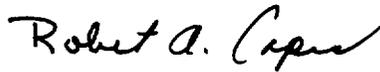
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated February 25, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 192, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 12, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 192

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise Appendix A as follows:

Remove Pages

42
65
77d
107
123
124
124b
142
178
180a
186
217
238
241

Insert Pages

42
65
77d
107
123
124
124b
142
178
180a
186
217
238
241

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TABLE 3.1-1 (cont'd)
REACTOR PROTECTION SYSTEM (SCRAM) INSTRUMENTATION REQUIREMENT

Minimum No. of Operable Instrument Channels per Trip System (1)	Trip Function	Trip Level Setting	Modes in Which Function Must be Operable			Total Number of Instrument Channels Provided by Design for Both Trip Systems	Action (1)
			Refuel (6)	Startup (16)	Run		
4	Turbine Stop Valve Closure	≤ 10% valve closure			X(4)(5)	8 Instrument Channels	A or C

NOTES OF TABLE 3.1-1

1. There shall be two operable or tripped trip systems for each function, except as specified in 4.1.D. From and after the time that the minimum number of operable instrument channel for a trip system cannot be met, that affected trip system shall be placed in the safe (tripped) condition, or the appropriate actions listed below shall be taken.
 - A. Insert all operable control rods within four hours.
 - B. Reduce power level to IRM range and place Mode Switch in the Startup Position within eight hours.
 - C. Reduce power level to less than 30 percent of rated within four hours.
2. Permissible to bypass, if Refuel and Shutdown positions of the Reactor Mode Switch.
3. Deleted.
4. Bypassed when turbine first stage pressure is less than 217 psig or less than 30 percent of rate.
5. The design permits closure of any two lines without a scram being initiated.
6. When the reactor is subcritical and the reactor water temperature is less than 212°F, only the following trip functions need to be operable:
 - A. Mode Switch in Shutdown.
 - B. Manual Scram.

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TABLE 3.2-1 (Cont'd)
INSTRUMENTATION THAT INITIATES PRIMARY CONTAINMENT ISOLATION

NOTES FOR TABLE 3.2-1

1. Whenever Primary Containment integrity is required by Section 3.7, there shall be two operable or tripped trip systems for each function.
2. From and after the time it is found that the first column cannot be met for one of the trip systems, that trip system shall be tripped or the appropriate action listed below shall be taken.
 - A. Place the reactor in the cold condition within 24 hours.
 - B. Isolate the main steam lines within eight hours.
 - C. Isolate Reactor Water Cleanup System within four hours.
 - D. Isolate shutdown cooling within four hours.
3. Deleted
4. Deleted
5. Two required for each steam line.
6. These signals also start SBGTS and initiate secondary containment isolation.
7. Only required in run mode (interlocked with Mode Switch).
8. Bypassed when mode switch is not in run mode and turbine stop valves are closed.
9. The trip level setpoint will be maintained at ≤ 3 times normal rated full power background. See note 16 to Table 3.1-1 for re-setting trip level setpoint just prior to and following the Hydrogen Addition Test.

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TABLE 3.2-8 (Cont'd)
ACCIDENT MONITORING INSTRUMENTATION

NOTES FOR TABLE 3.2-8

- A. With the number of operable channels less than the required minimum, either restore the inoperable channels to operable status within 30 days, or be in a cold condition within the next 24 hours.
- B. With the number of OPERABLE channels less than required by the minimum channels OPERABLE requirements, initiate an alternate method of monitoring the appropriate parameter(s) within 72 hours and: (1) either restore the inoperable channel(s) to OPERABLE status within 7 days of the event, or (2) prepare and submit a Special Report to the Commission within 14 days following the event outlining the cause of the inoperability, the action taken, and the plans and schedule for restoring the system to OPERABLE status.
- C. Each Safety/Relief Valve is equipped with two acoustical detectors, one of which is in service. Each SRV also has a backup thermocouple detector. In the event that a thermocouple is inoperable, SRV performance shall be monitored daily with the associated in service acoustical detector.
- D. From and after the date that both of the acoustical detectors are inoperable, continued operation is permissible until the next outage in which a primary containment entry is made provided that the thermocouple is operable. Both acoustical detectors shall be made operable prior to restart.
- E. In the event that both primary (acoustical detectors) and secondary (thermocouple) indications of this parameter for any one valve are disabled and neither indication can be restored in forty-eight (48) hours, the reactor shall be in a Hot Shutdown condition within twelve (12) hours and in a Cold Shutdown within the next twenty-four (24) hours.
- F. Refer to Specification 3.7.A.9.
- G. This parameter and associated instrumentation are not part of post-accident monitoring.
- H. This instrument shall be operable in the Run, Startup/Hot Standby, and Hot Shutdown modes.
- J. This instrument shall be operable in the Run and Startup/Hot Standby modes.

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3.4 (Cont'd)

C. Sodium Pentaborate Solution

The standby liquid control solution tank shall contain a boron bearing solution with a minimum enrichment of 34.7 atom percent of B-10 that satisfies the volume-concentration requirements of Fig. 3.4-1 at all times when the Standby Liquid Control System is required to be operable and the solution temperature including that in the pump suction piping shall not be less than the temperature presented in Fig. 3.4-2. Tank heater and the heat tracing system shall be operable whenever the SLCS is required in order to maintain solution temperature in accordance with Fig. 3.4-2. If these requirements are not met, restore the system to the above limits within eight hours or take action in accordance with Specification 3.4.D.

D. If specifications 3.4.A through C are not met, the reactor shall be in at least hot shutdown within the following 12 hours.

4.4 (Cont'd)

C. Sodium Pentaborate Solution

The availability of the proper boron bearing solution shall be verified by performance of the following tests:

1. At least once per month -

Boron concentration shall be determined. In addition, the boron concentration shall be determined any time water or enriched sodium pentaborate is added or if the solution temperature drops below the limits specified by Fig. 3.4-2.

2. At least once per day -

Solution volume and the solution temperature shall be checked.

3. At least once per operating cycle -

- a. The temperature and level elements shall be calibrated.
- b. Enrichment of B-10 (in atom percent) shall be checked.

D. Not Used

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3.5 (cont'd)

condition, that pump shall be considered inoperable for purposes of satisfying Specifications 3.5.A, 3.5.C, and 3.5.E.

H. Average Planar Linear Heat Generation Rate (APLHGR)

During power operation, the APLHGR for each type of fuel as a function of axial location and average planar exposure shall be within limits based on applicable APLHGR limit values which have been approved for the respective fuel and lattice types. These values are specified in the Core Operating Limits Report. If at anytime during reactor power operation greater than 25% of rated power it is determined that the limiting value for APLHGR is being exceeded, action shall then be initiated within 15 minutes to restore operation to within the prescribed limits. If the APLHGR is not returned to within the prescribed limits within two (2) hours, the reactor power shall be reduced to less than 25% of rated power within the next four hours, or until the APLHGR is returned to within the prescribed limits.

4.5 (cont'd)

2. Following any period where the LPCI subsystems or core spray subsystems have not been maintained in a filled condition; the discharge piping of the affected subsystem shall be vented from the high point of the system and water flow observed.
3. Whenever the HPCI or RCIC System is lined up to take suction from the condensate storage tank, the discharge piping of the HPCI or RCIC shall be vented from the high point of the system, and water flow observed on a monthly basis.
4. The level switches located on the Core Spray and RHR System discharge piping high points which monitor these lines to insure they are full shall be functionally tested each month.

H. Average Planar Linear Heat Generation Rate (APLHGR)

The APLHGR for each type of fuel as a function of average planar exposure shall be determined daily during reactor operation at $\geq 25\%$ rated thermal power.

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3.5 (cont'd)

I. Linear Heat Generation Rate (LHGR)

The linear heat generation rate (LHGR) or any rod in any fuel assembly at any axial location shall not exceed the maximum allowable LHGR specified in the Core Operating Limits Report.

If anytime during reactor power operation greater than 25% of rated power it is determined that the limiting value for LHGR is being exceeded, action shall then be initiated within 15 minutes to restore operation to within the prescribed limits. If the LHGR is not returned to within the prescribed limits within two (2) hours, the reactor power shall be reduced to less than 25% of rated power within the next four hours, or until the LHGR is returned to within the prescribed limits.

4.5 (cont'd)

I. Linear Heat Generation Rate (LHGR)

The LHGR shall be determined daily during reactor operation at $\geq 25\%$ rated thermal power.

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3.5 (cont'd)

- 2. Within 2 hours after completing an increase in thermal power of 5 percent or more of rated thermal power.**

 - b. If the APRM and LPRM neutron flux noise levels are greater than 5 percent and greater than three times their established baseline noise levels, initiate corrective action within 15 minutes to restore the noise levels to within the required limits within 2 hours, by increasing core flow and/or reducing thermal power.**
- 3. If during single-loop operation, core thermal power is greater than the limit defined by line A of Figure 3.5-1, and core flow is less than 39 percent, immediately initiate corrective action to restore core thermal power and/or core flow to within the limits, specified in Figure 3.5-1, by increasing core flow and/or initiating an orderly reduction of core thermal power by inserting control rods.**
- 4. The requirements applicable to single-loop operation in Specifications 1.1.A, 2.1.A, 3.1.A, 3.1.B, 3.2.C and 3.5.H shall be in effect within 8 hours following the removal of one recirculation loop from service, or the reactor shall be placed in at least the hot shutdown condition within 12 hours.**

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3.6 (cont'd)

5. With the Primary Containment Sump Monitoring System (Equipment Drain Sump Monitoring or Floor Drain Sump Monitoring) inoperable, restore the system to operable status within 24 hours or be in at least hot shutdown within the next 12 hours and in the cold condition within the following 24 hours.
6. With the Primary Containment Atmosphere Radioactivity Monitoring System (gaseous) or the Primary Containment Atmosphere Radioactivity Monitoring System (particulate) inoperable, operation may continue for up to 30 days provided grab samples of the containment atmosphere are obtained and analyzed at least once per 24 hours. Otherwise be in at least hot shutdown within the next 12 hours and in cold shutdown within the following 24 hours.

4.6 (cont'd)

3. Drywell Continuous Atmosphere Radioactivity Monitoring System instrumentation shall be functionally tested and calibrated as specified in Table 4.6.2.

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3.7 (cont'd)

breaker is sooner made operable, provided that the repair procedure does not violate primary containment integrity.

5. Pressure Suppression Chamber - Drywell Vacuum Breakers

- a. When primary containment integrity is required, all drywell suppression chamber vacuum breakers shall be operable and positioned in the fully closed position except during testing and as specified in 3.7.A.5.b below.
- b. One drywell suppression chamber vacuum breaker may be non-fully closed so long as it is determined to be not more than 1° open as indicated by the position lights.
- c. One drywell suppression chamber vacuum breaker may be determined to be inoperable for opening.
- d. Deleted

4.7 (cont'd)

5. Pressure Suppression Chamber - Drywell Vacuum Breakers

- a. Each drywell suppression chamber vacuum breaker shall be exercised through an opening - closing cycle monthly.
- b. When it is determined that one vacuum breaker is inoperable for fully closing when operability is required, the operable breakers shall be exercised immediately, and every 15 days thereafter until the inoperable valve has been returned to normal service.
- c. Once each operating cycle, each vacuum breaker valve shall be visually inspected to insure proper maintenance and operation.
- d. A leak test of the drywell to suppression chamber structure shall be conducted once per operating cycle; the acceptable leak rate is ≤ 0.25 in. water/min, over a 10 min period, with the drywell at 1 psid.

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3.7 (Cont'd)

- (1) This differential pressure shall be established within a 24 hour period subsequent to placing the reactor in the run mode. The differential pressure may be reduced to less than 1.7 psid 24 hours prior to a scheduled shutdown.**
 - (2) The differential pressure may be decreased to less than 1.7 psid for a maximum of four (4) hours during required operability testing of the HPCI, RCIC, and Suppression Chamber - Drywell Vacuum Breaker System.**
 - (3) If the specifications of Item a, above, cannot be met, and the differential pressure cannot be restored within the subsequent six (6) hour period the reactor shall be in a Hot Shutdown condition in six (6) hours and a Cold Shutdown condition in the following eighteen (18) hours.**
- 8. If the specifications of 3.7.A.1 through 3.7.A.6 cannot be met the reactor shall be in the cold condition within 24 hours.**

4.7 (Cont'd)

- 8. Not applicable.**

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3.7 (cont'd)

2. With one or more of the containment isolation valves inoperable, maintain at least one isolation valve operable in each affected penetration that is open and:
 - a. Restore the inoperable valve(s) to operable status within 4 hours; or
 - b. Isolate each affected penetration within 4 hours by use of at least one deactivated automatic valve secured in the closed position. Isolation valves closed to satisfy these requirements may be reopened on an intermittent basis under administrative control; or
 - c. Isolate each affected penetration within 4 hours by use of at least one closed manual valve or a blind flange.
3. If Specifications 3.7.D.1 or 3.7.D.2 cannot be met the reactor shall be in the cold condition within 24 hrs.

4.7 (cont'd)

- (2.) With the reactor at reduced power level, trip main steam isolation valves and verify closure time.
 - d. At least twice per week, the main steam line power-operated isolation valves shall be exercised by partial closure and subsequent reopening.
 - e. The RBCLCWS isolation valves shall be fully closed and reopened any time the reactor is in the cold condition exceeding 48 hours, if the valves have not been fully closed and reopened during the preceding 92 days.
2. Whenever a containment isolation valve is inoperable, the position of at least one other valve in each line having an inoperable valve shall be recorded daily.
3. Not Used

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3.9 (cont'd)

3. From and after the time that one of the Emergency Diesel Generator Systems is made or found to be inoperable, continued reactor operation is permissible for a period not to exceed 7 days provided that the two incoming power sources are available and that the remaining Diesel Generator System is operable. At the end of the 7 day period, the reactor shall be placed in a cold condition within 24 hours, unless the affected diesel generator system is made operable sooner.
4. When both Emergency Diesel Generator Systems are made or found to be inoperable restore at least one system to operable status within two hours or place the reactor in the cold condition within the following 24 hours.
5. Deleted

4.9 (cont'd)

3. The emergency diesel generator system instrumentation shall be checked during the monthly generator test.
4. Once each operating cycle, the conditions under which the Emergency Diesel Generator System is required will be simulated to demonstrate that the pair of diesel generators will start, accelerate, force parallel, and accept the emergency loads in the prescribed sequence.
5. Once within one hour and at least once per twenty-four hours thereafter while the reactor is being operated in accordance with Specifications 3.9.B.1, 3.9.B.2, or 3.9.B.3 the availability of the operable Emergency Diesel Generators shall be demonstrated by manual starting and force paralleling where applicable.

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3.11 (cont'd)

ventilation air supply fan and/or filter may be out of service for 14 days.

2. The main control room air radiation monitor shall be operable whenever the control room emergency ventilation air supply fans and filter trains are required to be operable by 3.11.A.1 or filtration of the control room ventilation intake air must be initiated.
3. The control room emergency ventilation system shall not be out of service for a period exceeding 3 days during normal reactor operation or refueling operations. In the event that the system is not returned to service within 3 days, the reactor shall be in cold shutdown within 24 hours and any handling of irradiated fuel, core alterations, and operations with a potential for draining the reactor vessel shall be suspended as soon as practicable
4. Not Used

4.11 (cont'd)

- b. Di-octylphtalate (DOP) test for particulate filter efficiency greater than 99% for particulate greater than 0.3 micron size.
 - c. Freon-112 test for charcoal filter bypass as a measure of filter efficiency of at least 99.5% for halogen removal.
 - d. A sample of charcoal filter shall be analyzed once a year to assure halogen removal efficiency of at least 99.5%.
2. Operability of the main control room air intake radiation monitor shall be tested once/3 months.
 3. Temperature transmitters and differential pressure switches shall be calibrated once/operating cycle.
 4. Main control room emergency ventilation air supply system capacity shall be tested once every 18 months to assure that it is $\pm 10\%$ of the design value of 1000 cfm.

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3.11 (cont'd)

2. From and after the time that one Emergency Service Water System is made or found to be inoperable for any reason continued reactor operation is permissible for a period not to exceed 7 days, provided that:
 - the operable Emergency Diesel Generator System is demonstrated to be operable immediately and daily thereafter; and,
 - all Emergency Diesel Generator System emergency loads are verified operable immediately and daily thereafter.
3. If specification 3.11.D.2 cannot be met the reactor shall be placed in the cold condition within 24 hours.

4.11 (cont'd)

- | | | |
|----|---|--|
| e. | ESW instrumentation-check
calibrate test | Once/day
Once/3 months

Once/3 months |
| f. | Logic System
Functional Test | Once/each
operating cycle |

2. ESW will not be supplied to RBCLC system during testing.
3. Not Used



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 192 TO FACILITY OPERATING LICENSE NO. DPR-59

POWER AUTHORITY OF THE STATE OF NEW YORK

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

1.0 INTRODUCTION

By letter dated February 25, 1993, the Power Authority of the State of New York (the licensee) submitted a request for changes to the James A. FitzPatrick Nuclear Power Plant, Technical Specifications (TS): The requested changes would revise certain TS Limiting Conditions for Operation (LCO) action statements to adopt consistent terminology for the action statements and to more clearly distinguish between the different actions associated with each LCO. The requested changes would also add required completion times to action statements that previously did not specify completion times and would revise the resulting mode to be achieved if the action specified in the LCO action statement cannot be achieved.

2.0 EVALUATION

The proposed changes to the TS are categorized into three categories: (1) clarification of LCO action statements, (2) addition of completion times to LCO action statement requirements, or (3) revision of LCO action statement requirements. The first category of proposed changes would include the following:

Category A Changes - Clarification of LCOs

1. TS Page 42, Table 3.1-1, Note 1.A Replace the phrase "Initiate insertion of operable rods and complete insertion of all operable rods within four hours" with "Insert all operable control rods within four hours."
2. TS Page 65, Table 3.2-1, Note 2.B Replace the phrase "Initiate an orderly load reduction and have main steam lines isolated within eight hours" with "Isolate the main steam lines within eight hours."
3. TS Page 77d, Table 3.2-8, Note E Replace the phrase "an orderly shutdown shall be initiated and the reactor shall be in a Hot Shutdown condition in" with "the reactor shall be in a Hot Shutdown condition within."

4. TS Page 107, Specification 4.4.D Insert with the words, "Not Used."
5. TS Page 123, Specification 3.5.H Join the last two sentences by replacing the phrase "an orderly reactor power reduction shall be commenced immediately. The" with "the."
6. TS Page 124, Specification 3.5.I Join the last two sentences by replacing the phrase "an orderly reactor power reduction shall be commenced immediately. The" with "the."
7. TS Page 178, Specification 3.7.A.5.d Delete this specification and replace with the word "Deleted."
8. TS Page 180a, Specification 3.7.A.7.a(3) Delete the phrase ", an orderly shutdown shall be initiated and."
9. TS Page 180a, Specification 3.7.A.8.a Renumber the specification as "3.7.A.8" and replace the phrase ", an orderly shutdown shall be initiated, and the reactor shall be in a" with "the reactor shall be in the."
10. TS Page 186, Specification 3.7.D.3 Replace the phrase ", an orderly shutdown shall be initiated and the reactor shall be in" with "the reactor shall be in the."
11. TS Page 186, Specification 4.7.D.3 Insert with the words "Not Used."
12. TS Page 217, Specification 3.9.B.4 Replace the phrase ", a reactor shutdown shall be initiated within two hours and the reactor placed in a cold condition within 24 hours after initiation of shutdown" with "restore at least one system to operable status within two hours or place the reactor in the cold condition within the following 24 hours."
13. TS Page 217, Specification 3.9.B.5 Insert with the word "Deleted."
14. TS Page 238, Specification 3.11.A.3 Replace the phrase:

"within 3 days, the reactor will be shutdown in an orderly manner and in the Cold Shutdown Condition within 24 hours or if refueling operations are in progress, such operations will be terminated in an orderly manner"

with

"within 3 days, the reactor shall be in cold shutdown within 24 hours and any handling of irradiated fuel, core alterations, and operations with a potential for draining the reactor vessel shall be suspended as soon as practicable."

15. TS Page 238, Specification 3.11.A.4 Insert with the words, "Not Used."
16. TS Page 241, Specification 3.11.D.2 Delete the words, "total for any calendar month."
17. TS Page 241, Specification 3.11.D.3 Replace the phrase "an orderly shutdown shall be initiated and the reactor shall be placed in a" with "the reactor shall be placed in the."
18. TS Page 241, Specification 4.11.D.3 Insert with the words "Not Used."

The NRC staff has reviewed the proposed changes included in the first category and has determined that phrases such as "initiate an orderly load reduction" and "an orderly shutdown shall be initiated" are not defined in the facility TS or in the NRC's "Standard Technical Specifications for General Electric Boiling Water Reactors (GE-STs)." The proposed changes would delete these phrases. The revised TS would then contain statements such as "place the reactor in the cold shutdown condition" or "isolate the main steam lines." The proposed deletions would not change any TS requirements but would provide clearer and more definitive instructions. Therefore, these proposed changes are acceptable. Elimination of these phrases is applicable to items 1, 2, 3, 5, 6, 8, 9, 10, 12, 14, and 17 of the Category A proposed changes.

The proposed deletion of the phrase "total for any calendar month" from TS 3.11.D.2 (item 16 of the Category A proposed changes) was evaluated by the NRC staff. This phrase was originally intended to limit the allowable out-of-service time of one Emergency Service Water System during a given month to a maximum of 7 days. However, as presently written, this phrase could be misinterpreted to allow an out-of-service time began at the end of one calendar month to continue into the beginning of another calendar month and for the total out-of-service time to then exceed the intended allowable out-of-service time. The removal of this phrase precludes such a possibility and makes TS 3.11.D.2 consistent with other TS in applying out-of-service times and is, therefore, acceptable.

The licensee also proposed to add the words "Not Used" or "Deleted" to several TS sections for which there currently are no associated LCO or surveillance requirements for an existing TS (e.g., Surveillance Requirement 4.11.A.4 was added by License Amendment No. 114 but no corresponding LCO was added at that time, therefore, inserting "Not Used" as LCO 3.11.A.4 is now being proposed; likewise, inserting the word "Deleted" as LCO 3.9.B.5 is now being proposed to signify the deletion of LCO 3.9.B.5 by License Amendment No. 95.) These changes are purely administrative, do not change any TS requirements, and are, therefore, acceptable. Addition of the words "Not Used" or "Deleted" is applicable to items 4, 11, 13, 15, and 18 of the Category A proposed changes.

TS 3.7.A.5.d requires the reactor to be placed in a cold condition if the requirements of TS 3.7.A.5.a, b, and c cannot be met. TS 3.7.A.8.a also requires the reactor to be placed in a cold condition if the requirements of

TS 3.7.A.1 through 3.7.A.6 cannot be met. Item 7 of the Category A proposed changes would delete the requirements of TS 3.7.A.5.d and replace them with the word "Deleted." The NRC staff has concluded that this proposed change is acceptable since the requirements of TS 3.7.A.8.a are redundant to the requirements of TS 3.7.A.5.d.

The second category of proposed changes would include the following:

Category B Changes - Addition of Completion Times to LCO Action Statements

1. TS Page 42, Table 3.1-1, Note 1.C Replace the sentence:

"Reduce power to less than 30 percent of rated."

with

"Reduce power level to less than 30 percent of rated within four hours."

2. TS Page 65, Table 3.2-1, Note 2.C At the end of note 2.C, insert the words, "within four hours."

3. TS Page 65, Table 3.2-1, Note 2.D At the end of note 2.D, insert the words, "within four hours."

4. TS Page 107, Specification 3.4.C At the end of the existing paragraph, add the sentence:

"If these requirements are not met, restore the system to the above limits within eight hours or take action in accordance with Specification 3.4.D."

5. TS Page 124b, Specification 3.5.J.4 Replace the phrase "the hot shutdown condition" with "at least the hot shutdown condition within 12 hours."

The current TS contain five LCOs (items 1, 2, 3, 4, and 5 of the Category B proposed changes) where the action statements do not include a time limit for completing the specified actions. The proposed changes would add time limits for completing the specified actions. The addition of time limits for completing actions specified in action statements is authorized by 10 CFR 50.36(c)(2) which states that when an LCO of a nuclear reactor is not met, the licensee shall shut down the reactor or follow any remedial action permitted by the TS until the condition can be met. The proposed time limits are consistent with time limits specified elsewhere in the TS for similar actions. Therefore, the NRC staff finds the proposed changes acceptable.

The third category of proposed changes would include the following:

Category C Changes - Revision of LCO Action Statement Requirements

1. IS Page 65, Table 3.2-1, Note 2.A Replace note 2.A "Initiate an orderly shutdown and have the reactor in cold shutdown condition in 24 hours" with "Place the reactor in the cold condition within 24 hours."
2. IS Page 107, Specification 3.4.D Replace the phrase "the cold condition within 24 hours" with "at least hot shutdown within the following 12 hours."
3. IS Page 142, Specification 3.6.D.5 Replace the phrase "immediately initiate an orderly shutdown and be in at least hot standby condition within the next 12 hours and in" with "be in at least hot shutdown within the next 12 hours and in the."

Item 1 of the Category C proposed changes would require the reactor to be placed in a "cold condition" rather than in "cold shutdown." The proposed change from "cold shutdown" (defined in TS Definition 1.0.1.3) to "cold condition" (defined in TS Definition 1.0.C) differs only in that cold shutdown requires the reactor mode switch be in the shutdown position and the reactor vessel to be vented. Placing the reactor mode switch in the shutdown position injects a scram signal into the reactor protection system. Initiation of a scram signal is undesirable and unnecessary in that it causes a control rod drive system pressure transient. It is also unnecessary to vent the reactor vessel in this condition. Therefore, the NRC staff finds this proposed change acceptable.

Item 2 of the Category C proposed changes would revise TS 3.4.D to require the reactor to be in hot shutdown within 12 hours rather than in the cold condition within 24 hours as is required in the current TS if the standby liquid control system is inoperable. This proposed change in time and mode requirement is consistent with current NRC staff positions and with the guidance of the GE-STs. Therefore, the NRC staff finds the proposed change acceptable.

Item 3 of the Category C proposed changes would revise TS 3.6.D.5 to require the reactor to be in hot shutdown within 12 hours rather than in hot standby within 12 hours if the primary containment sump monitoring system is inoperable. This proposed change in mode requirement is consistent with current NRC staff positions and with the guidance of the GE-STs. Therefore, the NRC staff finds the proposed change acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (58 FR 32390). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor:
D. Brinkman

Date: July 12, 1993

Docket No. 50-333

July 12, 1993

Mr. Ralph E. Beedle
Executive Vice President - Nuclear Generation
Power Authority of the State of New York
123 Main Street
White Plains, New York 10601

Dear Mr. Beedle:

SUBJECT: ISSUANCE OF AMENDMENT FOR JAMES A. FITZPATRICK NUCLEAR POWER PLANT
(TAC NO. M86249)

The Commission has issued the enclosed Amendment No. 192 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications (TS) in response to your application transmitted by letter dated February 25, 1993.

The amendment revises certain TS Limiting Conditions for Operation (LCO) action statements to adopt consistent terminology for the action statements and to more clearly distinguish between the different actions associated with each LCO. The amendment also adds required completion times to action statements that previously did not specify completion times and revises the resulting mode to be achieved if the action specified in the LCO action statement cannot be maintained.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by:
John E. Menning, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 192 to DPR-59
- 2. Safety Evaluation

cc w/enclosures:
See next page

Distribution:

See attached sheet

*See previous concurrence

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