



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

July 29, 1993

Docket No. 50-333

Mr. Ralph E. Beedle
Executive Vice President - Nuclear Generation
Power Authority of the State of New York
123 Main Street
White Plains, New York 10601

Dear Mr. Beedle:

SUBJECT: ISSUANCE OF AMENDMENT FOR JAMES A. FITZPATRICK NUCLEAR POWER PLANT
(TAC NO. M86325)

The Commission has issued the enclosed Amendment No. 194 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications (TS) in response to your application transmitted by letter dated April 15, 1993.

The amendment makes several changes to Table 4.7-2, "Exception to Type C Tests." Specifically, the changes (1) added system numbers to the valve identification numbers for seven control rod drive containment isolation valves and clarified the penetration arrangement, (2) removed valves 10MOV-57 and 10MOV-67 from the table, (3) added valves 10RHR-729A and 10RHR-729B to the table, and (4) corrected errors introduced in Amendment No. 143.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

for Maudette Griggs
John E. Menning, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 194 to DPR-59
2. Safety Evaluation

cc w/enclosures:
See next page

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Mr. Ralph E. Beedle
Power Authority of the State of New York

James A. FitzPatrick Nuclear
Power Plant

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Regional Administrator, Region I
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DATED: July 29, 1993

AMENDMENT NO. 194 TO FACILITY OPERATING LICENSE NO. DPR-59-FITZPATRICK

Docket File

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cc: Plant Service list

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 194
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated April 15, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 194, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 29, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 194

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Revise Appendix A as follows:

Remove Pages

212
213
213a

Insert Pages

212
213
213a

TABLE 4.7-2

EXCEPTION TO TYPE C TESTS

CONTAINMENT PENETRATION	PENETRATION FUNCTION	VALVE NUMBER	LOCAL LEAK RATE TEST PERFORMED
35C	Traversing In-Core Probe "C"	07EV-104C	This valve is an explosive shear valve which cannot be Type C tested.
35D	Traversing In-Core Probe "B"	07EV-104B	This valve is an explosive shear valve which cannot be Type C test
37A 37B 37C 37D	Control Rod Drive (below piston)	03SOV-120 03SOV-123 03AOV-126 03CRD-138	Will not be tested as lines (there are 137 lines, with 31 to 38 lines per penetration, and each has the four indicated valves) are sealed by process fluid.
38A 38B 38C 38D	Control Rod Drive (above piston)	03SOV-121 03SOV-122 03AOV-127	Will not be tested as lines (there are 137 lines, with 31 to 38 lines per penetration, and each has the three indicated valves) are sealed by process fluid.
39A	RHR Cont. Spray	10MOV-31A	This valve will be tested in the reverse direction.
39B	RHR Cont. Spray	10MOV-31B	This valve will be tested in the reverse direction.
45	Drywell Pressure Sensing	16-1AOV-101A	This valve will be tested in the reverse direction.
50C	Instrumentation - Sensing DW Pressure	Various	These instrument root valves are tested during a Type A test.

TABLE 4.7-2

EXCEPTION TO TYPE C TESTS

CONTAINMENT PENETRATION	PENETRATION FUNCTION	VALVE NUMBER	LOCAL LEAK RATE TEST PERFORMED
202B	Vacuum Breaker - Reactor Building to Suppression Chamber	27AOV-101A 27AOV-101B	These valves will be tested in the reverse direction.
205	Pressure Suppression Chamber Purge Exhaust (Air or Nitrogen)	27AOV-117 27MOV-117	These valves will be tested in the reverse direction.
210A	RHR to Suppression Pool, RCIC, Core Spray Test to Suppression Pool	10MOV-16A 10MOV-21A 10MOV-34A 10MOV-167A 13MOV-27 14MOV-5A 14MOV-26A 10RHR-95A 14CSP-62A	Will not be tested as lines are water sealed by suppression chamber water.
210B	RHR to Suppression Pool, HPCI, Core Spray Test to Suppression Pool	10MOV-16B 10MOV-21B 10MOV-34B 10MOV-167B 14MOV-5B 14MOV-26B 23MOV-25 10RHR-95B 14CSP-62B	Will not be tested as lines are water sealed by suppression chamber water.
211A	RHR to Suppression Spray Header	10MOV-38A	This valve will be tested in the reverse direction.

**TABLE 4.7-2
EXCEPTION TO TYPE C TESTS**

CONTAINMENT PENETRATION	PENETRATION FUNCTION	VALVE NUMBER	LOCAL LEAK RATE TEST PERFORMED
211B	RHR to Suppression Spray Header	10MOV-38B	This valve will be tested in the reverse direction.
218	Torus Pressure Sensing	16-1AOV-102B	This valve will be tested in the reverse direction.
220	Torus Purge Inlet (Air and/or Nitrogen)	27AOV-116 27AOV-132A 27AOV-132B	These valves will be tested in the reverse direction.
221	RCIC - Vacuum Pump to Torus	13RCIC-07	Will not be tested as line is sealed by suppression chamber water.
222	HPCI - Turbine Drain Trap to Torus	23HPI-13	Will not be tested as line is water sealed by suppression chamber water.
224	RCIC - Pump Suction (Torus)	13MOV-39 13MOV-41	Will not be tested as lines are water sealed by suppression chamber water.
225A	RHR - Pump Suction, RHR to Radwaste	10MOV-13A 10MOV-13C 10RHR-729A	Will not be tested as lines are water sealed by suppression chamber water.
225B	RHR - Pump Suction	10MOV-13B 10MOV-13D 10RHR-729B	Will not be tested as lines are water sealed by suppression chamber water.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 194 TO FACILITY OPERATING LICENSE NO. DPR-59
POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT
DOCKET NO. 50-333

1.0 INTRODUCTION

By letter dated April 15, 1993, the Power Authority of the State of New York (the licensee) submitted a request for changes to the James A. FitzPatrick Nuclear Power Plant Technical Specifications (TSs). The requested changes pertain to Technical Specification Table 4.7-2, "Exception to Type C Tests," for select containment isolation valves. Specifically, the changes would (1) add system numbers to the valve identification numbers for seven control rod drive (CRD) containment isolation valves and clarify the penetration arrangement, (2) remove valves 10MOV-57 and 10MOV-67 from the table, (3) add valves 10RHR-729A and 10RHR-729B to the table, and (4) correct errors introduced in Amendment No. 143.

2.0 EVALUATION

Technical Specification Table 4.7-2 provides the containment isolation valves that are exempt from Type C local leak rate testing. The licensee has proposed several changes to the table.

First, the licensee has proposed a change to the table to add system numbers to the valve identification numbers for seven control rod drive containment isolation valves to be consistent with valve identifiers elsewhere in the TSs. In addition, the licensee has proposed a clarification to the penetration arrangements. Specifically, the change clarified the fact that each penetration is actually a cluster of 31 to 38 lines for a total of 137 lines, one inlet and outlet pair for each CRD. Further, each inlet line below a CRD piston has the four specified containment isolation valves while each corresponding outlet line above the CRD piston has the three specified containment isolation valves. The NRC has reviewed the proposed changes and finds that they are all administrative in nature to enhance the information provided in the table. Therefore, the staff finds these changes acceptable.

Second, the licensee has proposed removing valves 10MOV-57 and 10MOV-67 from the table. The valves are located on the drain line leading to the radwaste system from the "A" side of the residual heat removal (RHR) system. Removal of the valves from the table is based on a safety evaluation performed by the

licensee that concluded the valves are not containment isolation valves, as defined by the current FitzPatrick licensing basis. Specifically, the licensee determined that several normally closed manual valves located upstream of 10MOV-57 and 10MOV-67 provide boundary isolation. The staff has reviewed the Final Safety Analysis Report (FSAR), the TSS, and James A. FitzPatrick Nuclear Power Plant Safety Evaluation JAF-SE-92-033, Revision 0, dated February 4, 1992. The FSAR indicated that valves on lines that penetrate the primary containment but do not communicate with the reactor vessel, with the primary containment free space, or with the environs require at least one containment isolation valve which shall be either automatic, or locked closed, or capable of remote manual operation. Since manual isolation valve 10RHR-729A, located upstream of 10MOV-57 and 10MOV-67, is normally locked closed, it provides the isolation necessary to assure containment isolation. Therefore, 10MOV-57 and 10MOV-67 are not required for containment isolation, and the staff finds removal of the valves from the table to be acceptable.

Third, the licensee has proposed adding valves 10RHR-729A and 10RHR-729B to the table. The licensee determined that the normally closed manual valves form a part of the containment isolation boundary by isolating the branch lines out of penetrations X-225A and X-225B, respectively, leading to the RHR/Radwaste drain down lines. Since: (1) the valves provide a containment isolation function as defined by the current FitzPatrick licensing basis, and (2) the valves are excepted from Type C testing as specified in Surveillance Requirement 4.7.A.2.c.(3), the staff finds adding the valves to the table to be acceptable.

Fourth, the licensee has proposed correcting errors introduced in Amendment No. 143. Specifically, the licensee has proposed: (1) removing the Type C surveillance requirements for valves 10MOV-34A and 10MOV-34B, which were erroneously added to the table, and (2) adding a functional identifier for containment penetration X-221. Since the discharge lines for valves 10MOV-34A and 10MOV-34B terminate below the low torus water level, the requirements for exception from Surveillance Requirement 4.7.A.2.c.(3) are satisfied, and removal of the Type C surveillance requirements is acceptable. In addition, adding the functional identifier for containment penetration X-221 is an administrative change, which is also acceptable. The staff has reviewed the above changes and finds them acceptable.

In summary, the staff has reviewed the proposed changes to TS Table 4.7-2, and finds all changes to be acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (58 FR 32391). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor:
Kristine Shembarger

Date: July 29, 1993

July 29, 1993

Docket No. 50-333

Mr. Ralph E. Beedle
Executive Vice President - Nuclear Generation
Power Authority of the State of New York
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Sincerely,

Original signed by Maudette Griggs for:
John E. Menning, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

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2. Safety Evaluation

cc w/enclosures:
See next page

*See previous concurrence

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CVogan <i>CS</i>	KShembarger: <i>av</i>	JMenning <i>Y</i>	*RBarrett	<i>Carico</i>	RACapra <i>Ru</i>
7/16/93	7/16/93	7/16/93	07/15/93	7/29/93	7/29/93

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