

October 22, 1996

Mr. William J. Cahill, Jr.  
Chief Nuclear Officer  
Power Authority of the State of  
New York  
123 Main Street  
White Plains, NY 10601

SUBJECT: ADMINISTRATIVE ERROR IN ISSUANCE OF AMENDMENT NO. 234 - JAMES A.  
FITZPATRICK NUCLEAR POWER PLANT (TAC NO. M95099)

Dear Mr. Cahill:

On October 4, 1996, the Commission issued Amendment No. 234 to Facility  
Operating License No. DPR-59 for James A. FitzPatrick Nuclear Power Plant.  
The amendment consists of changes to the Technical Specifications (TSs) in  
response to your application transmitted by letter dated March 27, 1996, as  
supplemented April 24, 1996, August 15, 1996 and June 20, 1996.

The amendment would permit implementation of 10 CFR Part 50, Appendix J, Option  
B, with an exception to the guidelines of Regulatory Guide 1.163 for Type C  
testing of primary containment isolation valves in the reverse (non-accident)  
direction.

Due to an administrative error, we issued the incorrect page 285e of the TSs.  
Enclosed is the correct page 285e. Please update the FitzPatrick TSs  
accordingly. If you have any questions, please contact me at (301) 415-1438.

Sincerely,  
*/s/*

Karen R. Cotton, Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-333

Enclosure: As stated

cc w/encl: See next page

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Due to an administrative error, we issued the incorrect page 285e of the TSs. Enclosed is the correct page 285e. Please update the FitzPatrick TSs accordingly. If you have any questions, please contact me at (301) 415-1438.

Sincerely,

*Karen R. Cotton*

Karen R. Cotton, Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

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Enclosure: As stated

cc w/encl: See next page

William J. Cahill, Jr.  
Power Authority of the State  
of New York

James A. FitzPatrick Nuclear  
Power Plant

cc:

Mr. Gerald C. Goldstein  
Assistant General Counsel  
Power Authority of the State  
of New York  
1633 Broadway  
New York, NY 10019

Resident Inspector's Office  
U. S. Nuclear Regulatory Commission  
P.O. Box 136  
Lycoming, NY 13093

Mr. Harry P. Salmon, Jr.  
Resident Manager  
James A. FitzPatrick Nuclear  
Power Plant  
P.O. Box 41  
Lycoming, NY 13093

Ms. Charlene D. Faison  
Director Nuclear Licensing  
Power Authority of the State  
of New York  
123 Main Street  
White Plains, NY 10601

Supervisor  
Town of Scriba  
Route 8, Box 382  
Oswego, NY 13126

Mr. Robert G. Schoenberger,  
Vice President  
and Chief Operating Officer  
Power Authority of the State  
of New York  
123 Main Street  
White Plains, NY 10601

Charles Donaldson, Esquire  
Assistant Attorney General  
New York Department of Law  
120 Broadway  
New York, NY 10271

Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, PA 19406

Mr. F. William Valentino, President  
New York State Energy, Research,  
and Development Authority  
2 Rockefeller Plaza  
Albany, NY 12223-1253

Mr. Richard L. Patch, Director  
Quality Assurance  
Power Authority of the State  
of New York  
123 Main Street  
White Plains, NY 10601

Mr. Gerard Goering  
28112 Bayview Drive  
Red Wing, MN 55066

Mr. James Gagliardo  
Safety Review Committee  
708 Castlewood Avenue  
Arlington, TX 76012

Mr. Arthur Zaremba, Licensing  
Manager  
James A. FitzPatrick Nuclear  
Power Plant  
P.O. Box 41  
Lycoming, NY 13093

6.19 POSTACCIDENT SAMPLING PROGRAM

A program shall be established, implemented, and maintained which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the following:

- A) Training of personnel,
- B) Procedures for sampling and analysis,
- C) Provisions for maintenance of sampling and analysis

6.20 PRIMARY CONTAINMENT LEAKAGE RATE TESTING PROGRAM

A program shall be established to implement the leakage rate testing of the Primary Containment as required by 10 CFR 50.54 (o) and 10 CFR 50, Appendix J, Option B, as modified by approved exemptions. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program", dated September 1995, as modified by the exception that Type C testing of valves not isolable from the containment free air space may be accomplished by pressurization in the reverse direction provided that testing in this manner provides equivalent or more conservative results than testing in the accident direction. If potential atmospheric leakage paths (e.g., valve stem packing) are not subjected to test pressure, the portions of the valve not exposed to test pressure shall be subjected to leakage rate measurement during regularly scheduled Type A testing. A list of these valves, the leakage rate measurement method, and the acceptance criteria, shall be contained in the Program.

- A. The peak Primary Containment internal pressure for the design basis loss of coolant accident ( $P_a$ ), is 45 psig.
- B. The maximum allowable Primary Containment leakage rate ( $L_a$ ), at  $P_a$ , shall be 0.5% of primary containment air weight per day.
- C. The leakage rate acceptance criteria are:
  - 1. Primary containment leakage rate acceptance criteria is  $\leq 1.0 L_a$ . During unit startup following testing in accordance with this program, the leakage rate acceptance criteria are  $\leq 0.60 L_a$  for the Type B and Type C tests and  $\leq 0.75 L_a$  for the Type A tests;
  - 2. Airlock testing acceptance criteria are:
    - a. Overall airlock leakage rate is  $\leq 0.05 L_a$  when tested at  $\geq P_a$ ,
    - b. For each door seal, leakage rate is  $\leq 120$  scfd when tested at  $\geq P_a$ .
  - 3. MSIV leakage rate acceptance criteria is  $\leq 11.5$  scfh for each MSIV when tested at  $\geq 25$  psig.
- D. The provisions of Specification 4.0.B do not apply to the test frequencies specified in the Primary Containment Leakage Rate Testing Program.
- E. The provisions of Specification 4.0.C are applicable to the Primary Containment Leakage Rate Testing Program.